U. S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
REGION IV

IE Inspection Report No. 50-267/79-10

Docket No. 50-267 License No. DPR-34

Licensee: Public Service Company of Colorado
P. O. Box 840
Denver, Colorado 80201

Facility Name: Fort St. Vrain Nuclear Generating Station

Inspection At: Fort St. Vrain Site, Platteville, Colorado

Inspection Conducted: May 14-19, 1979

Inspector: T. F. Westerman for 6/1/79
M. W. Dickerson, Reactor Inspector Date

Approved by: T. F. Westerman 6/1/79
T. F. Westerman, Chief, Reactor Projects Section Date

Inspection Summary

Inspection on May 14-18, 1979 (Report No. 50-267/79-10)

Areas Inspected: Routine, unannounced inspection of procedures; review of plant operations; followup on previous items of noncompliance; and followup inspector identified unresolved items. The inspection involved thirty-five (35) inspection-hours on site by one (1) NRC inspector.

Results: No items of noncompliance or deviations were identified.

422 044

7907270118

9

DETAILS

1. Persons Contacted

Public Service Company of Colorado

W. Crane, Supervisor Mechanical Maintenance
M. Ferris, Engineer Technical Services
W. Franek, Results Supervisor
R. Frost, Engineer Technical Services
*J. Gamm, Supervisor Technical Services
*T. Howard, Superintendent Operations QA
J. Liebelt, Supervisor Electrical Maintenance
*F. Mathie, Operations Manager
M. McBride, Engineering Coordinator
J. Solakiewicz, QA Engineer
*D. Warembourg, Manager Nuclear Production
G. Wiltscheck, Results Engineer

*Attended exit interview.

2. Licensee Action on Previous Inspection Findings

(Closed) Item of Nonconformance (50-267/79-04): Failure to comply with procedural requirements. The inspector examined the licensee's corrective action relative to failure of the licensee to comply with certain procedual requirements. The results of IE reports 79-04 was discussed in detail with all plant management and supervisors. Additionally instruction relative to requirements relative to review of documents before signoff and individual responsibilities for completeness was conducted.

(Open) Open item (50-267-79-08): Hydraulic Snubbers. The testing and rework of all Class I Hydraulic Snubbers is continuing. The failure rate of those tested has been about 19 out of 20 tested. The primary reason for failure has been the inability of the snubbers to initially (as found) to meet the activation and release rates. At the time of the inspection 122 snubbers had been tested and 59 of these were ready for reinstallation. Additionally twenty-two (22) snubbers with a 10 to 15 inch stroke had been sent to Wylie Laboratories for testing.

(Open) Open item (50-267/79-08): ACM installation. The ACM installation has been completed, however, the functional testing had not been completed at the time of the inspection.

(Open) Open item (50-267/79-08): Stage 3 Fire Protection. Spray Shield installation in the reactor building and the auxiliary electrical equipment

room is continuing. Inspection and cleaning of relays within the relay cabinets in the auxiliary electrical equipment room is in progress. The inspection and cleaning was necessary when some of the silicone foam ulilized in the spray shield installation was allowed to gain access into the cabinets and impinged upon some of the relays. The inspector noted that relays 50 CR 3109 and 1921781B were thus effected.

(Open) Open item (50-267/79-08): Fire Water Booster pumps. The installation of the fire water booster pumps and FCN 2651B is essentially complete. One weld remained to be examined for acceptability.

3. System 21 Instruments

The requirement for System 21 instruments to be calibrated and/or functionally tested prior to resumption of operation following the first refueling outage has been completed. The status of those instruments still uncalibrated or functionally tested at the previous inspection (50-267/79-07) was determined to be as follows:

Instruments No.	Date Calibrated Functionally Tested
LSL 21116	May 1, 1979
PS 21120	April 13, 1979
XSL 21185	April 13, 1979
XSL 21187	April 13, 1979
XPS 21189	April 13, 1979
LS 21287	April 21, 1979
PS 21358	May 11, 1979
PS 21359	May 11, 1979
LS 21385	April 7, 1979
LSH 21451	April 27, 1979
LSH 21452	April 7, 1979
LSH 2153	April 7, 1979
LS 21502	May 11, 1979

This completes the required action by the licensee for System 21 instruments. However pressure switches 21358 and 21359, although calibrated, remained to be installed pending completion of CN No. 890.

4. Review of Plant Operations

The inspector verified that the licensee has established administrative controls and checklists for returning safety related system to an operating status. Included are those systems which underwent maintenance or were disturbed during the first refueling outage. Additionally the inspector established that startup and operating procedures are being revised to reflect changes made to the facility and that the changes are being reviewed and approved as required by Technical Specifications.

No items of noncompliance or deviations were identified.

5. Procedures

a. Areas Inspected

The inspector reviewed plant procedures to verify that they had been reviewed and approved, that control of changes was in accordance with the Technical Specifications, that Technical Specification revisions have been incorporated into procedural changes, and that the content of the procedures conform to TS requirements. In addition, the content of the procedures were reviewed for their technical adequacy to control the specific operation.

The following areas were inspected by selective review of representative procedures in each area:

(1) Administrative Control

ADM-02 Administrative Procedure for Plant Security, Revision 11, February 27, 1979.

OP-02 Administrative Procedure for Licensed Operators, Revision 5, February 27, 1979.

(2) Operating Procedures

(a) Overall Plant Operating Procedures (OPOP), Revision 35, May 3, 1979.

OPOP III Startup Procedures, Revision 35, May 3, 1979.

OPOP VII Recovery from Reactor Scram, Revision 34, March 1, 1979.

(b) Standard Operating Procedures (SOP)

SOP 11 Prestressed Concrete Reactor Vessel, Revision 8, February 27, 1979.

SOP 12-01 Reactivity Control System Control Rods, Revision 7, January 2, 1979.

SOP 12-04 Orificing System, Revision 5, August 21, 1978.

SOP 31 Feedwater and Condensate Systems, Revision 19, March 22, 1979.

SOP 42 Service Water System, Revision 10, March 16, 1979.

SOP 63 Radioactive Gas Waste, Revision 11, January 26, 1979.

SOP 82 Instrument and Service Air, Revision 4, February 27, 1979.

(3) Maintenance Procedures

MP 98.1 Repair of Hydraulic Pipe Snubbers, Revision 5, May 11, 1979.

(4) Emergency Procedures

F.2 Loss of Condenser Vacuum, Revision 17, June 8, 1977.

App. F.2 Loss of Condenser Vacuum, Revision 11, April 30, 1976.

H.1 Abnormal Release to Coolant Tower Blowdown, Revision 33, September 26, 1978.

App. H.1 Abnormal Release to Coolant Tower Blowdown, Revision 11, April 30, 1976.

H.2 Abnormal Radioactive Gas Release from Plant, Revision 33, September 26, 1978.

App. H.2 Abnormal Radioactive Gas Release from Plant, Revision 11, April 30, 1976.

N. Loss of Emergency Bus, Revision 10, March 22, 1976.

App. N. Loss of Emergency Bus, Revision 12, September 24, 1976.

(5) Abnormal (Alarm) Condition Procedures

The inspector reviewed each of the procedures identified in paragraph 5.a(2) above, to determine that abnormal conditions have been provided for associated alarms.

b. Findings

The inspector's review of the above listed procedures established that Overall Plant Operating Procedure III and Standard Operating Procedures 12-01, 12-04, 31, 42 and 63 each contain deficiencies. The specific deficiencies relative to each procedure are listed below. Each was discussed with representatives of the licensee at which time it was indicated that this matter would remain open pending corrective action relative to the procedures.

(1) OPOP III - Startup Procedure

(a) Page 3, step 8 should reference SOP 12-01 instead of SOP 21-01.

(b) Page 6, step 14 should reference OPOP 1 A4 and A5.

(c) Page 13, step 3a. Delete the reference to page 113.

(2) SOP 12-01 - Reactivity Control System Control Rods

A number of references are made to Table 1 of the procedure. At present the information (Table 1) is the "Rod Withdrawal Sequence" for the initial Core and requires replacement with information relerent to the present core loading.

(3) SOP 12-04 - Orificing System

(a) Section 3.2.3.3 and Data Sheet 4. The references to Section 2.1 and 2.2 do not appear to be correct.

(b) Data Sheet 8, step 2 should refer to 2a thru 2d instead of 2a and 2b.

(4) SOP 31 - Feedwater and Condensate Systems

(a) Page 11(c) V-31106 and V-31107 should be checked open instead of closed.

(b) Page 14(d) Valve 7542 should be noted as for pump 1D.

(c) Section 2.2.2.6.(4)(a) Specify where this information is to be recorded.

(d) Section 2.2.7(3) V-41197 does not appear on the System 41 Checklist.

(e) Where the procedure is general in nature, as in Section 2.2.2.7(4) so state in the procedure.

(5) SOP 42 - Service Water System

(a) Section 2.5.1.2. The exchanger should be identified as E-51095X instead of #-5109X.

(b) Section 2.5.3.3 V-46304 incorrectly identified on Appendix D to SOP 46 as "Outlet Header drain for Bearing Water Cooler "B" instead of "C".

(c) Section 2.7.4.2 V-2133 incorrectly identified on Appendix D to SOP 46 as "D/G 1B outlet TCV-4267 Bypass instead of TCV-4268 bypass.

(d) Section 2.5.2.2(b) Valve V-45146 should be V-46146.

(e) Section 5.3.2.1 V-45221 "Fire Water to Compressor 1C," is labeled as "Emergency Cooling to Air Compressor on Appendix 1, SOP 45. This is to be clarified.

(f) In Section 5.3.2.2(b), the correct valve designation is questioned (V-4299 or V-4229)

(g) Section 5.5.1, V-46321 and V-46322 are for 1A not 1D.

(h) Section 5.5.4, V-46492 and V-46494 are for ID and IA.

(i) Section 5.6.2.2, V-42371 is bypass around TCV-4235 instead of 4234.

(j) Section 5.7 refers to SOP 48 which is the IACM and is no longer in use.

(6) SOP 63 - Radioactive Gas Waste

(a) Section 2.3.5(3) 2nd Sentence, clarify the requirement for filling ie: + 0.5, -0.

(b) Section 2.5.2 Note refers to Page 12 instead of page 14.

(c) Section 5.2.3.(5) Reference to SOP 93-30 should be SOP 93-03.

The inspector had no additional Questions in this area.

6. Exit Interview

An exit interview was conducted with representatives of the licensee on May 18, 1979, at the conclusion of the inspection. At the interview, the inspector discussed the findings indicated in the previous paragraphs. The licensee acknowledged these findings.