MAY 2 3 1979

DOCKET NUMBER

Secretary of the Commission U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Attention: Docketing and Service Branch

Re: Docket No. PRM-20-13, Petition for Rulemaking by Mr. Victor E. Anderson

Dear Sir:

The petitioner proposes to amend the Nuclear Regulatory Commission regulation "Standards for Protection Against Radiation," 10 CFR 20, to require that Health Physics personnel be certified by the Commission. The proposed amendment also provides for certification of Health Physicists at five levels.

The petitioner also states that there are no official legal standards, only a regulatory guide, for radiation managers and some ANSI Standards for Technicians.

The primary reason for suggesting that amendment 10 CFR 20.600 be added is to prevent management from placing pressure on Health Physics personnel to engage in bad practice.

The American Board of Health Physics submits the following comments to this petition:

The American Board of Health Physics was established on October 29, 1959, and for a period of almost 20 years has provided a certification program for Health Physicists by examination. The certificate indicates that its holder has completed certain requirements of study and professional experience, which the Board considers to constitute an adequate foundation in health physics and has passed an examination designed to test the individual's competence in this field.

It should be recognized that the certificate awarded by the Board is not a license and, therefore, d as not confer a legal qualification to practice nealth physics. Since the inception of this program, the ABHP has certified a total of over 640 individuals as of January, 1979.

Under the direction of the ABHP, a National Registry of Radiation Protection Technologists was established and incorporated on April 12, 1976. The objectives of the NRRPT were to develop standards and procedures for the registration of Radiation Protection Technologists, to institute examinations and to register qualified individuals after demonstrating technical competence by examination. The NRRPT has certified a total of over 115 individuals as of January 1979.

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The purposes of the NRRPT Board is to:

. . . .

1. Evaluate the standards and advance the profession of applied Health Physics by encouraging its study and improving its practice.

2. To encourage and insist on the highest standards of professional ethics and integrity in the practice of radiation protection.

3. To determine the competence of specialists in radiation protection and to arrange, control and conduct investigations and examinations, to test the qualifications of voluntary candidates for registration by the Board.

 To grant and issue certificates in the field of applied Health Physics to voluntary applicants and maintain a registry of those who qualify.

The certificate of registration indicates that its holder has completed certain requirements of study and competence which the Board considers to constitute an adequate foundation in radiation protection and has passed an exam designed to test the individual's knowledge in this field.

It should be recognized that the certificate of registration by the Board is not a license and, therefore, does not confer a legal qualification to practice Health Physics.

In September 1978, after consideration for over three years, the ABHP decided to offer a Specialty Certification in Power Reactor Health Physics, in addition to the Comprehensive Certification the Board appointed a Power Specialty Examination Panel to prepare, administer and grade the Power Reactor Specialty Examination under the guidance and approval of the loard.

The ABHP feels that at present these programs are more than adequate to certify and register professionals and technologists in the field of Health Physics. These programs have received both National and International recognition, and have been quite effective. Duplication of this effort by a Federal Regulatory Agency would serve no useful purpose. As a result, the ABHP feels the petition is without merit and should be denied.

The Board would also like to submit the following information and documentation to the NRC dealing with the current ABHP certification and NRRPT Registration programs.

(1) Complete set of applications and handout materials required of prospective candidates in order to qualify for the ABHP Certification Examination.

(2) Complete set of application and handout materials required of prospective candidates in order to qualify for the NRRPT Registration Examination.

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(3) ABHP Item Classification Scheme.

(4) NRRPT Item Classification Scheme.

(5) ABHP Examination Preparation Guide, 1979, with addendum for Power Reactor Health Physics Specialty Examination.

(6) NRRPT Examination Preparation Guide, 1978.

(7) American Board of Health Physics Board and Panel Members.

(8) National Registry of Radiation Protection Technologists Board and Panel Members.

(9) The ABHP Continuing Certification Program.

(10) NJ.RPT Brochure.

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(11) 'aper entitled "An Inside View of the American Board of Health Physics Programs Report on ABHP Examination Program Part I 1968-1975, Michael S. Terpilak, Health Physics Society Meeting, July 16, 1975, Buffalo, New York.

The Board certainly hopes that these comments will be useful to the NRC and appreciates the opportunity to comment on this proposal.

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Michael S. Terpilak Chairman, ABHP

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AMERICAN BOARD OF HEALTH PHYSICS a to be to be to be to be to be to be CONTRACTOR 362 004

### AMEPICAN BOARD OF HEALTH PHYSICS HISTORY

Shortly after its organization, the Health Physics Society established a Committee to study the need for certification of health physicists and to develop plans for certification if this appeared to be desirable. After an intensive study, the Committee recommended that an American Board of Health Physics be established to develop standards and procedures, to examine candidates, and to issue written proof of certification to individuals who have satisfied the requirements established by the Board. The Board of Directors of the Society decided that these recommendations had merit and appointed a temporary American Board of Health Physics on November 8, 1958.

The temporary ABHP developed a set of .....um requirements for certification after carefully reviewing the professional background of 100 selected individuals believed to be representative of those recognized as competent health physicists. These minimum requirements were submitted to the membership of the Society for comment. At the Society's Annual Meeting in June 1959, the matter was discussed in an open meeting and there was general support for the plan. The Board of Directors of the Society formally established the American Board of Health Physics by approving an amendment to the By-Laws of the Society on October 29, 1959.

The ABHP was incorporated in the State of New York on December 1, 1960. Provision was made for organizations other than the Health Physics Society to be represented on the Board.

The Board has seven members. Five are sponsored by the Health Physics Society, one by the American Association of Physicists in Medicine, and one bit the American Public Health Association. Each member serves a 5-year term.

An Examination Panel consisting of Certified Health Physicists appointed by the Board prepares, administers, and grades the written certification examination under the guidance and approval of the Board.

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#### PURPOSES OF THE BOARD

- First: To elevate the standards and advance the profession of health physics by encouraging its study and improving its practice.
- Second: To encourage and insist on the highest standards of professional ethics and integrity in the practice of health physics.
- Third: To determine the competence of specialists in health physics and to arrange, control, and conduct investigations and examinations to test the qualifications of voluntary candidates for certificates to be issued by the Board.
- Fourth: To grant and issue certificates in the field of health physics to voluntary applicants and to maintain a registry of holders of such certificates.

#### MEANING OF CERTIFICATION

The certificate indicates that its holder has completed certain requirements of study and professional experience, which the Board considers to constitute an adequate foundation in health physics and has passed an examination designed to test his competence in this field.

It should be recognized that the certificate awarded by the Board is not a license and, therefore, does not confer a legal qualification to practice health physics.

#### PROFESSIONAL RESPONSIBILITIES OF CERTIFIED HEALTH PHYSICISTS

In achieving certification, the Certified Health Physicist recognizes and assumes responsibilities due the profession of health physics.

In order to maintain his technical competence, the Certified Health Physicist has a commitment to remain active in the field of health physics and acquainted with the scientific, technical and regulatory developments in his field.

In order to uphold the professional integrity of health physics implied in this certification, his relations with others, including clients, colleagues, governmental agencies, and the general public shall always be based upon

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and reflect the highest standards of professional ethics and integrity.

The Certified Health Physicist shall represent himself as an authority only in those areas in which he is considered expert by his peers.

#### GENERAL REQUIREMENTS

Requirements for Candidates for certification are as follows:

- ACADEMIC. The Applicant must have a Bachelor's Degree in a physical science or in a biological science with a minor in physical science. In exceptional cases, persons who have demonstrated adequate knowledge of health physics but who are deficient in these academic requirements may, at the discretion of the Board, be permitted to substitute experience for academic requirements.
- EXPERIENCE. An applicant must have at least six years of FTE (full-time equivalent) professional experience in health physics. At least three years of the experience must have been in applied radiation protection work. Additional education may be substituted for up to a maximum of 2½ years of experience as follows:

Type of Study	of study or degree	credit for experience
General-related to HP	1	1/2
General-related to HP	2 or MS	1
General-related to HP	PhC	2
Health Physics	1	1
Health Physics	2 or MS	11/2
Health Physics	PhD or ScD	21/2

An applicant may not claim professional experience for an advanced degree and work experience for the same period. For example, if an applicant attends night school for 4 years resulting in an MS degree and during the same period he is employed as a health physicist, he may claim four years professional experience, but may not claim an additional year of experience for his MS.

 PROFESSIONAL. Each applicant must be engaged in the professional practice of health physics a substantial portion of his time. Reference statements are reguired from the applicant's supervisor (If appropriate)



and from at least two other individuals who are professionally qualified to evaluate the applicant's ability in health physics. It is recommended (but not required) that at least one reference be a health physicist already certified b; the ABHP.

4. WRITTEN REPORT. The Board, after examination of the application for certification, may request reports on radiation protection evaluations made personally by or under the supervision of the applicant. Each applicant must be capable of making a satisfactory evaluation on several installations or operations involving possible radiation hazaros of which those listed below are examples:

- a. Radiographic installation-industrial medical
- b. Fluoroscopic installation
- c. Therapy installation
- d. Radionuclide laboratory
- e. Air and water sampling and environmental survey
- f. Nuclear fuel processing plant
- g. Nuclear reactor
- h. Major decontamination operation
- i. Particle accelerator
- 5. EXAMINATION. Written examinations will be mandatory; oral examinations will be at the discretion of the Board. The written examination has two parts. Part I determines competence of the applicant in fundamental aspects of health physics and Part II determines his competence in practical health physics topics. The examination must be taken within two years of notification of eligibility, or a new application must be submitted.

#### EARLY ADMISSION TO WRITTEN EXAMINATION

Applicants are permitted to take Part I of the written examination if they have fulfilled the academic requirements for the MS Degree in Radiation Safety or have two years of professional experience at the time of the examination. Applicants must meet all the requirements listed in the preceding section before being admitted to Part II.

The purpose of early admission to Part I of the examination is two-fold: (1) to allow the recent graduate an opportunity to demonstrate his competence in the fundamentals of health physics at the beginning of his career, and (2) to encourage younger health physicists to pro-

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ceed toward certification. Applicants who successfully complete this step in the examination procedure will be required to take only Part II of the written examination when they later apply for regular certification.

#### APPLICATION AND FEE

Application for examination must be made on the prescribed form which is available from the Chairman. Applications should be filed with the Chairman at least two months before the date of the examination. Certification fees are as follows:

Certification Step	Fee
Application to take Part I of written examination	\$50
Application for Regular Examination to take Parts I and II of the written examination together	\$100
Application to take Part II of written examination only	\$50
Charge for certification plaque	\$15

Re-examination fees following failure of the exam are the same as the original application fee schedule above. \*Effective January 1, 1977.

#### EXAMINATIONS

Examinations are usually given once a year-at the time of the Annual Meeting of the Health Physics Society. They are held at the location of the Society's meeting and at other selected locations where the demand warrants.

Permits are required for entry into the examination room. No reference material may be brought into the room.

#### **RE-EXAMINATIONS**

A candidate who fails his first examination may be admitted to a second examination after one year. A candidate who fails to appear for re-examination within two years must submit a new application.

After a second failure, a new application must be filed. The candidate must also submit evidence of substantial additional study before he may take the examination for the third time.

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### FINAL ACTION OF THE BOARD

The final action of the Board is based on its evaluation of the applicant's total professional record, i.e., his training and experience, the achievements he has obtained in health physics and related fields, the maturity of his judgment, and the ethical nature of his professional conduct its indicated by his associates and peers, and often the results of oral interviews as well as the written examinations. Anyone meeting the education and experience requirements and who is practicing health physics in a competent and ethical manner is strongly urged to apply to the Board for admission to the written examination is a necessary but not a sufficient requirement, persons who are admitted to and who perform well on the examination usually receive certification by the Board.

### REVOCATION OF CERTIFICATE

Certificates may be revoked for actions considered by the Board to be in violation of the statement "Professional Responsibilities of Certified Health Physicists." Any person for whom such action is contemplated shall have the right of appearance before the Board.

### CHANGE IN REQUIREMENT

Current requirements, procedures, and 'ees of the American Board of Health Physics are described in this brochure. These are subject to change without notice; however, changes will be published before their effective date whenever practical. No changes will be retroactive.

#### CORRESPONDENCE

All correspondence to the American Board of Health Physics should be sent to:

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Michael S. Terpilak, Chairman American Board of Health Physics HEW, PHS, FDA, BRH (HFX 460) 12720 Twinbrook Parkway Rockville, MD 20857

"Certified Health Physicist" is a professional title. It is a recognition of professional competence conferred by the American Board of Health Physics, an organization founded in 1960 to establish standards of education, experience and competence in the practice of health physics. The certificate indicates that its holder has completed certain requirements of study and professional experience which the Board considers to constitute an adequate foundation in health physics\*, and that he has passed a comprehensive examination designed to test his competence in this field.

The Certified Health Physicist today is ably Solsting government and industry, and the research and health professions in achieving the maximum benefits of the nuclear age with a record of safety that is unsurpassed. With the

Requirements for application for certification include a bachelor's degree in science and/or engineering and six years of responsible professional experience in health physics.

commitment of the nation to nuclear energy for electrical power production, and with the increased use of radiation producing equipment in medicine and industry the role of the Certified H.P. takes on new and greater significance. The industry and indeed the general public, expects nothing less than the high standard of safety that the nuclear field has enjoyed to date. Certified Health Physicists are already applying their expertise to meet this necessary objective both effectively and economically.

Recognizing the demonstrated professional competence of CHP's government agencies, industry and other organizations require Certified Health Physicists in certain key radiation protection positions.

Within their commitment to keep radiation doses to employees and the public as low as practicable, CHP's provide the following services:

## Monitoring Programs

Continually assess the adequacy of radiation protection control facilities and procedures, by thorough monitoring programs as part of the facility operation.

## Radiation Safety

### Analyses

Perform analyses on new or existing facilities to determine and minimize their impact on employee radiation dose and the public health and safety.

## Shielding and Containment

Designs to protect personnel and the general public from radiation exposure and the disposal of radioactive materials.



### Dosimetry

Detect, evaluate and control radiation exposures from external sources through portable and fixed radiation detection devices and to detect, evaluate and control radiation doses from internal sources through calculations based on bioassay analyses and whole body counters.



### Instrumentation

Select, calibrate, and interpret results from:

- effluent monitors for liquid and gaseous wastes
- 2. portable survey meters
- laboratory detectors, ranging from surface contamination counters to multichannel spectrometers
- area monitors for direct radiation and airborne radioactive materials

## Plans and

## Procedures

Develop and keep current the plans and procedures necessary to control on-and off-site exposures.

## Emergency Planning

Participate in and guide the development of appropriate plans to minimize the consequences of radiation accidents.



## Environmental Evaluation

Design and carry out sampling and analytical programs to detect minute levels of all radionuclides in the environs of nuclear facilities (flora, fauna, soil, water, air). Evaluate the possible short and long-term consequences to man and his environment from these levels or postulated levels.



## Waste Management

Programs to control releases of radioactivity and ensure the safe storage of radioactive wastes.



## Manage Regulatory Programs

At all levels within government and industry to ensure compliance with Federal, State and local regulations.

## Educational Programs

Provide general and specific training or orientation in all aspects of radiation safety.



## TRAINING

A Certified Health Physicist on your staff or as a consultant will provide a competent and professional approach to radiation protection and control problems. For a Registry of Certified Health Physicists or further information, please write to the Chairman of the American -Board of Health Physics.

### ANNOUNCEMENT

#### AMERICAN BOARD OF HEALTH PHYSICS

The continued efforts of the American Board to restrict expenses to a bare minimum have been successful as evidenced by the fact that the amount of money realized through the examination fees has nearly met the expenses of the program during the past year. However, during the last Board meeting, September 26-27, 1978, it was recognized that there are some increased expenses resulting from the need to expand and modify the examination questions in Part I; increased fee assessment from the Professional Examination Service and additionally, the initiation of a new specialty certification program for power plant reactor health physicists. In any event, it was felt by the Board that it would be necessary to raise the examination and certification plaque fees for new candidates, effective January 1, 1979, in order that the program be self-supporting.

Effective January 1, 1979, the fees will be as follows:

CERTIFICATION STEP		FEE
Application to take Part I of the written examination		\$ 75
Application for regular examination to take Parts I and II of the written examination together		\$150
Application to take Part II of written examination only		\$ 75
Charge for certification plaque		\$ 25
Please send application forms to:		
Michael S. Terpilak, Chairman American Board of Health Physics HEW, PHS, FDA, BRH (HFX-466) 12720 Twinbrook Purkway Rockvillo, Maryland 20857	362	014

### DEFINITION OF HEALTH PHYSICS\*

Health Physics is a profession devoted to the protection of man and his environment from unwarranted radiation exposure. A health physicist is a person engaged in the study of the problems and practices of providing radiation protection. He is concerned with an understanding of the mechanism of radiation damage, with the development and implementation of methods and procedures necessary to evaluate radiation hazards and with providing protection to man and his environment from unwarranted radiation exposure.

\*Of\_icially adopted by the Health Physics Society in 1959.

### REQUIREMENTS FOR ADMISSION TO PART I OF THE WRITTEN EXAMINATION

The ABHP has announced the formal establishment of a special program to permit younger health physicists to complete an initial step in the certification procedure. Under this arrangement, radiation protection personnel who have:

- Received a Bachelor's Degree in a physical science or a biological science with a minor in the physical sciences, and
- completed a minimum of two additional calendar years of professional experience or graduate training in health physics,\*

will be permitted to take Part I of the written examination.

In permitting a candidate to take this step, the Board makes no commitment concerning the candidates eligibility to complete additional steps in the certification procedure at a later date. This will de and upon performance on Part I and results of a thorough review of past training, professional experience, and statements contained in references submitted by candidate of time of application for regular certification.

The fee for admission to Part I of the written examination is \$75.00 and applications should be submitted on the form required for regular certification. To designate that they are applying for Part I only of the written examination, applicants should write the words, "Part I" in the uppper right hand corner (above the date) on the first page of the form. Such applicants should also note that they are required to submit only one reference statement in support of their application (rather than three as required for candidates applying for regular certification).

Normally, the written examination will be given in June of each year on the Monday of the week of the Annual Meeting of the Health Physics Society. The deadline for submission of applications is April 1.

\* Applicants are eligible immediately after meeting the requirements for an MS degree in health physics. Also, individuals who have successfully completed a Bachelor of Science program in Health Physics and who have at least one year of practical (professional level) experience.



## AMERICAN BOARD OF HEALTH PHYSICS

## Application For Certification

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## Present employment. Describe in your own words. (Dc not use official job descriptions.)

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Date Assigned to Position	Name of Employer	Place of Employment:	Name and Title of Immediate Supervisor
Exact Title of Present Position			
Description of work. Include major re	esponsibility and specific fields and	indicate percent of time in Health Phy	sics work

10. Previous Employment. (Start with most recent position and work back. Emphasize those portions of work that are Health Physics or closely related.)

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 Industrial Radiographic Installations	Reactor Facilities	
 Medical Radiological and Fluoroscopic Installations	 Chemical Separations Plants	
 . Therapy Installations	 Particle Accelerators	
 Isotope Laboratories	 Complete Hazards Evaluation	
 Environmental Monitoring	 Major Decontamination Operation	
Other (specify)	 Other (specify)	

362 019

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certify that the my faisification	i statements above (including any attachments I have submitted hereto) are, to the best of my knowledge, accurate, and I understand to of information in this application will be cause for rejection of the application or withdrawal of a certification already made.

A non-returnable application fee of \$100 must accompany the application. An additional fee of \$15,00 will be charged when and in certification is approved, in addition the Board serves notice to all applicants that, if future operations of the Board require it, some system of annual dues from all health physicists previously certified by the Board may become necessary. Make check payable to AMERICAN BOARD OF HEALTH PHYSICS.

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Mail Application to:		والمراجع والمعارية		
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## A MERICAN BOARD OF HEALTH PHYSICS

## Application For Certification

	DATE	
NSTRUCTIONS:	C Initial Application	Part 1, only
1. Type or print in block capitals.	C Re-application	C Regular Certification
2. Submit in duplicate.		
3. If space is inadequate for any answer, use extra sheet of		
paper and number items to correspond with items as insted.		
		Date
Name	Citizenship	2. Birth
(last) (first) (middle)		
Home Address		
Business Address		
Hon	ne Telephone Number:	
Busi	iness Telephone Number	
Academic Degrees Attained:		
	Years of	
Institution Major	Minor Full Attend.	Degree
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#### Present employment. Describe in your own words. (Do not use official job descriptions.)

9.

Date Assigned to Position	Name of Employer:	Place of Employment	Name and Title of Immediate Supervisor
Exact Title of Present Position			
Description of work. Include major	responsibility and specific fields and	indicate percent of time in Health Phi	vsic) work
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10. Previous Employment. (Start with most recent position and work back. Emphasize those portions of work that are Health Physics or closely related.)

Dates of Employment	Name of Employer:	Place of Employment
From: To:		
Exact title of position.		
Description of work. Include	major responsibility and specific fields and indic	ate percent of time in Health Physics work :
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Environmental Monitoring

Other (specify)

Other (specify) 362 023

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a. Mi	dals, Citations, or other awards:
. Co	nmittee Activities:
. Jo	rnal Publications and Books:
. Sp	eches and Lectures to outside organizations (last two years)
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Profe Healt refe	sional References: name and address of at least ( ) persons other than your supervisor who are qualified to evaluate your Physics competence. (If possible, at least one reference should be a Certified Health Physicist; do not use a Board member as ence.,
certi ny fa	v that the statements above (including any attachments I have submitted hereto) are, to the best of my knowindge, accurate, and I understand t ification of information in this application will be cause for rejection of the application or withdrawal of a certification already made.
	(Signature in ink.)
and the second se	returnable application fee of \$100 must accompany the application. An additional fee of \$15,00 will be charged when and

Mail Application to:

362 024

		CONFERENTIAL PROFESSIONAL REFERENCE FORMS
		Michael S. Terpilak, Chairman
eas	e ret	urn promptly to: American Board of Health Physics
		HEW, PHS, FDA, BRH (HFX-460)
		12720 Twinbrook Farkway
oli	cant'	s Name: Rockville, Maryland 20857
	How 1	long have you known the applicant?ears.
	Wh a t	has been the nature of your association?
	a.	Do you know him personally? Yes No
	Ъ.	Are you in a position to evaluate his technical carabilities?
		YesNo
	Desci	ribe briefly your impression of the work the applicant does.
		Constraints and the second statements of the second statement of the second statement of the second stat statement of the second statement of the s
	a.	Is the work primarily technical in scope? Yes No
	ь.	How and by whom are his decisions used?
	~	What type of problems does he have to face?
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	d.	what are his responsibilities in case of energenetics
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•	How	well do you think the applicant does the work assigned to him.

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5.	What	specific	and notewo	orthy	accom	plishments.	if	any,	has	he	ma	e	in
	the	radiation	protection	1 fiel	1d?								

a.	How well does he work with others?
b.	How effectively does (or would) he perform as a consultant?
What	is your estimation of:
a. b.	His professional ethics?
Do y tifi	ou have any reservations about recommending the applicant for cor- cation? Yes No (If yes, please explain)
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re aut	Michael S. Terpilak, Chairman turn promptly to: American Board of Health Physics HEW, PHS, FDA, BRH (HFX-460) 12720 Twinbrook Parkway Rockville, Maryland 20857 Application No.
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d.	What are his responsibilities in case of emergencies?
e.	How much supervision does he have and exercise?
How	well do you think the applicant does the work assigned to him?

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5.	What	specific	and notewor	thiy acc	omplishments,	if	any,	has	he	made	ín
	the t	radiation	protection	field?							

What limitations, if any,	does the	applicant have	which might adversely	r
influence his capacity to	practice	health physics	on a responsible pro-	ŕ
fessional level?				

a. How well does he work with others?

b. How effectively does (or would) he perform as a consultant?

7. What is your estimation of:

- a. His honesty?
- b. His professional ethics?

 Do you have any reservations about recommending the applicant for cartification? Yes No (If yes, please explain)

Printed Name:	Title:	
Signature:	Adress:	
Data		

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Please return promptly to:	Michael S. Terpilak, Chairman American Board of Health Physics HEW, PHS, FDA, BRH (HFX-460) 12720 Twinbrook Parkway	
Applicant's Name:	Rockville, Maryland 20857 Application No.	

How long have you known the applicant? years.

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2. What has been the nature of your association?

- a. Do you know him personally? Yes\_\_\_\_ No\_\_\_\_
- b. Are you in a position to evaluate his technical capabilities? Yes No\_\_\_\_\_

3. Describe briefly your impression of the work the applicant does.

a. Is the work primarily technical in scope? Yes\_\_\_\_ No\_\_\_\_

b. How and by whom are his decisions used?

c. What type of problems does he have to face?

d. What are his responsibilities in case of emergencies?

e. How much supervision does he have and exercise?

4. How well do you think the applicant does the work assigned to him?

\*Note: If you wish to make additional comments, please include them on a separate sheet.

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5.	What	specific	and	notewor	thy	acco	omplishments,	11	any,	has	he	made	in
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rell do you think the applicant does the work assigned to him?

\*Pote: If you wish to make additional comments, please include them on a separate sheet.

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e.	How much supervision does he have and exercise?
How	well do you think the applicant does the work assigned to him?

\*Note: If you wish to make additional comments, please include them on a separate sheet.

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	d.	What are his responsibilities in case of emergencies?
	e.	How much supervision does he have and exercise?
	How	well do you think the applicant does the work assigned to him?

\*Note: If you wish to make additional comments, please include them on a separate sheet.

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5.	What	specific	and notewor	rthy	accomplishments	, if	any,	has	he	made	in
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What limitations, if any, does the applicant have which might ac influence his capacity to practice health physics on a responsib fessional level?								
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b.	How effectively does (or would) he perform as a consultant?							
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nature	Adress:							
The remainder of this atttachment forthcoming at a later date.

# AMERICAN BOARD OF HEALTH PHYSICS ITEM CLASSIFICATION SCHEME

	FUN	VEN ALS	No. of ABH Bank Item	P
	01	Courses	24	
	02	Units	14	
	03.	Atomi Structure		
	04.	Decay	2	
	05.	Interaction of Radiation		
		with Matter	28	
	06.	Radiobiology	24	
			92	
02.	MEAS	UREMENT		
	01.	Personnel Dosimetry	7	
	02.	Bioassay and Whole Body Counting	6	
	03.	Instruments	24	
	04.	Calibration	4	
	05.	Measurement of Radiation	23	
	06.	Statistics		
	07.	Radiochemistry and Sample		
		Preparation	5	
	08.	Dose Estimates	76	
03.	OPER	ATIONAL HEALTH PHYSICS		
	01.	Laboratory Design	2	
	02.	Shielding and Equipment		
		Design	23	
	03.	Contamination Control	4	
	04.	Surveys and Inspection	3	
	05.	Waste Processing		
	06.	Emergency Response	2	
	07.	Criticality Controls	2	
	08.	Accelerator Safety	1	
	09.	Reactor Health Physics	3	
	10.	Environmental Surveillance	12	
	11.	Waste Disposal	6	
	12.	Hazards Analysis		
	13.	Medica. Health Physics	67	
04.	HEAD	LTH PHYSICS ADMINISTRATION		
	01	Standards, Guides and		
	01.	Regulacions	39	
	02	Medico-legal Aspects		
	03	Data Evaluation	1	
	04	Emergency Planning	2	
	05.	Public Relations		
	06.	Procedures		
			42	
		Total Trems		77

# AMERICAN BOARD OF HEALTH PHYSICS

ITEM CLASSIFICATION SCHEME

		No. of Items In Process Ready for Outside Reviewer	No. of Reviewed Items ready for Carding
01.	FUNDAMENTALS		
	01. Sources 02. Units 03. Atomic Structure 04. Decay	2 2 5 3	5 4 2 5
	05. Interaction of Radiation with Matter 06. Radiobiology	2	10 1
02.	MEASUREMENT		
	01. Personnel Dosimetry 02. Bioassay and Whole Body		3
	03. Instruments 04. Calibration	1	1 1 7
	06. Statistics 07. Radiochemistry and Sample	1	1
	08. Dose Estimates		ĩ
03.	OPERATIONAL HEALTH PHYSICS		
	<ol> <li>Laboratory Design</li> <li>Shielding and Equipment Design</li> <li>Contamination Control</li> <li>Surveys and Inspection</li> </ol>	2	2 6 2
	<ul> <li>05. Waste Processing</li> <li>06. Emergency Response</li> <li>07. Criticality Controls</li> <li>08. Accelerator Safety</li> </ul>	1	2 4
	09. Reactor Health Physics 10. Environmental Surveillance 11. Waste Disposal 12. Hazards Analysis 13. Medical Health Physics	1	1
04.	HEALTH PHYSICS ADMINISTRATION		
	Ol. Standards, Guides and Regulations O2. Medico-regal Aspects O3. Data Evaluation O4. Emergency Planning O5. Public Relations O6. Procedures	3 1 1 25	5
			Total 90

This attachment will be forthcoming at a later date.

# American Board Of Health Physics

Addendum to Examination Preparation Guide for

Power Reactor Specialty Examination

Candidates taking the examination for Power Reactor Specialty Certification take the same Part I examination as candidates for the comprehensive certification.

Candidates taking the examination for Power Reactor Specialty Certification take a special Part II emamination. The Power Reactor, Part II, examination is made up of two sections. In section 1, there are ten questions which require short answers. These questions may be multiple choice, or fill-in questions or may require one or two sentence answers. The questions are worth two points each, and are designed to require about six minutes each to read and answer. Candidates must answer all ten questions. In section 2, there will be seven essay/calculation problems similar to those on Part II of the comprehensive examination, but specific to power reactor health physics. The questions are worth ten points each and are designed to require about 30 minutes each to read and answer.

Candidates must answer five of the seven questions. Subjects which may be covered in Part II of the Power Reactor specialty examination are:

Technical Administration Professional Judgement Design Review Plant Systems ALARA Radioactive Material Control Radwaste Management Emergency Planning Procedures Training Regulations and Standards Medical-Legal Aspects Guides and Limits Shielding Radiation Measurement Contamination Control

Air Sampling Protective Clothing and Equipment Respiratory Protection Instrument Selection, Operation and Calibration (includes survey, effluent monitors and counting room instruments) Decontamination Personnel Dosimetry Bioassay and Uptake Analysis In-plant Dose Assessment Environmental Off-site Dose Projection Transportation Current Topics

Grading criteria for the Power Reactor Specialty examination . The same as those for the comprehensive examination.

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#### Addendum to Examination Preparation Guide

The subject content of Part I of the examination is changed to eliminate questions in specific areas of expertise. The goal is to have Part . of the examination cover the more fundamental knowledge that all professional health physicists are expected to know. Therefore, in Section 4 (page 4) of the Guide, the following topics should be deleted from Category 3, Operational Health Physics:

h. Accelerator safety

i. Reactor health physics

m. Medical health physics

Inquiries for further information should be directed to:

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Telephone: (301) 443-3426

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# EXAMINATION PREPARATION GUIDE

American Board of Health Physics

# EXAMINATION PREPARATION GUIDE

# American Board of Health Physics

# EXAMINATION PREPARATION GUIDE American Board of Health Physics

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#### Section 1

#### Message to Candidates

This guide will help you prepare for the ABHP certification examination. However, use of the Guide by itself will not be adequate preparation for the exam. Successful candidates usually start their preparation months before the test. Preparation should include a careful review of health physics fundamentals and then review of applied aspects of health physics in several of the specialty areas. The suggested study references in Section 7 will help guide you to some of the pertinent information. Joining either a formal or informal study group (particularly those that continue over a period of months) can assist you by forcing a systematic review of various topics and by exposing you to the knowledge of people expert in subjects which you are not familiar with.

The Board warns against approaching the exam in a casual fashion. We find that most unsuccessful candidates did not prepare adequately. In contrast, the successful candidates have usually planned and followed a comprehensive study program.

Because candidates credentials are reviewed carefully, the Board feels that all applicants declared eligible to take the examination have a good probability of passing. You can avoid the disappointment of poor performance by recognizing from the start that the exam will be a rigorous test of your professional knowledge. Your grade will represent, for the most part, the thoroughness of your preparation.

Now that you know the key to good performance on the examination, the Board wishes you success in achieving certification. Part II questions and problems are designed to test judgment, the ability to analyze and organize complex problems, and the use of practical skills at a high professional level.

Constants needed for the solution of numerical problems are provided. Logarithm and exponential tables are also made available to examinees. Standard slide rules and non-programable calculators may be used during the exam, but so-called "health physics" slide rules are not permitted.

Part II of the exam is made up of new questions each year, so copies of old exams are available. (Copies of the six most recent exams are included in Section 5 of this Guide.)

Further information about the certification program may be obtained from the chairman of the American Board of Health Physics. Please write to:

> Mr. Bryce L. Rich Allied Chemical Corp. 550 2nd Street Idaho Falls, Idaho 83401

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#### Section 2

#### Content of the Examination

#### The examination has two parts.

Part I is made up of 150 multiple choice questions, divided into three general categories: Fundamentals, measurements, and operational health physics. (A more detailed breakdown by subject matter is given in Section 3 of this Guide.) Each question has five answers, and each of the answers is a plausible answer. Selecting the proper answer requires thorough knowledge of the subject matter. For example, in questions that require calculations, answers other than the correct one are obtained by making some of the common calculational errors. Three hours are allowed to answer Part I (given in the morning of the examination day). Not all of the questions in Part I are replaced each year. As a consequence, this part of the examination is held in strict confidence and copies of past exams are not distributed. Section 4 of this Guide gives some typical Part I questions.

Part II is an essay type exam which is made up of sixteen questions. The candidate may select any seven of the questions to answer, and has four hours in which to complete Part II (given in the afternoon of the examination day). Part II contains four general questions which cover topics such as dosimetry, shielding, emergency response, instrumentation, effluent monitoring, waste disposal, air sampling, meteorology, radiation biology, standards and regulations, and topical subjects. The exam also includes two questions on the health physics aspects of each of the following specialty areas:

Accelerators

Environmental

Fuel Cycle (mining, milling, fuel fabrication and fuel reporcessing) Medical

Power Reactors

University

Under each specialty area, one of the two questions is specific to the specialty area to allow the specialist to demonstrate his experience and ability; while the other question is kept more general so a person without detailed experience in that specialty, but who has studied in the specialty, should be able to answer it.

- p. Data Evaluation
- q. Emergency Planning
- r. Public Relations
- s. Procedures

14 A

t. Non-ionizing Radiation

#### Section 3

#### Part I - Topics Covered

Part I of the exam is broken down into three general categories. The number of questions in each category and the subjects covered in each category are:

- 1. Fundamentals 50 questions
  - a. Sources
  - b. Units
  - c. Atomic Structure
  - d. Decay
  - e. Interaction of Radiation with Matter
  - f. Radiobiology
- 2. Measurements 30 questions
  - a. Personnel Dosimetry
  - b. Bio-assay and Whole Body Counting
  - c. Instruments
  - d. Calibration
  - e. Measurement of Radiation
  - f. Statistics
  - g. Radiochemistry and Sample Preparation
  - h. Dose Estimates
- 3. Operational Health Physics 70 questions
  - a. Laboratory Design
  - b. Shielding and Equipment Design
  - c. Contamination Control
  - d. Surveys and Inspection
  - e, Waste Processing
  - f. Emergency Response
  - g. Criticality Controls
  - h. Accelerator Safety
  - i. Reactor Health Physics
  - j. Environmental Surveillance
  - k. Waste Disposal
  - 1. Hazards Analysis
  - m. Medical Health Physics
  - n. Standards, Guides and Regulations

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o. Medical-Legal Aspects

-4-

Which one of the following statements concerning radioactive decay is correct?

- Secular equilibrium exists when the decay constant of the daughter is slightly greater than that of the parent.
- In secular equilibrium the activity of the daughter is inversely proportional to that of the parent.
- In transient equilibrium the activity of the daughter is less than that of the parent.
- 4. Equilibrium exists if the half-life of the daughter is shorter than that of the parent.
- Transient equilibrium exists if the half-life of the parent is very much longer than that of the daughter.

In tissue, fast neutrons lose from 80% to 95% of their energy in interactions with:

- 1. Sodium
- 2. Nitrogen
- 3. Oxygen
- 4. Hydrogen
- 5. Carbon
- 6.

5.

4.

An investigator has received some Zirconium-95 ( $T_{\frac{1}{2}} = 65$  days) for use in a long-term study. He finds the Zirconiúm to be contaminated with Cobalt-60 ( $T_{\frac{1}{2}} = 5.24$  years) such that the ratio of  $\mu$ Ci  $^{60}$ Co/ $\mu$ Ci  $^{95}$ Zr is 0.012. After the initial assay, the activities of the two emitters will become equal in:

1.	280	days
2.	290	days
3.	340	days
4.	360	days
5.	430	days

7.

The International Commission on Radiation Units and Measurements has considered it necessary in radiation protection to provide a factor that denotes the modification of the effectiveness of a given absorbed dose by linear energy transfer. This factor is:

- 1. Dose equivalent
- 2. Relative distribution function
- 3. Quality factor
- 4. Relative biological effectiveness
- 5. Distribution factor

-7-

#### Section 4

#### Part I - Typical Questions

#### Fundamentals

1.

The Roentgen is equal to:

1. 1.0 Coulomb/kg.

2. 1.00 x 10<sup>-3</sup> Coulomb/kg.

3. 5.28 x 10<sup>-3</sup> Coulomb/kg.

4. 2.58 x 10-4 Coulomb/kg.

5. 5.28 x 10<sup>-4</sup> Coulomb/kg.

2.

3.

The term solubility or transportability, when applied to the metabolism or radionuclides, refers to the:

- Metabolic breakdown of a radionuclide containing compound which allows its incorporation into body tissues.
- Solubilization of a radionuclide containing compound by means of hydration, ion exchange, or esterification reactions.
- Translocative dissimilation of a radionuclide containing compound by means of biological-chemical action such as enzymatic attachment and catabolism.
- Property of a radionuclide containing compound which results in its transfer across body membranes.
- Translocation of a radionuclide containing compound from one point to another under conditions of physiological dysfunction.
- The collection of ions produced as a result of X or gamma ray interactions in a given small volume of air under electronic equilibrium conditions is a measure of the:
  - 1. Dose equivalent
  - 2. Linear energy transfer
  - 3. Absorbed dose
  - 4. Specific ionization
  - 5. Exposure

#### Operational Health Physics

1.

The half-value thickness for 1 MeV photons in lead approximates 1 cm. A 100- millicurie essentially massless source of Zinc-65 (gamma-ray energy = 1.12 MeV) produces a dose rate of 30 milliroentgens/hour at 1 meter without shielding. What would the dose rate be at about 10 cm from this source with the addition of a 5-cm thick lead shield if the build-up factor is 2.1?

- 1. 0.02 milliroentgen/hour
- 2. 0.9 milliroentgen/hour
- 3. 2 milliroentgens/hour
- 4. 20 milliroentgens/hour
- 5. 200 milliroentgens/hour

2.

3.

In routine environmental surveillance, certain samples are collected and analyzed for specific reasons. In this regard, which one of the following statements is incorrect?

- Foodstuffs are analyzed because they are generally the main route of radionuclide intake by the general population.
- Air and water are analyzed because they are always the most sensitive indicators of environmental releases.
- Muds are analyzed because they are often good indicators of the history of radionuclide wastes in an aquatic environment.
- Aquatic organisms are analyzed because they concentrate certain radionuclides and aid in the assessment of radionuclide contamination.
- Milk and milk products are analyzed because these are generally the major avenue of intake of Strontium-90, particularly among younger population groups.
- The method most commonly used today for removing noble gases from effluent waste streams from nuclear reactors and chemical processing plants is:
- 1. Cryogenic distillation
- 2. Chelation with EDTA
- 3. Adsorption on activated carbon
- 4. Countercurrent ion exchange
- 5. Absorption in freon

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#### Measurements

1.

Which one of the following solid-state materials has the most constant response per roentgen over the energy range of 0.01 to 1 MeV when used as a dosimeter without special shields to correct for energy dependence?

- 1. Calcium Sulfate
- 2. Calcium Fluoride (CaF<sub>2</sub>: Mn)
- 3. L. hium Drifted Germanium
- 4. Low-Z Glass Rods
- 5. Lithium Fluoride (TLD-100)

2.

The response time of an ionization chamber-type survey meter used to measure an X-ray beam is <u>not</u> influenced by the:

- 1. Inertia of the meter movement
- 2. Range selector resistance
- 3. Circuit capacitance
- 4. RC time constant
- 5. Incident X-ray photon energy

3.

In a satisfactory 'air-walled' ionization chamber the ionization per cubic centimeter would be:

- Inversely proportional to the density of the gas in the chamber.
- Inversely proportional to the gamms ray energy absorbed per cubic centimeter of wall material.
- Directly proportional to the stopping power of the walls for electrons.
- Independent of the density of the gas in the chamber.
- 5. Independent of the volume of the chamber.

4.

Unless some type of internal or external quenching is used, a geiger detector will retrigger because of the:

- Breakdown of the detector gas caused by interaction with the negative ion sheath.
- Bremsstrahlung produced by the negative ion sheath during the avalanche.
- Decrease in the density of the positive ion sheath caused by recombination of the ion pairs.
- 4. Electrons released while the positive ion sheath is being neutralized at the outer cathode wall.
- 5. Extraneous noise produced by the high-voltage power supply in the circuit.

362 014

A readistion survey outside the shield at an 8 WeW electron lipear externation becaming intho as comment trangent read unters the excerciple off cares in altonsing approximate instruments and conducting the : States set wernes

- Neutron artivation of Nell sociatilization counters may cause 11. erronerous diaste : rate measurements.
- Fulse mile-up in G-M counters may cause erroneous dose 2. rate measurements.
- Fulse mile-up in HFA counters may cause pronecus neuton. 33 . manager permanentiss.
- Indiametication problem. 44.
- High radiation fields may saturate jon zation chambless, 55 . crausing errorecous chose ratio meessurements.

In performing a macrimum credible resourcer and then tand yes is, which within following assumptions is not generally applied?

- Cound hatel bass of f open ad inneent hass openinged . 11.
- 1000% off the multiple gasses, 500% off the halbgens, and 19% off 22 . thee sodilitis aree red based it to the out many system. .
- 500% of fthe had boxenss red bassed tot be coon ad inneent boulliding 33. milate out tamidage motived asseid to the atmosphere.
- Class F weather conditions exist at thettime of the 4. accident.
- A double ended primary system pipe failure has cocourred. 5.



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99.

The medical radiation exposure of a patient <u>cannot</u> be reduced by using:

- 1. High KVP techniques
- 2. Short time, high MA techniques
- 3. A 3-mm aluminum filter placed in the X-ray beam
- 4. A high speed intensifying screen
- 5. A larger target to film distance

Photoneutron sources are generally made by surrounding a gamma-ray emitting nuclide with:

- 1. Tantalum
- 2. Carbon
- 3. Beryllium
- 4. Aluminu'n
- 5. Iron
- 6.

7.

4.

5.

According to 10CFR20, personnel monitoring is required when an individual:

- Enters an area such that he is likely to receive 1.25 rems to the whole body in a guarter.
- Performs an operation such that he may receive 18.75 rems to his hand, in a quarter.
- Under 18 years of age may receive any amount of radiation regardless of how little the exposure may be.
- Enters an area such that he is likely to receive an exposure in excess of 10% of legal exposure values.
- L'nters an area such that he is likely to receive an exposure in excess of 25% of legal exposure values.

When air is sampled by being pulled through a filter paper, the radioactivity at equilibrium on the filter paper due to naturally courring radon daughters is:

- 1. Proportional to the flow rate of the sampler.
- 2. Dependent only on the total volume of air sampled.
- Dependent on the period of time re red for radioactive equilibrium on the filter paper to be established.
- 4. Dependent on the volume of air sampled after radioactive equilibrium on the filter paper has been est plished.

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5. Independent of the flow rate of the sampler.

### Section 5

Part II - Copies of Past Six Exams

# 362 057

### Answers

### Fundamentals

Question #1	4	
Question #2	1	
Question #3	5	
Question #4	4	
Question #5	4	
Question #6	5	
Question #7	3	

# Measurements

Question #1	5
Question #2	5
Question #3	5
Question #4	4 ·

# Operational Health Physics

Question #1	5
Question #2	2
Question #3	3
Quest.on #4	2
Question #0	3
Question #6	5
Question #7	1
Question #8	2
Question #9	1

#### Question #3

A technician in a pharmaceutical company will handle 500 mCi <sup>131</sup>I, 1000 mCi <sup>198</sup>Au and 25 mCi <sup>42</sup>K. She was employed at the age of 18 and will work under conditions such that she will be exposed to radiation from these sources for one hour per day, five days per week, over an extended period of time. During the exposure period her body position is 60 cm from the radioactive materials located in a laboratory hood. What is the minimum amount of lead shielding (or equivalent) you would prescribe for a barrier at the front of the hood? Manipulators will be provided so that hand exposure within the barrier is not necessary. Assume source strength maintained at levels given, i.e. you may neglect decay of isotopes.

Given:

	131 <sub>I</sub>	198 <sub>Au</sub>	42 <sub>K</sub>
HVL in lead, cm	0.3	0.3	1.2
R/mCi-hr at 1 cm	2.18	2.35	1.50

#### Question #4

You are to survey a new diagnostic x-ray tube unit and collimator for compliance with NCRP recommendations for tube housing and collimator leakage and for total filtration. The tube unit is rated for and operated from a 150 kVp three-phase twelve-pulse generator capable of operation from 0.1 mA tube current for fluoroscopy to 1000 mA for diagnostic radiography. The generator is connected to the tube unit by 35 feet long high-voltage cables.

- a) Describe the instrumentation and procedure you would use, including x-ray equipment operating factors, to determine the maximum tube housing and collimator leakage (mR in 1 hour at 1 meter from the focal spot).
- b) What, if any, is the effect of high-voltage cables on tube housing leakage measurements?
- c) How does half-value layer and the corresponding total filtration determination vary with tube current and high voltage cable length at a constant kVp?
- d) What is meant by the "narrow beam", or "unique" half-value layer and how can this determination be made?

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e) What is the minimum total filtration recommended for a 150 kVp d'agnostic x-ray machine and what is the rationale benind this recommendation?

#### ABHP EXAMINATION 16

362 060

June 12, 1972

PART II - ANSWER ANY SEVEN

#### TOTAL TIME: 3 HOURS

Question #1

You are asked to measure absorbed dose from gamma radiation in various materials under various conditions. For each case shown below list the quantities you need to know to make the measurement to an accuracy of a few percent. Show also the formula you would use to calculate the dose from the measurement you made.

a) You have a small air-filled, air-equivalent wall ion chamber calibrated in roentgens for 1 MeV gamma rays.

You are asked to measure absorbed dose in water from 0.5 MeV gamma rays.

b) You have a small capsule of thermo-luminescent dosimeter (TLD) material. The capsule walls are tissue equivalent. The TLD is calibrated in roentgen for 0.662 MeV gamma rays.

You are asked to measure absorbed dose in tissue from 0.662 MeV gamma rays.

c) You have a small air-filled ion chamber with aluminum walls calibrated in roentgen for 250 kVp x-rays.

You are asked to measure absorbed dose in lead from <sup>60</sup>Co gamma rays.

#### Question #2

In assessing the radiological environmental impact of a power reactor located on a fresh water lake, many possible pathways or modes of off-site human exposure must be considered.

- a) List 10 such possible pathways.
- b) Assume a boiling water reactor with once-through secondary cooling water released to the lake, and a gaseous effluent system equipped with a standard 30 minute delay line for gases and no charcoal absorbers. Which pathway would you expect to contribute the most dose to the populations within a 50-mile radius?
- c) Discuss (briefly) some of the information you should have to do a more complete evaluation of the relative importance of each pathway.

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ABHP Examination 16 - Part II

#### Question #8

Some tritium water vapor was released in a laboratory. An air sample was taken using a freeze out technique (100% freeze out). Ten cubic feet of air were drawn through a trap, the collected moisture was diluted to 50 ml, one ml of the dilution was counted for <sup>3</sup>H beta using a liquid scintillation counter.

Given:

- (1) The instrument background is 12 c/m.
- (2) The counting efficiency is 31%.
  (3) 3200 counts per minute were found in the 1 ml.
- (4)  $2.832 \times 10^4 \text{ cc} = ft^3$ .
- (5) Principal intake by inhalation.

- (6) Biological half-life is 10 days.
  (7) Breathing rate is 10<sup>7</sup> cc/8 hrs.
  (8) 70% of inhaled <sup>3</sup>H assimilated by body water.
- (9) Effective absorbed energy is 0.01 MeV/disintegration.
- (10) Mass of critical organ is 4.3 X 104 grams.
- a) Determine the uCi/cc of tritium in air.
- b) A technician, working for eight hours in this atmosphere, left for a vacation without submitting a urine sample. Estimate his dose in rems based on the air sample data.
- c) If the technician submits a urine sample for tritium analysis when he returns from vacation 20 days after his exposure, what concentration of tritium would you expect to find in this urine sample?

#### Question #9

A transient burst of 1 X 1015 fissions in an unshielded accumulation of fissile material causes a total dose equivalent of 25 rem at 6 feet.

- a) Assuming a neutron-to-gamma dose equivalent ratio of 9, what is the gamma absorbed dose?
- b) Your criticality detector is a gamma response instrument with an alarm point of 50 mR/hr. If the detector responds to 1/2500 of the actual gamma dose rate during a short transient, what is the maximum distance over which this device will be effective in signalling an unshielded, one millisecond transfent of 1 X 1010 fissions? Neglect absorption by the air.
- c) Should sufficient shielding materials be present between the source of the burst and the detector to result in attenuation by a factor of three, what will the maximum distance of (b) be reduced to for a 1 X 1010 fission ourst?

ABHP Examination 16 - Part II

#### Question #5

Assume you have been asked as a consultant to audit the radiation safety program of one of the following nuclear facilities. Prepare a check list of the items you should consider, and discuss the reasons why each is important.

-3-

State your choice:

- 1. Nuclear Fuel Reprocessing Plant
- 2. Radiopharmaceutical Manufacturing and Supply
- 3. Industrial Radiography Involving X-Rays and Isotopes
- 4. Power Reactor Facility

#### Question #6

An employee at a facility where you are the health physicist has been involved in an incident wherein he is suspected of having inhaled plutonium oxide (insoluble).

- a) Compare the advantages and disadvantages of attempting to determine his lung burden due to Pu-239 by direct counting techniques.
- b) Compare the advantages and disadvantages of using the 60 KeV photon from the Am-241 present to determine the Pu-239 lung burden.
- c) Compare the techniques of (a) and (b) with urine and fecal sampling.

#### Question #7

You are asked to consult in the design and installation of a 15 MeV electron accelérator to be used for cancer therapy. The electron beam will strike a thick tungsten target in the accelerator to produce x-rays.

- a) There is a choice of material for collimators and target shielding, U-238 or Pb. Which would you recommend and why?
- b) The accelerator is to be installed in a room which previously housed a Co-60 teletherapy unit. The room has 2 ft thick concrete walls and one wooden door with a 1/4 in layer of Pb affixed to the inside surface. Discuss what you would consider in evaluating the adequacy of the accelerator and room shielding.

#### ABHP EXAMINATION #17

PART II

#### June 18, 1973

#### PART II - ANSWER ANY SEVEN

#### TOTAL TIME: 3 HOURS

#### Question #1

Describe in detail the advantages and disadvantages, energy dependence and sensitivity of 2 of the following personnel neutron dosimetry systems.

- 1. NTA film 2. <sup>6</sup>LiF-<sup>7</sup>LiF TLD
- 3. Albedo
- 4. Fission Track

#### Question #2

The health physicist's evaluation of radiological exposures to man and his environment from man-made sources is complicated by the existence of natural sources.

Consider the following natural contributors; <sup>40</sup>K, cosmic radiation, uranium series and thorium series. For each category below, briefly state how they might affect a health physicist's measurements.

- 1. Air monitoring,
- 2. Sample counting,
- 3. In vivo counting,
- 4. Radiation background measurements,
- 5. Calibration of low-level instruments,
- Materials for construction and shielding of low-level counting facilities,
- 7. Radiochemical analyses including materials and equipment used.

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ABHP Examination 16 - Part II

#### Question #10

A radiochemist is planning to analyze lunar samples by activation analysis. In developing his analysis procedures he activates a number of knowns including antimony.

-5-

- a) Based on the information provided, how much <sup>122</sup>Sb activity would be have at the time be initiated his work?
- b) What would be the dose rate at 1 foot from the unshielded sample? State all assumptions. Use "rules of thumb" if you wish.
- c) What precautions would you recommend for handling this sample assuming it is to be pulverized?
- d) In working up the Sb sample he has a spill which results in inhalation of 122Sb. His initial body burden is determined to be  $4 \times 10^2 \ \mu$ Ci. If the (MPC)<sub>a</sub> for 122Sb is  $2 \times 10^{-7}$   $\mu$ Ci/ml, would this incident require reporting to the AEC (or State) Authorities? (Assume a breathing rate of  $10^7$  ml per 8 hours.)

#### Data for Antimony Sample

Sample Mass:		1 mg antimony
Isotopic Abundance of <sup>121</sup> Sb:		57.25%
Cross Section for $^{121}$ Sb (n,y)	122 <sub>Sb</sub>	
	Reaction:	6 barns
Half-life of <sup>122</sup> Sb:		2.8 days
Principal Gamma Ray:		0.564 MeV (70% abundance)

8 Particles

1.97 MeV (70% abundance) 1.40 MeV (30% abundance)

E <sub>g</sub> (MeV)	(Range S/cm <sup>2</sup> )	
0.5	0.2	
0.7	0.3	
1.5	0.7	
2.0	1.0	
Fradiation Conditions		
Fradiation Time:	6 days	
lux Density:	$5 \times 10^{13}$ n/cm <sup>2</sup> -sec	
lapsed time between end of irradia- ion and start of work:	2 days	3

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Ticana

#### ABHP Examination #17

#### Question #5 (Continued)

the various radioactive wastes resulting from the reprocessing of irradiated fuel.

 a) <u>Briefly</u> discuss in general terms the quantities and hazards of the gaseous, liquid and solid waste materials generated in the reprocessing.

- 3 -

- b) Describe the treatment and disposal methods appropriate to the various radioactive wastes identified above.
- c) Identify the radionuclides which will continue to represent a hazard over the first several hundred years, and those which represent the hazard over thousands to millions of years; describe some of the proposed solutions to the problem of "ultimate disposal" of these materials.

#### Question #6

A release of airborne, particulate, alpha-emitting activity has occurred in a large room in which there are glove box operations with various heavy metal alpha emitters. The release was detected by an alpha air monitor which alarmed. The four men who were in the room left following the alarm. You were notified within a few minutes and reported immediately to the scene and find that the exact source of the release is unknown, the four men are contaminated and none was wearing respiratory protection.

You are the lead health physicist, have adequate staff assistance, and your facility has a medical staff, in vivo gamma/x-ray counter, bicassay lab and radioanalytical labs.

List, in a rough chronological order, the actions you would take, the recommendations you would make and the reason for each.

#### Question #7

Radiation effects are influenced by the density of energy deposition of the impinging radiation. Some radiation delivers energy to a relatively large volume of the cell (e.g., gamma rays) and has a low relative biological effectiveness (RBE). Other radiation delivers energy to a highly localized part of the cell (e.g., alpha particles) and has a high RBE. Several closely spaced ionization events are referred to as an "ion cluster."

Tumor cells having critical structures with 10<sup>-7</sup> cm diameter are being irradiated. Assume that one ion cluster has an energy density of 100 eV/ion cluster and that one ion cluster will destroy or inactivate one cell.

ABHP Examination #17 - Part II - 2 -

Question #3

<sup>85</sup>Kr is continuously released from operating nuclear reactors to the environment.

(1) Briefly describe how you could monitor for <sup>85</sup>Kr in the stack effluent when it is masked by other short-life noble gases.



- (2) Briefly describe how you could monitor the environment near a reactor boundary for <sup>85</sup>Kr.
- (3) Would you expect any significant uptake of <sup>85</sup>Kr by biota? Why?
- (4) Describe how you would calculate a maximum estimated radiation dose-rate (skin and whole-body) to nearby resident's (e.g., a few miles away) based on the measured release rate at the point of release.

#### Question #4

NBS Handbook 97 lists neutron attenuation coefficients for various shield materials. In particular, for 4 MeV neutrons, the attenuation coefficient for iron is 0.31 cm<sup>-1</sup> while the attenuation coefficient for ordinary concrete is 0.157 cm<sup>-1</sup>.

- a) Ignoring cost factors, why is iron alone not satisfactory for neutron shielding?
- b) Design a combined iron and concrete shield for a 4 MeV neutron source emitting 5 x 10<sup>10</sup> neutrons per second isotropically such that the fast neutron flux outside the shield at 2 ft from the source is less than 5 neutrons-cm<sup>-2</sup>-sec<sup>-1</sup>.
- c) What would be the flux density if only 2 ft of concrete were used for shielding.

362 050

#### Question #5

One of the important health physics problems arising from the generation of electrical energy by the use of nuclear reactors is the safe disposal of

# ABHP Examination #17 - Part II - 5 -

# Question #9 (Continued)



TABLE S	- Shielding	requirements,	for	radiographic	installations
---------	-------------	---------------	-----	--------------	---------------

WUT" in mA min/week					listanc	• is 7	est in	am So	urce C	X-Ray	Tube	Tarret	
100 kVp         125 kVp           1,000         400           500         200           250         100           125         50           62.5         25		kVp LS0 kVp	te Occupied Area										
		00 00 00 50 25	0 200 100 100 100 100 100 100 100	5	7	10 7 5	14 10 7 5	20 14 10 7 5	28 20 14 10 7	40 23 20 14 10	40 28 20 14	40 28 20	40 28
Type of Are		3	Kateriai	Prier in, Totective Barrier Talekaess									
Controlled Noncontrolled		Lead, mm <sup>3</sup> Lead, mm <sup>3</sup>		1.9 2.65	1.65	1.4	1.2	1.0	0.75	0.5	0.3	0	) 0.5
Controlled Noncontrolled		Conc	rete, in* rete, in*	5.9	5.2 1.3	4.6	4.0	3.3 3.4	2.7 4.8	2.1 4.1	1.6	1.0	0.4
				Secondary Protective Barrier Thickness							-		
Controlled Noncontrolled		Lead, Lead	min* . mm*	0.55	0.4	0.2	0.1	0	0.25	0 0.1	0	0	0
Controlled Noncontrolled		Concrete, in* Concrete, in*		1.0	1.4 3.2	0.8	0.2	0 1.5	0	0 0.4	00	0	0

\* W-workload in mA min/week, U-use factor, T-occupancy factor.

\* See Table 25 for conversion of thickness in millimeters to inches or to surface density.

\* Thickness based on concrete density of 2.35 g/cm\* (147 1b/ft\*).

JOR ORIC

#### ABHP Examination #17 - Part II - 4 -

#### Question #7 (Continued)

- a) Which of three radiations having linear energy transfer's (LET'e) of 10 keV/µ, 100 keV/µ and 500 keV/µ would you expect to be the most efficient for tumor destruction where the tumor is given the same total dose for each of the three radiations? Why?
   (1µ = 10<sup>-4</sup> cm)
- b) If the tumor were irradiated, using the most effective radiation, with 1000 rads, how much would the average temperature increase in each cell? (4.18 x 10<sup>7</sup> ergs/cal; assume tumor tissue = water).
- c) Many tumors are poorly vascularized, particularly near the center, and hence are far from oxygen-saturation. Discuss the "oxygeneffect" for low LET radiations.

#### Question #8

The liquid contents of a beaker containing 10 millicuries of  $^{126}$ I accidentally boils to dryness in a laboratory measuring 4 meters x 4 meters x 3 meters high. A person working in the room breathes the vapor for 30 minutes before discovering the accident. Assuming a breathing rate of  $1.25m^3$  per hour, and the fraction of the inhaled iodine reaching the critical organ was 0.23:

- 1) Calculate the maximum uptake by the critical organ
- 2) Calculate the maximum dose commitment to the critical organ (rems)

Whole body weight = 70 kg Thyroid weight = 20 g T<sub>r</sub> = 13.3 days T<sub>b</sub> = 138 days  $\sum EF$  (RBE)n = 0.16 MeV

- 3) Why does this probably represent a maximum dose estimate?
- 4) Would you expect this dose to produce any observable biological effects? Why?

#### Question #9

Shown below is the plan view of a proposed 125 kVp radiographic x-ray installation to be used for general radiography. The useful beam can strike all barriers except A - B. For a workload of 400 mA-min/week, specify the lead thickness required at 5 of the 7 points. State all assumptions on which your calculations are based.

362 068

See the following page for tabular data.

#### ABHP EXAMINATION #18

PART II

July 8, 1974

### PART II - ANSWER ANY SEVEN TOTAL TIME: 3 HOURS

#### Question #1

A solvent vapor explosion has taken place in a source encapsulation facility at the U.S. Radionics Company site. The most recent isotope inventory for the facility indicates a total content of 10 mg of <sup>252</sup>Cf as the oxide. Continuous air monitors with audible alarms indicate significant amounts of alpha activity. Three of the facility occupants have evacuated to a pre-assigned hold-point just outside the facility entrance. As director of the health physics emergency response term discuss the following:

#### Point Value

2

3

3

2

- 1. Your priorities in the initial response.
- 2. The steps you would take for proper total response.
- 3. Personnel protection for team members.
- 4. Monitoring and surveillance for cleanup operations.

Data on 252Cf:

- a) Specific neutron dose rate: 2.4 x 10<sup>3</sup> rem/hr.gm. at 1 m.
- b) Specific gamma dose rate: 1.4 x 10<sup>2</sup> rem/hr.gm. at 1 m
- c) Specific activity (alpha): 5.37 x 10<sup>2</sup> Ci/gm
- d) MPC, (40-hr week): 3 x 10-11 uCi/cm3

#### Question #2

An employee working in a glove box containing  $^{239}PuO_2$  discovers that he has a heavily contaminated hand. It was determined that the contalination was the result of a hole in one of the glove box gloves. It was estimated from a recording air monitor (with a defective alarm) that the employee was exposed to an airborne  $^{239}PuO_2$  concentration of 4 x  $10^{-8}$  uCi/cc for one hour. From cascade impactor results the mass median aerodynamic particle size (MMAD) was estimated to be 0.5 u.

#### Point Value

7

3

- a. Given the revised lung model data on the attached page and assuming uniform energy deposition in the lungs, calculate the total integrated dose in rem to the pulmonary region of the lungs.
- b. Briefly discuss the current controversy surrounding the assumption of uniform energy deposition in the lungs for an inhalation exposure of this type.

ABHP Examination #17 - Part II - 6 -

#### Question #10

A quantity of tritium was accidentally released. Bioassay data indicated assimilation of tritium by exposed individuals.

- a) An initial tritium activity measurement in urine for one case was 3.4 x 10<sup>-3</sup> بن المار (ml and 5 days later was 2.4 x 10<sup>-2</sup> بن المار (ml. Estimate the retention half-period of tritium for this individual.
- b) Identify one treatment that might be instituted to reduce the total integrated dose.
- c) One urine sample measured 23.4 c/m, including counter background, compared to a background count rate of 19.1 c/m. If both rates were determined by 100 minute count times, estimate whether or not the observed count rates are statistically different.
- d) Some of the accidentally released tritium is ultimately discharged to the environment. Mechanisms which have been found to be important in the reconcentration and redistribution of environmental radionuclides include bioconcentration and transpiration. State in one or two sentences the importance of these mechanisms in determining the environmental behavior of tritium.

#### ASHP Examination #18 - Part II -2-

#### Point Value Question #3

10

A radiation dosimeter is made from a cubical plastic scintillator (5 cm on a side). The light output is detected with a photomultiplier tube and the resulting current is measured with an electrometer. The dosimeter is calibrated with a 137Cs source with an activity of 4.5 mCi. The electrometer reads 6.0 x  $10^{-7}$  amperes (background subtracted) when the source is placed 1 meter from the center of the detector.

An Iodine-125 source gives a reading of 2.3 x  $10^{-6}$  amperes at the same distance (background subtracted).

What is the exposure rate at 1 meter from the Iodine-125 source? Neglect the effect of scattered radiation or the inverse-square law distance variation through the detector. Assume the energy flux density falls off in the crystal as  $e^{-\mu enX}$  and the radiation is incident normally on the crystal face.

Data	137Cs 0.66 MeV	1251 0.027 MeV
Mass energy absorption coefficient, cm <sup>2</sup> /g, for crystal	0.031	0.097
Gamma photons emitted per disintegration	0.935	.07
Mass energy absorption coefficient, cm <sup>2</sup> /g, for air	0.029	0.25

Density of air at 0°C, 760 mm	0.001293 g/cm <sup>3</sup>
Density of crystal	1.0 g/cm <sup>3</sup>
Mean electron volts to produce 1 ion pair	34
Charge on electron	1.6 x 10 <sup>-19</sup> Coulomb
Ion pairs/cc - Roentgen	2.08 x 10 <sup>9</sup>
Coulomb/gram - Roentgen	2.58 x 10-7

#### Question #4

oint Value	Give the physiological effects to be expected from the following acute exposures:
2	a) 300 rad to the whole-body from <sup>60</sup> Co gamma rays;
2	<li>b) 25 rad to the whole-body from <sup>60</sup>Co gamma rays;</li>
2	<ul> <li>c) 1000 rad to the hands from <sup>32</sup>P beta rays;</li> </ul>
2	d) 500 rad to the whole-body from fast neutrons.
2	How would you modify your answer if the exposures were uniformly spread over a period of one year?

-27-

#### DATA for PROBLEM #2



R glon	Pathway	Compound class		
		(D)	(.4)	(٢)
N-P	(a) (b)	0.01 d/0.5 0.01 d/0.5	0.01 d/0.1 0.4 d/0.9	0.01 d/0.01 0.4 d/0.99
т-з	(c) (d)	0.01 d/0.95 0.2 d/0.05	0.01 1/0.5 0.2 1/0.5	0.01 J/0.01 0.2 J/0.99
P	(c) (f) (3) (5)	0.5 d/0.8	50 d/0.15 1 d/0.4 50 d/0.4 50 d/0.05	500 d/0.05 1 d/0.4 500 d/0.4 500 d/0.15
L	(i)	0.5 d/1.0	50 d/1.0	1000 d/0.9



\* The first value listed is the biological half-life; the second is the regional fraction.



FIG. 14. The deposition estimate for the pulmonary compartment while breathing at a moderate work rate. Different acrosol distributions are represented by the mass or activity median acrodynamic diameters (0.01 to ~100).
DATA FOR PROBLEM #7



400A NULUTION

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## ASHP Examination #18 - Part II

handle each of the following?

#### Point Value Question #5

10 Differentiate the merits of NaI(T1), Ge(Li), and Si(Li) for gamma spectroscopy of environmental samples from uncontrolled areas.

#### Question #6

Point Value

- 5
- a) You are required to analyze the stack effluent for radioiodine in particulate, elemental, organic and other forms. Briefly describe how you would sample this effluent, analyze the sample, and interpret the results.

As a health physicist in a fuel reprocessing plant, how would you

-3-

2-1/2

2-1/2

Point Value

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4

- b) You are asked to show that your stack sample probe is icokinetic. How would you do it?
- c) List the factors which determine the rate at which liquid waste can be discharged to a stream. List at least four of them.

#### Question #7

A cullical room, 8 meters on a side, contains a 14 MeV neutron source at the center emitting 1015 n/sec. It is desired to shield the room so that the dose equivalent rate in adjacent rooms is less than 2.5 mrem/h. Ten-foot chick walls of ordinary concrete are planned. The roof will have two feet of concrete.

- - a) Calculate the dose equivalent rate "coming through" the walls, and the percentage due to gamma radiation.
  - b) Discuss the adequacy of the overall shielding design. Do you foresee any problems?
  - c) What additional radiation problems would be associated with the operation of this facility.

State any assumptions you make that affect your answers, and support your discussion with calculations where appropriate.

Given: Figure (attached)

for .
14 MeV
neutrons {
0F = 7.5
12 n/cm<sup>2</sup>-sec = 2.5 mrem/h
neutron 0E absorption coefficient in ordinary concrete = 0.07 cm<sup>-1</sup>
(includes neutron buildup)

362 074

ABHP Examination #18 - Part II -5-

#### Question #9

Point Value	a low 1	evel lab, and a counting room all in one complex.
4	a.	Discuss the design of the air supply and exhaust system for this complex. Include such things as air flow paths, flow rates, filtration, air treatment, placement of exhaust fans, etc. Give reasons to justify each of your design recommendations.
4	b.	Discuss your recommendations for the drain system in this complex. Include such things as appropriateness of segregated drains recommended materials of construction, routing, traps, etc. Give reasons to justify each of your design recommendations. (Assume the facility of which this lab complex is a part also
2	c.	has a radwaste treatment complex.) Discuss your recommendations for floor and wall coatings.

A radiochemistry laboratory in a facility consists of a high level lab,

#### Question #10

Give an explanation of why the following are examples of situations Point Value in which charged particle equilibrium (CPE) conditions do not exist:

- 3-1/3 a. an air-tissue boundary,
- 3-1/3 b. near a point source of radiation,
- 3-1/3 c. a 10 MeV photon beam in air incident upon an air-aquivalent dosimeter.

362 075

#### ABHP Examination #18 - Part II -4-

#### Question #8

You are a health physicist at an accelerator facility and are asked to participate in the design of shielding required for a new experimental beam. The information you are given is:

Accelerator - electron LINAC for industrial radiography Beam energy - 25 MeV Peak current - 1 amp Beam pulse width - 2 usec Pulse repetition rate - 360 pulses/sec

The electron beam is to be bent 90° with a radius of curvature of 10 cm.

#### Point Value

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1

 Where would one expect to find significant radiation sources within the machine? What types of radiation will you consider under:

- a) normal operating conditions,
- b) a failure in some portion of the beam transport system at the bend?
- Qualitatively, what would your shield design be at the bend and why?

 You are told that the continuous beam loss in the bend will be less than 0.1% and that the interlock systems will reliably turn off the beam within 2 pulses if any failure occurs in the beam transport system.

- a) Would you consider the continuous beam loss or the failure situation to be the limiting case for determining shielding? State any assumptions and all considerations.
- b) Is it necessary to consider activation of machine parts when designing the shield? Why or why not?

You have just been hired as a health physicist by Acme Rad Services, Inc., which is planning to install an 11 kCi 60Co source for industrial purposes. The source pig is to be located in an existing room shown in Figure 1 below. It has been previously determined that with the planned workload of 10 h/week, the exposure rates just outside of walls A and S are 0.17 R/yr and 5 R/yr, respectively. Wall D is a very thick concrete wall because of an accelerator on the far side. Wall C is a thin wallboard wall to be rebuilt of ordinary concrete.

-2-

#### Point Value

5

5

. .

- a) Using the data below and Figures 1 and 2, calculate the minimum shielding for Wall C required to reduce the exposure rate on the far side to 5 R/yr. Neglect any build-up factors and consider only radiation scattered at  $90^{\circ}$  from the object.
- b) Comment on the entire installation from a health physics point of view.

Given:

Density of ordinary concrete: 2.35 g/cc

Ratio of 90° scattered radiation at 1 meter from radiographed object to incident exposure: 10-3

For 60Co: 1.3 RHM per Curie.

Energy of scattered photon  $\epsilon' =$ 

 $\frac{E}{1 + (E/m_0 c^2)(1 - \cos \theta)}$ 

 $m_{c}c^{2}$  = rest mass energy equivalent of electron = 0.51 MeV

E = initial photon energy

Mass attenuation coefficients for ordinary concrete (See Figure 2)



#### ABHP EXAMINATION #19

#### July 14, 1975

#### Part II - Answer any seven

#### Total Time: 4 hours

#### Question #1

A serious accident has resulted in the dispersal of reactor-grade plutonium dioxide on a busy interstate highway. You have survey instruments from which you can estimate the average plutonium activity per unit area of contaminated surface. As the health physicist on the emergency response team, you are asked to establish an exclusion zone to limit public access during cleanup operations.

Point Value

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A

- a) Briefly discuss the health physics considerations which you would use in establishing a maximum contamination level at the exclusion zone barricades immediately post-accident, and during cleanup.
- b) What health physics considerations would bear on the establishment of an acceptable residual contamination level for long-term public access after cleanup?

#### Question #2

Maximum Permissible Concentrations in air of many insoluble radioactive isotopes as recommended by the ICRP, NCRP, and codified in 10 CFR Part 20 of the Code of Federal Regulations, are based on the assumption that the material is uniformly deposited in the lung, and that there is a uniform distribution of energy per gram of lung tissue.

- a) Is this a reasonable assumption with regard to large numbers of beta and gamma emitting particles? Why?
- b) Is this a reasonable assumption with regard to alpha emitting particulates such as 239Pu? Why?
- c) Would you expect the assumption of uniform distribution of particulates and energy in the lung to result in an underestimate or overestimate of the risk of cancer from inhalation of 239Pu? Why?

-32-

#### Question #4

Two important reactions for thermal neutrons in tissue are  ${}^{14}N$  (n,p)  ${}^{14}C$  and  ${}^{1}H$  (n,y) ${}^{2}H$ . Calculate:

-3-

#### Point Value

- 6
- a) The absorbed dose and dose equivalent for each reaction in tissue per unit thermal neutron fluence (n<sub>th</sub>/cm<sup>2</sup>).
- 4
- b) The maximum permissible thermal neutron flux density based on the sum of these two reactions.

State any assumptions necessary in making calculations.

Given:

 $N_1 = 6.02 \times 10^{22} \text{ atoms/g tissue}$ H  $N_{14} = 0.11 \times 10^{22} \text{ atoms/g tissue}$ 

N

 $\sigma_{th}({}^{1}H) = 0.33 \text{ barns}$   $E_{\gamma} = 2.2 \text{ MeV}$  $\sigma_{th}({}^{14}N) = 1.3 \text{ barns}$   $E_{p} = 0.6 \text{ MeV}$ 1 rad = 10<sup>-2</sup> J/kg 1 MeV = 1.6 x 10<sup>-13</sup> J

Fraction of y energy absorbed in body = .28

 $1 \text{ barn} = 10^{-24} \text{ cm}^2$ 

tissue density = 1 g/cc

## Point Value Question #5

10

You are hired as a Health Physics Consultant by a utility planning to build a nuclear power reactor. Discuss in general terms the basic elements of the environmental surveillance program (including rationale) for radiation and radioactivity you would recommend.

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### DATA FOR QUESTION #3

## MASS ATTENUATION COEFFICIENTS FOR OPDINARY CONCRETE

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#### Question #9

A graduate student was working with 10 Ci of tritium gas in a hood. As the result of a small explosion in the tritium gas container, the container was ruptured and the front of the hood was blown out. The considerably shaken, but otherwise uninjured student, suspected that he might have received some tritium uptake. He collected a urine sample approximately 15 minutes following the incident and submitted it to the Radiation Safety Officer (RSO). The RSO requested that the student submit another urine sample in 2 hours. The analysis of this second urine sample indicated a tritium concentration of 2 mCi/2.

Point Value

1

3

4

2

1.0

a) In your judgment was the RSO correct in requesting the second urine sample to evaluate the uptake? Why?

- 5-

- b) Calculate the students integrated dose equivalent assuming an effective elimination half-life of 10 days.
- c) What would the student's average daily liquid intake have to be to reduce the integrated dose equivalent to 2.5 rem.
- d) If you were the RSO would you recommend to the student the increased fluid intake necessary to reduce his dose equivalent to 2.5 rem.

Given: Critical Organ for Tritium is Body Water (43 litres) QF for tritium = 1 Energy of tritium beta:  $E_{max.} = 18 \text{ keV}, E_{ave.} = 5.6 \text{ keV}$ 1 eV = 1.6 x 10<sup>-13</sup> joules 1 rad = 10<sup>-2</sup> J/kg

#### Question #10

University research operations often utilize a variety of radiation sources, such as large fixed gamma sources, X-ray machines, nuclear reactors, particle accelerators, neutron sources, and unsealed radioisotope sources. Each of these radiation sources must be installed and used so as to minimize the radiation dose to individuals. Considering the basic principles for reducing personnel dose, discuss which method(s) you would emphasize in each of the following cases. Explain your reasons in each case.

Point Value

- 3-1/3 a) 50 mCi of <sup>32</sup>P used in a biochemical labeling experiment.
- 341/3 b) 5000 Ci of 60 Co as a sealed source used for radiation damage studies.
- 3+1/3 c) A one time transfer of 1 mg of <sup>252</sup>Cf as a sealed source from its shipping container to a large experimental water tank.

362 081

-4-

#### Question #6

It is recognized that 137Cs (T<sub>1/2</sub> = 30 y) comprises a significant fraction of fallout radioactivity. Given an initial background exposure rate of 1.5 uR/hr from 137Cs in the soil:

# Point Value

3

3

3

- a) Calculate its initial annual exposure rate contribution.
- b) Calculate the integrated 30-year exposure to each individual in the population at risk, assuming no additional fallout.
- c) Compare the 30-year exposure value with the I.C.R.P. population gonadal dose limit and briefly discuss its significance.
- d) Discuss the other factors (in addition to external exposure) which should be considered in evaluating the radiological significance of 137Cs fallout to the genera' population.

#### Question #7

Radwaste handling and processing is an important part of a power reactor health physics program.

Point Value

- a) In a power reactor, list three (3) sources of each of the following types of radioactive waste.
  - Liquid waste
     Gaseous waste
  - 3. Solid waste
- 2.5 b) Briefly discuss at least three (3) methods for processing liquid waste.
- C) Briefly discuss at least three (3) methods for processing gaseous waste.
- 2.5 d) Briefly discuss at least three (3) methods for processing solid waste.

#### Question #8

The radiation doses received during the annual outages at power reactors contribute significantly to the total personnel dose in these facilities. Select either a PWR or BWR and discuss.

Point Value 3

- a) Which outage jobs are the major sources of exposure?
  - b) As the health physicist, what specific recommendations would you make to reduce the exposure received on the jobs listed in (a, above? 0.82

#### Question #13

Point Value

10

12.1

A patient is to be given a 200 mCi <sup>131</sup>I oral therapeutic dose (as iodide) for an inoperable thyroid metastasis. The thyroid has been surgically removed during a previous hospitalization. Briefly discuss the health physics aspects of the dose administration and the following hospitalization of the patient.

#### Question #14

The plan below (Figure 3) shows a proposed <sup>60</sup>Co teletherapy installation. The useful beam can be directed only at the floor and at wall BC.

#### Point Value

10

a) Using the attached table, specify the concrete shielding required for Point 2 and any three (3) of the others shown for a workload, W, of 120,000 R/week at one meter. List and explain <u>all</u> assumptions used in arriving at the shielding specified.



FIGURE 3

#### Question #11

Neutron radiation is often a major contributor to the radiation environment around particle accelerators.

-6-

#### Point Value

2

8

- a) List four (4) important processes by which neutrons interact with matter.
- b) For each interaction process listed in part (a), describe a neutron detector based on that interaction process. Briefly discuss the application of each detector in measuring neutrons around an accelerator.

#### Question #12

You are hired as a consultant by an industrial firm who proposes to use an electron accelerator for the unique application of excavating rock. Two alternative designs are proposed, one producing an energy of 2 MeV with an average beam current of 5 amps, the other using a beam energy of 10 MeV with the same average beam power. There is no difference in the efficiency of either accelerator in excavation; they may be manufactured at the same cost.

- a) Which accelerator would you recommend be produced? Why?
- b) Calculate the maximum radiation level at the surface of the ground when a 2 MeV, 10 MW accelerator is operating 2 meters underground.

Given:

The forward Bremsstrahlung intensity, I, produced by an electron beam impinging on a thick target is given by:

I (wat:: cm <sup>-2</sup> per amp at 1 meter) =  $5.0 \times 10^{-2} T (T+0.51)^2 \ln (950 R/x_0)$ T = Electron energy in MeV (Rock may be assumed identical to aluminum in its atomic properties.) R = range of 2 MeV electrons in Al = 0.95 gm cm <sup>-2</sup>  $x_0$  = radiation length of Al - 25.3 gm cm <sup>-2</sup> Assume 10<sup>6</sup> photons cm <sup>-2</sup> sec <sup>-1</sup> = 1 rem h<sup>-1</sup>. 1 MeV =  $1.6 \times 10^{-13}$  joules 1 joule/sec = 1 watt Attenuation coefficient of photons in rock = 0.15 cm<sup>-1</sup> Assume a buildup factor of 2.

#### Question #15

In a fuel reprocessing plant, irradiated fuel is dissolved so that it is chemically separated into three main streams:

3-1/3 a) uranium

Point Value

- 3-1/3 b) plutonium
- 3-1/3 c) fission products

Assume that maintenance work must be done on a pump in each of these streams. Briefly discuss the health physics precautions which must be taken for the work on each stream.

#### Question #16

One area of a fuel reprocessing plant is made up of the five rooms shown on the attached sketch (Figure 4). You are being consulted by the facility engineer to assist him in properly designing the ventilation system. He gives you the attached sketch and the following information:

- a) The ventilation supply and exhaust for this area will service only the five rooms shown.
- b) Each room will have its own supply and exhaust duct and any volume of air can be supplied to and exhausted from any room. (The facility engineer will design the pressure drop between areas so the proper air flow patterns will exist when doors are opened.)
- c) It is felt that the NR. ill agree to waive the Reg. Guide 3.12 requirement for roughing filters on the exhaust of each room if their absence will permit a single alpha constant air monitor to service the entire area and detect 1 x MPC within 4 hours if it occurs in any one of the five rooms.
- d) Pu-239 is the limiting radionuclide (MPC, for  $^{239}$ Pu is 2 x 10<sup>-12</sup> µCi/cc)
- e) The design criteria states that each of the five rooms must have at least 5 air changes per hour.
- f) Ceiling heights:

Crane and Equipment Maintenance Area = 20' Product Container Storage Area and the Plutonium Loadout Operating Station = 14' Air Lock and Corridor = 12'

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## TABLE 21-Coball-50 shielding requirements for controlled areas. 2

a For a weekly design level of 100 mR; add one tenth-value layer (TVL) for regions in the environs to reduce radistion to 10 mR/week.

bW -workload in R/week at 1 m, U-use factor, T-occupancy factor.

c Thickness based on concrete density of 2.35 g/cm3 (147 lb/(14).

d Refers to leakage radiation from source housing when source in "ON" condition; may be ignored if less than 2.5 mR/h at 1 m.

e For large field (20 cm) and a source to skin distance of 40 to 60 cm. This ineludes scattering from the collimator and from the phantom [3].

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10.0

DATA FOR QUESTION 16 FIGURE 4



In checking various alpha constant air monitors, the one you have decided to recommend uses a kinetic impactor system with a step advance tape. This monitor has the following specifications:

-9-

- a. Nominal flow rate = 20 cfm
- b. Detector efficiency = 30%
- c. The tape advance frequency is adjustable so that a 4 hour sample time is possible.
- d. Normal background on the monitor is 10 cpm.
- e. The meter scale and time constant of the monitor are such that 20 cpm above background is easily recognized as a positive reading.

#### Point Value

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Calculate the minimum airflow which must be used for each of the five rooms so that you meet all the design criteria and can detect 1 x MPC in any room within 4 hours by sampling the common exhaust header. (Neglect the volume of air that is in any personnel door opening.) (1 cu. ft. = 28,300 cc)

362 088

ABHP EXAMINATION #20

ine 28, 1976

Part II - Answer any seven

Total Time: 4 hours

#### Question #1

The transportation and disposal of radioactive waste have received Point Value considerable attention in recent years.

- a) Discuss the environmental impact of accidents involving the transportation of radioactive waste. Include in the discussion the relation of transportation regulations to the environmental effects of accidents.
- b) Discuss the environmental aspects of the present disposal of 1) activation products and 2) transuranic wastes. Include in the discussion packaging requirements and environmental considerations of disposal site selection and operation.

#### Question #2

Electron capture detectors for gas chromatographs us ditium or <sup>63</sup>Ni foils in the cells. Release rates for each, at their normal operating temperatures, are 10 uCi/min and 10 nCi/min, respectively. One of each type is located in a room 8m x 6m x 3m in a laboratory building fairly accessible to the general public. Assume that the tritium is released as the oxide whereas only 50% of the nickel released is soluble.

A fan in the room provides reasonably complete mixing. The ventilation system, which exhausts the room air directly to the outdoors, provides Point Value three air changes per hour.

- 4 a) 1
- a) What are the average room concentrations of tritium and <sup>53</sup>Ni at equilibrium when the gas chromatographs are operating?
- 6

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b) Discuss the health physics program that you would recommend for this operation.

 $\begin{array}{c|cccc} \underline{Data:} & \underline{MPC (air) (\underline{uCi/m1})} \\ \hline \\ \hline \\ \hline \\ Tritium (as HTO or T_2O) & \underline{Sx10^{-6}} & \underline{2x10^{-7}} \\ \underline{63}_{Ni} (S) & \underline{6x10^{-8}} & \underline{2x10^{-9}} \\ (I) & \underline{3x10^{-7}} & \underline{1x10^{-8}} \end{array}$ 

-43-

A radiographer installed x-ray film around a pipe weld prior to inserting a sealed source of gamma radiation into the pipe as part of the inspection procedure. After completion of the film installation, it was discovered that he had been exposed from the back to the source which had not been fully retracted into its shield.

The radiographer's dosimeter was processed and he described and re-enacted (the source having been removed) the installation of the film. It was found that he wore his dosimeter near the midline of his chest about 20 cm above his belt; the source was at the level of his belt, in line with the midline of his back, and at a distance of 10 cm from the surface of his back; and the dose equivalent at the location of his dosimeter was 0.1 rem for the duration of this exposure.

- a) Calculate the maximum dose equivalent at the surface of his back given the following: the HVL for the radiation in any tissue in this case is 5 cm; neglect any other effect of scattering or buildup and any attenuation by air or clothing; the radiation at any point considered is in equilibrium with soft tissue; the radiographer's torso is assumed to be a slab of soft tissue, 25 cm thick and 35 cm wide; and the source is a point source.
- b) List the corrective measures which you would institute to prevent a recurrence of this incident.

#### Question #4

Point Value

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The following average life span data on a large group of young adult rats that survived early mortality (more than 30 days) was collected using cobalt-60 gamma radiation delivered in single acute doses.

Dose (rads)	Life Span (days
15	997
70	980
150	909
300	832
425	827
550	780
650	739

The control (unirradiated) rot lifespan was 1000 days. The dose leading to 50% mortality (LD<sub>50-30</sub>) in a third group of previously unirradiated rats was 700 rads.

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(Continued on next page)

#### Point Value

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4

3

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- a) Estimate the average life-span shortening due to acute <sup>60</sup>Co irradiation for rats in days per rad.
- b) Determine an equation for the life-span as a function of gamma dose. Express the life-span as percent of control lifespan. Make the equation as simple as possible that fits the data approximately.

-3-

c) Assuming that equal fractions of the LD<sub>50-30</sub> for the different species of mammals produce the same percentage lifespan loss, estimate the life-shortening effect in years in man from a single acute dose of 100 rads. The LD<sub>50-30</sub> for man in this case is 300 rads at the midline. The life expectancy is 70 years for unirradiated humans.

#### Question #5

As the Health Physicist at a large university you have been asked to set up an in-house bioassay program to monitor biology and chemistry research workers who at various times work with up to the following quantities of radioisotopes.

Quantity and Isotope	Half-Life for Critical Organ
2 Ci of <sup>3</sup> H	$T_e = 10 \text{ days}$
200 mCi of <sup>32</sup> P	$T_{p} = 14 \text{ days}$
100 mCi of <sup>14</sup> C	$T_p = 12 \text{ days}$
50 mCi of <sup>125</sup> I	$T_a = 42 \text{ days}$

Your previous experience at this university indicates that 99% of the bioassay results are less than 1 investigation level (as defined by the ICRP).

Point Value Discuss your recommendations and reasons for the following points.

- a) What type of bioassay would you recommend for each radioisotope? (Bioassay includes any method used to evaluate internal deposition of radionuclides)
   Assume you have access to any type of counter desired.
- b) Discuss the rationale for the routine bioassay frequency you would recommend for each radioisotope.
- c) What calibration methods would you recommend for each type of bioassay analysis you choose?

#### Question #6

A biologist wearing a labcoat but no gloves was homogenizing a cell culture containing 50 mCi of 32P-phosphate. The tube shattered and uniformly contaminated both of his hands. After three scrubbings with detergent, you measured the non-removable activity with a 5 cm diameter detector at contact to be  $1.6 \times 10^6$  cpm anywhere on either hand. The worker is now on his way to your medical facility and you stop by your office to get the following data before conferring with the Institute physician.

-4-

1) From Radiological Health Handbook

 $\overline{E}_{g}$  for  $^{32}P = 0.69$  MeV, T 1/2 = 14.3 day, maximum range of  $\beta$  in tissue = 320 mg/

Standard man:

rmis	500	gm
S	4400	gm
area	18000	cm <sup>2</sup>

Assume that the thickness of the epidermis and dermis is uniform over the body.

2) From your files

C

Total detector efficiency (including geometry) = 3.0%

epide dermi skin

Effective removal half-life for 3 previous <sup>32</sup>P-phosphate hand contamination incidents was 2.7 days.

3) From Radiation Dosimetry (Hine and Brownell)

For a  $\beta$  source on an infinite thin plane inside a uniform absorbing material the dose at point (x) is:

- $D(x) = 2.66 \times 10^{-9} \overline{E}_{S\sigma} \left\{ C \left[ (1+\ln\frac{C}{vx}) \exp(1 \frac{vx}{C}) \right] + \exp(1 vx) \right\}$ expression in brackets [] = 0 if x > C/
  - D(x) = dose rate in rad/min. at depth x in gram/cm<sup>2</sup>
  - Es = avg. s energy in MeV
    - = dpm/cm2

= 1

= 9.2 cm<sup>2</sup>/gm tissue

Point Value

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- a) Calculate the maximum dose to the dermis and the subcutaneous tissue.
  - b) How do these calculated doses compare to maximum permissible doses for these tissues?
  - c) What recommendations would you make to the physician regarding initial and follow-up treatment procedures?

#### **Ouestion** 3

-2-

A composite whole milk sample collected on April 10 from cows on pasture at a dairy farm revealed the following:

Nuclide	Measured Concentration
<sup>40</sup> K	1100 pCi/l
<sup>89</sup> Sr	< 2 pCi/l
90 <sub>Sr</sub>	12 pCi/l
131	< 0.2 pCi/l
137 <sub>Cs</sub>	1.7 pCi/l
Ca	1030 mg/1

The dairy farm is located 14.7 miles ENE from a 860 MW boiling water power reactor. Prevailing winds are from the south, and X/Q values at 15 miles from the plant reveal the maximum X/Q to be in the NNW sector, decreasing radially in a counterclockwise direction. X/Q in the ENE sector is approximately 10% of the maximum X/Q. Measured rainfall in the area during the week of April 3 - 10 was 0.88 inches. Since the start of the year, reactor operations have been pormal and 24 kCi of gaseous radioactivity including 26 mCi of 13 I have been discharged to the atmosphere; approximately 8% of this amount was discharged the week of April 3 - 10.

Point Value

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- a) Discuss the significance of the radionuclide concentrations given above, and suggest an explanation for any anomalies.
- b) What sort of radionaclide distributions would you expect in milk from a dairy farm located 6 miles south, in the sector of lowest X/Q? A goat milk dairy farm 3 miles NNW? Why?
- c) Suppose that a small (100 KT) nuclear weapons test (atmospheric) had been set off in Siberia 2 weeks prior to collection of the sample. Would you expect any change? Why?

-65-

#### Question #9

Fuel fabrication facilities may be required to manufacture plotonium as well as uranium based fuels.

-6-

Point Value Discuss:

- a) The specific changes in the routine and emergency environmental monitoring programs, and
- b) The specific changes in facility design philosophy as it relates to the off-site environment

required at a nuclear fuels fabrication facility in order to fabricate plutonium fuel in addition to uranium fuel.

#### Question #10

The concentration of  $^{226}$ Ra in the atmosphere at a particular location has been measured to be  $10^{-16}$  uCi/cc on the average. The average concentration of  $^{226}$ Ra in the soil at this location is 2.2 dpm/g and it is approximately uniformly distributed in the soil. The density of the soil is 2 g/cc.

- a) Assuming a resuspension factor of  $5 \times 10^{-7}$  m<sup>-1</sup>, what is the resuspendable 226Ra activity per m<sup>2</sup>?
  - b) What is the effective thickness of the resuspendable layer of soil?
  - c) Assuming an adult inhales <sup>225</sup>Ra at the concentration measured at this location for 30 years, what would be the total integrated dose equivalent to the bone at the end of the 30 year period?
  - Given: Breathing rate =  $2 \times 10^7$  cc/day

Fraction of 226Ra inhaled reaching the bone is 0.2  $T_E$  for removal of 226Ra from bone = 1.6x10<sup>4</sup> days Dose equivalent rate to bone from 226Ra = 0.8 rem/day-µCi

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Point Value

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DISTANCE DOWNWIND (meters)

Relative axial concentration versus distance downwind by stability class -- 15 to 60 minute release time (Markee).

-8-Figure for Question 11



#### Question #12

A three room nuclear medicine department in a community hospital is conducting routine diagnostic studies with a rectilinear scanner and a gamma camera. It receives a 300 mCi Mo-99 generator each week. Other radiopharmaceuticals are obtained in individual patient doses. Radiotherapy is limited to I-131 for hyperthyroidism and thyroid cancer.

362 096

Design a continuing radiation control program for this facility. Be sure Point Value to discuss:

a) Records required

2

2

2

- b) Instrumentation needed
- c) Nursing instructions, if any
- d) Routine radiation and contamination surveillance
- 2 e) Personnel monitoring



-69-

- -10-
- 2
   2
   b) If the measurement of activity induced in tissue were to be taken in the radioactive environment of the accelerator where the y-background is a 1 mR/hr could the induced activity be detected? (The NaI detector gives 400 cps in an exposure rate of 10 µR hr<sup>-1</sup> due to <sup>225</sup>Ra y's).
  - 2

Point Value

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Point Value

2

c) What other st is would you take in investigating this incident?

#### Question #15

line containing 95% enriched uranium in solution as uranyl nitrate is Leing cut in order to install equipment to dislodge a plug in the line. As a health physicist you happen on the scene as maintenance people are cutting the line. They are dressed in coveralls, latex gloves, plastic boots, and are wearing respirators, hard hats, and goggles. The field of radiation is 350 mrem/hr and all personnel are standing around the equipment watching progress of the work. The area has been ribboned off and a plastic bag has been taped to the line around the cut area in order to prevent contaminants from splashing to the floor and adjoining equipment.

- 4 a) What is the major item of concern here?
  - b) What, if any, changes would you recommend in the procedure?

#### Question #15

You are the health physics member of a design review team responsible for evaluating the design of a fuel reprocessing plant. As means of reducing doses to personnel from radioactive material and releases of radioactive materials to off-site locations, briefly discuss the design of each of the following:

- 3 a) containment and confinement barriers
  - 3 b) shielding
  - 1 c) physical layout
    - d) ventilation
  - e) equipment design

#### ABHP EXAMINATION 22, PART II

June 19, 1978

Answer any seven

2

2

3

Total Time: 4 hours

Point Va a Question I

- Beams of protons or electrons which pass through air can produce radioactivity by interacting with air molecules. Identify the most commonly produced radionuclides and show methods of production.
- b) These radionuclides can produce exposure to several different parts of the body. Identify these parts and indicate which one is the limiting case.
  - c) What are the simplest methods of controlling radiation exposure in an occupied room from this source?

  - Data

Density of air at standard conditions =  $1.29 \times 10^{-3} \frac{g}{cm^3}$ Molecular weight of air =  $\frac{28 g}{mole}$ Avogadros Number =  $6.02 \times 10^{23} \frac{molecules}{mole}$ 

Point Value Question 2

5

- a) As a consultant to a university, you have been asked to provide recommendations for the shielding requirements of a particle accelerator. What information will be needed to provide these recommendations?
- 3

2

- b) Identify at least three types of radiation which are normally produced by the interaction of a particle beam and a target and which are significant from a health physics point of view. How is each produced?
- c) Which type of radiation would you expect for a 3 MeV electron beam interacting in a copper target? For a 30 GeV proton beam interacting in a copper target?

#### Question #3

An analyst inhaled about ten nanograms of  ${}^{35}$ S when he entered a laboratory module after a vial containing one milligram of  ${}^{35}$ S exploded. Floor surfaces in the laboratory were contaminated to 70 mCi/m<sup>2</sup>. His body surfaces were decontaminated with soap and water; contaminated body hair was removed by shaving. All urine and fecal samples were collected for  ${}^{35}$ S determinations until the concentration in the samples decreased to the limits of detection, 0.1 nCi/liter of urine or 0.1 nCi/100 grams feces.

-2-

Sample	Period Monitcred (days)	Total 355 Activity (µCi)	Fraction Eliminated %	Biological Halftime (days)
Urine	65	610	0.75	0.3
			0.25	7
Fecal	8	380	1.0	1

Point Value

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 a) Determine the dose equivalent from one millicurie of <sup>35</sup>S in the body for one day assuming the whole body to be the critical organ.
 b) Coloriate the application of the second state of the second s

b) Calculate the analyst's integrated whole body equivalent.

Given: <sup>35</sup>S data half-life = 87 days; beta energy = 0.167 MeV (max); = 0.05 MeV (ave) Quality Factor = 1.0 total body) Organ Weight, total body = 70 kg; lung = 1 kg Curie =  $3.7 \times 10^{10}$  dis/s eV = 1.602  $\times 10^{-12}$  ergs day = 8.64  $\times 10^4$  s Fraction of Systemic burden excreted in urine (Fu) = 0.9  $\int_{0}^{\infty} e^{-az} dz = \frac{1}{a}$ 

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#### -10-

#### Question #14

A 26 year old female was referred to radiology by her internist for the following examinations:

- a) Chest, PA and Lat.
- b) Barium Enema
- c) Intravenous Pyelogram

Subsequently it is discovered that this patient was two weeks pregnant at the time of the examination. As the hospital health physicist, you are consulted.

Point Value

- 5
- a) Estimate the fetal dose from each of these procedures. Describe your method and assumptions.
- b) Discuss your recommendations regarding patient management in light of your estimated fetal dose.
- 2.5

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2.5

c) What controls would you recommend be instituted in order to minimize the reoccurance of this type of problem?

#### Question #15

A graduate student is opening an irradiated quartz ampoule containing 500 Ci of 169 Yb oxide powder. He is working behind a 4 inch thick, lead glass shield and is using long tongs to unwrap the aluminium foil covering on the ampoule. He becomes impatient with the tongs, reaches aroung the shield with both hands and unwraps the foil, at which point he discovers that the ampoule is broken and the 109 Yb powder spills out. The student recognizing a potential problem. immediately stepped back from the area. Within minutes the exhaust air alarm (set for 10 X normal background at the absolute filter) sounds.

Subsequent reconstruction of the incident shows that the student's hands were close to but not less than one centimeter from the ampoule for 30 seconds. Measured dose rates were 1 rem/minute at one foot unshielded and 20 mrem/hr behind the shield. 109 Yo decays by electron capture (T1 = 32 d) emitting primarily Tm X-rays, plus 63 and 198 KeV gammas.

Discuss the following:

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5	a)	Exposure	evaluat
3	(c	Cleanup	of labor
2	- c)	Steps to	crevent
			A CONTRACTOR OF A CONTRACTOR O

- ion and management.
- atory.
  - recurrence of incident.

The continuation of stmospheric testing of nuclear devices by the People's Republic in China adds to the inventory of radiocontaminants in the northern hemisphere. This situation tends to complicate periodically the interpretation of routine radiological environmental surveillance data. This is particularly so for facilities which have been operating for some time. For the following situations discuss the methods by which you would estimate the origin of the contamination or the facility's contribution to sample activity. Assume the facility to be a pressurized light water reactor which has operated for two years and has been through two partial refuelings. Assume the weapon to be a pure fission device. Assume the time lapse between the test and sample collection to be 7 days. Assume heavy rains with the arrival of the fallout.

-4-

## Point Value

- 3.3
  - 3.3

3.3

- Silt samples collected at several downstream locations in the receiving stream showed the presence of  ${}^{58}Co$ ,  ${}^{60}Co$ ,  ${}^{134}Cs$ ,  ${}^{137}Cs$ ,  ${}^{140}Ba$ ,  ${}^{141}Ce$ ,  ${}^{95}Zr$ ,  ${}^{95}Nb$ ,  ${}^{131}I$ , and  ${}^{103}Ru$ . a)
- Milk samples collected at several local farms showed significant concentrations of  $^{131}{\rm I}$  ,  $^{132}{\rm I}$  ,  $^{89}{\rm Sr}$  , with no significant change b) in 90Sr and 137Cs.
- c) Composited raw domestic water collected at a downstream intake showed elevated tritium and small quantities of 58Co, 60Co, 54 Mn., and 140Bala.

#### Question #8

You are a health physics consultant for a uranium mining and milling company. Last night, at 11:00 p.m., at the Victorio Peak uranium mill, a retention dam broke releasing 2 million gallons of tailings water and slimes. About 50 acres of land adjacent to the mill have been flooded. The Shift Supervisor ordered a temporary berm thrown up which has confined the flow to land controlled by the mill. The flow stopped about 1/2 mile short of the small community of Black Sands (about 50 residents). The mill has been shut down.

The mill is located in a dry part of the country subject to frequent periods of high winds. Drinking, farm, and mill process waters are obtained from shallow aquifers.

int Value

4

- What program would you initiate to assess the potential radioa) logical impact of the tailings spill upon the environment? b) What actions would you recommend to the mill operators to 362 102
- minimize and to control the impact of the spill? c) What radioisotopes are of major concern?

#### Question #12

Possible accidents at power reactors have received considerable attention in the past several years. WASH-1400 (Rasmussen Report) is an in-depth investigation of potential accidents at power plants. Answer 4 of the following 5 sections:

-8-

- Point Value 2.5 per section
- a) Explain the event tree and fault tree technique used in the report. (What is the relationship between them?)
- b) What are the three sets of probabilities which are combined to reach the final probability of a specific consequence (death or injury) on the population?
- c) This report develops some new values for LD<sub>50</sub>/<sub>50</sub>. Approximately, what are these values, and what are the reasons for having more than one?
- d) What doses are combined to arrive at the total dose used to evaluate mortality rates?
- e) Based on this report, what natural phenomenon (or group of natural phenonmena) has an equivalent probability of mortality as do 100 operating nuclear power plants?

#### Question #13

A stainless steel bolt has come loose from one of a reactor vessel internals. Plans are to pick the bolt up with a remote set of tongs and and bring it up out of the water for local visual inspection and then send it off-site for metalurgical inspection. This bolt has been in the reactor for 910 effective full power days. The thermal neutron flux in this portion of the reactor is calculated to be 2.1 E 12 n/cm<sup>2</sup> - sec. The reactor will have been shut down for 17 days at the time the bolt will be removed. From drawings, the bolt is calculated to weigh 215 grams. The composition of the bolt is:

Iron	-		8	0	%
Nickel			1	9	30
Manganese		0	į	5	30
Carbon		0		5	20

## Point Value

Calculate the gamma radiation level expected at 12 inches from the bolt in air. (Use data from that provided on the attached table. 59 Co contamination in mickel and iron should be neglected. Avogadro's number = 6.022 E 23.)

#### Ouestion #10

A researcher in the Physics Department wishes to construct a photo neutron source of  $^9\text{Be}$  and  $^{124}\text{Sb}$ . He plans to use a very small sealed capsule of  $^{124}\text{Sb}$  surrounded by 30 cm of  $^9\text{Be}$ .

-6-

- Point Value
  - 8
- Calculate the approximate radiation dose rate from this source. Show calculations.
- 2

b) Discuss the nature of the shielding you would recommend.

#### Data:

10 Cl of <sup>124</sup>Sb <sup>9</sup>Be (y,n) cross section = 1 millibarn <sup>9</sup>Be (y,n) threshold = 1.66 MeV Be density = 1.8 g/cm<sup>3</sup>

124 Sb decay Scheme:

Energy	( <u>MeV</u> )	Sec. 1	%
0.6	503	2.4	97
0.6	544		7
1.6	592		50
2.0	880		7

Neutron energy = 24 KeV

 $400 \times 10^6 \text{ n/cm}^2 = 1 \text{ rem}$ 

362,04

#### Ouestion #10

A researcher in the Physics Department wishes to construct a photo neutron source of  ${}^9\text{Be}$  and  ${}^{124}\text{Sb}$ . He plans to use a very small sealed capsule of  ${}^{124}\text{Sb}$  surrounded by 30 gm of  ${}^9\text{Be}$ .

Point Value

4.1

8

 a) Calculate the approximate radiation dose rate from this source. Show calculations.

2

b) Discuss the nature of the shielding you would recommend.

-6-

#### Data:

10 Ci of 124Sb	<sup>9</sup> Be (y,n) cross section = 1 millibarn
	<sup>9</sup> Be (y, n) threshold = 1.66 MeV
	Pe density = 1.8 g/cm <sup>3</sup>

124 Sb decay Scheme:

Ene	rgy (MeV)	%
	0.603	97
	0.644	7
	1,692	50
	2.088	7

Neutron energy = 24 KeV 400 X  $10^6$  n/cm<sup>2</sup> = 1 rem

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#### Question #12

Possible accidents at power reactors have received considerable attention in the past several years. WASH-1400 (Rasmussen Report) is an in-depth investigation of potential accidents at power plants. Answer 4 of the following 5 sections:

- 8-

- Point Value 2.5 per section
- a) Explain the event tree and fault tree technique used in the report. (What is the relationship between them?)
- b) What are the three sets of probabilities which are combined to reach the final probability of a specific consequence (death or injury) on the population?
- c) This report develops some new values for LD50/60. Approximately, what are these values, and what are the reasons for having more than one?
- d) What doses are combined to arrive at the total dose used to evaluate mortality rates?
- e) Based on this report, what natural phenomenon (or group of natural phenonmena) has an equivalent probability of mortality as do 100 operating nuclear power plants?

#### Question #13

A stainless steel bolt has come loose from one of a reactor vessel internals. Plans are to pick the bolt up with a remote set of tongs and and bring it up out of the water for local visual inspection and then send it off-site for metalurgical inspection. This bolt has been in the reactor for 910 effective full power days. The thermal neutron flux in this portion of the reactor is calculated to be 2.1 E 12 n/cm<sup>2</sup> - sec. The reactor will have been shut down for 17 days at the time the bolt will be removed. From drawings, the bolt is calculated to weigh 215 grams. The composition of the bolt is:

Iron		80%
Nickel		19%
Manganese	***	0.5%
Carbon		0.5%

## Point Value

Calculate the gamma radiation level expected at 12 inches from the bolt in air. (Use data from that provided on the attached table. 59 Co contamination in mickel and from should be neglected. Avogadro's number = 6.022 E 23.)

#### Question #7

The continuation of atmospheric testing of nuclear devices by the People's Republic in China adds to the inventory of radiocontactinants in the northern hemisphere. This situation tends to complicate periodically the interpretation of routine radiological environmental surveillance data. This is reticularly so for facilities which have been operating for some time or the following situations discuss the methods by which you wou i estimate the origin of the contamination or the facility's contribution to sample activity. Assume the facility to be a pressurized light water reactor which has operated for two years and has been through two partial refuelings. Assume the weapon to be a pure fission device. Assume the time lapse between the test and sample collection to be 7 days. Assume heavy rains with the arrival of the fallout.

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Point Value 3.3

3.3

3.3

- a) Silt samples collected at several downstream locations in the receiving stream showed the presence of <sup>58</sup>Co, <sup>60</sup>Co, <sup>134</sup>Cs, <sup>137</sup>Cs, <sup>140</sup>Ba, <sup>141</sup>Ce, <sup>95</sup>Zr, <sup>95</sup>Nb, <sup>131</sup>I, and <sup>103</sup>Ru.
- b) Milk samples collected at several local farms showed significant concentrations of <sup>131</sup>I, <sup>132</sup>I, <sup>89</sup>Sr, with no significant change in <sup>90</sup>Sr and <sup>137</sup>Cs.

c) Composited raw domestic water collected at a downstream intake showed elevated tritium and small quantities of <sup>58</sup>Co, <sup>60</sup>Co, <sup>54</sup>Mn, and <sup>140</sup>BaLa.

#### Question #8

You are a health physics consultant for a uranium mining and milling company. Last night, at 11:00 p.m., at the Victorio Peak uranium mill, a retention dam broke releasing 2 million gallons of tailings water and slimes. About 50 acres of land adjacent to the mill have been flooded. The Shift Supervisor ordered a temporary berm thrown up which has confined the flow to land controlled by the mill. The flow stopped about 1/2 r is short of the small community of Black Sands (about 50 residents). The mill has been shut down.

The mill is located in a dry part of the country subject to frequent periods of high winds. Drinking, farm, and mill process waters are obtained from shallow aquifers.

Point Value		
4	a)	W
4	(ď	W
3	c)	W

What program would you initiate to assess the potential radiological impact of the tailings spill upon the environment? What actions would you recommend to the mill operators to

minimize and to control the impact of the spill? What radioisotopes are of major concern?

- 56 -

-10-

#### Question #14

A 26 year old female was referred to radiology by her internist for the fellowing examinations:

- a) Chest, PA and Lat.
- b) Barium Enema
- c) Intravenous Pyelogram

Subsequently it is discovered that this patient was two weeks pregnant at the time of the examination. As the hospital health physicist, you are consulted.

- a) Estimate the fetal dose from each of these procedures. Describe your method and assumptions.
- b) Discuss your recommendations regarding patient management in light of your estimated fetal dose.
  - c) What controls would you recommend be instituted in order to minimize the reoccurance of this type of problem?

#### Question #15

A graduate student is opening an irradiated quartz ampoule containing 500 Ci of 169 Yb ox de powder. He is working behind a 4 inch thick, lead glass shield and is using long tongs to unwrap the aluminium foil covering on the ampoule. He becomes impatient with the tongs, reaches aroung the shield with both hands and unwraps the foil, at which point he discovers that the ampoule is broken and the 199 Yb powder spills out. The student recognizing a potential problem immediately stepped back from the area. Within minutes the exhaust air alarm (set for 10 X normal background at the absolute filter) sounds.

Subsequent reconstruction of the incident shows that the student's hands were close to but not less than one centimeter from the ampoule for 30 seconds. Measured dose rates were 1 rem/minute at one foot unshielded and 20 mrem/hr behind the shield. 169yb decays by electron capture (T1 = 32 d) emitting primarily Tm X-rays, 362 108 plus 63 and 198 KeV gammas.

#### Point Value

Foint Value

2.5

2.5

Discuss the following:

5	a)
3	ic)
2	(7)

- Exposure evaluation and management.
- Cleanup of laboratory.
- Steps to prevent recurrence of incident.
#### Question #3

An analyst inhaled about ten nanograms of  ${}^{35}S$  when he entered a laboratory module after a vial containing one milligram of  ${}^{35}S$  exploded. Floor surfaces in the laboratory were contaminated to 70 mCi/m<sup>2</sup>. His body surfaces were decontaminated with soap and water; contaminated body hair was removed by shaving. All urine and fecal samples were collected for  ${}^{35}S$  determinations until the concentration in the samples decreased to the limits of detection, 0.1 nCi/liter of urine or 0.1 nCi/100 grams feces.

-2-

Sample	Feriod Monitored (days)	Total 355 Activity (µCl)	Fraction Eliminated %	Biological Halftime (days)
Urine	65	610	0.75	0.3
			0.25	7
Fecal	8	380	1.0	1

Point Value

4

6

 a) Determine the dose equivalent from one millicurie of <sup>35</sup>S in the body for one day assuming the whole body to be the critical organ.
 a) Calculate the analystic integrated whole body equivalent.

b) Calculate the analyst's integrated whole body equivalent.

Given: <sup>35</sup>S data nalf-life = 87 days; beta energy = 0.167 MeV (max); = 0.05 MeV (ave) Quality Factor = 1.0 (total body) Organ Weight, total body = 70 kg; lung = 1 kg Curie =  $3.7 \times 10^{10}$  dis/s eV = 1.602 X 10<sup>-12</sup> ergs day =  $8.64 \times 10^4$  s Fraction of Systemic burden excreted in urine (Fu) = 0.9  $\int_{0}^{\infty} e^{-3z} dz = \frac{1}{a}$ 

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## ABHP EXAMINATION 22, PART II

June 19, 1978

Answer any seven

Total Time: 4 hours

Point Value	Question 1
3	a) Beams of protons or electrons which pass through air can produce radioactivity by interacting with air molecules. Identify the most commonly produced radionuclides and show methods of production.
2	b) These radionuclides can produce exposure to several different parts of the body. Identify these parts and indicate which one is the limiting case.
2	c) What are the simplest methods of controlling radiation exposure in an occupied room from this source?
3	d) Given that the production cross-section for one of these radionuclides is 60 mb, calculate the equilibrium concentration in a room of 100 m <sup>2</sup> and no ventilation for a $6.25 \times 10 \frac{13 \text{ p}}{13 \text{ p}}$ proton beam and a 1 m air gap. Assume a 1 cm <sup>2</sup> beam. Specify which reaction you have chosen.
	Data
	Density of air at standard conditions = $1.29 \times 10^{-3} - \frac{g}{3}$
	Molecular weight of air = $\frac{28 \text{ g}}{\text{mole}}$ cm <sup>-</sup>
	Avogadros Number = $6.02 \times 10^{23} \frac{\text{molecules}}{\text{mole}}$
Point Value	Question 2
5	<ul> <li>As a consultant to a university, you have been asked to provide recommendations for the shielding requirements of a particle accelerator. What information will be needed to provide these recommendations?</li> </ul>
3	b) Identify at least three types of radiation which are normally produced by the interaction of a particle beam and a target and which are significant from a health physics point of view. How is each produced?
2	c) Which type of radiation would you expect for a 3 MeV electron beam interacting in a copper target? For a 30 GeV proton beam interacting

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in a copper target?

ABHP Exam #20

 $\begin{array}{c} \underline{Point \ Value} \\ 2 \end{array} b) \ \ \mbox{If the measurement of activity induced in tissue were to be taken in the radioactive environment of the accelerator where the <math>\gamma$ -background is  $\sim 1 \ mR/hr$  could the induced activity be detected? (The NaI detector gives 400 cps in an exposure rate of 10  $\mu R \ hr^{-1}$  due to  $\frac{220}{Ra} \ \gamma$ 's). }

2

c) What other steps would you take in investigating this incident?

-10-

#### Question #15

A line containing 95% enriched uranium in solution as uranyl nitrate is being cut in order to install equipment to dislodge a plug in the line. As a health physicist you happen on the scene as maintenance people are cutting the line. They are dressed in coveralls, latex gloves, plastic boots, and are wearing respirators, hard hats, and goggles. The field of radiation is 350 mrem/hr and all personnel are standing around the equipment watching progress of the work. The area has been ribboned off and a plastic bag has been taped to the line around the cut area in order to prevent contaminants from splashing to the floor and adjoining equipment.

- Point Value
  - a) What is the major item of concern here?
  - 6

Point Value

3

4

- b) What, if any, changes would you recommend in the procedure?

#### Question #15

You are the health physics member of a design review team responsible for evaluating the design of a fuel reprocessing plant. As means of reducing doses to personnel from radioactive material and releases of radioactive materials to off-site locations, briefly discuss the design of each of the following:

- 3 a) containment and confinement barriers
  - b) shielding
  - c) physical layout
  - 2 d) ventilation
  - e) equipment design



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-6-

Figure for Question 11

-8-



#### Question #12

A three room nuclear medicine department in a community hospital is conducting routine diagnostic studies with a rectilinear scanner and a gamma camera. It receives a 300 mCi Mo-99 generator each week. Other radiopharmaceuticals are obtained in individual patient doses. Radiotherapy is limited to I-131 for hyperthyroidism and thyroid cancer.

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113

Design a continuing radiation control program for this facility. Be surePoint Valueto discuss:

a) Records required
b) Instrumentation needed
c) Nursing instructions, if any
d) Routine radiation and contamination surveillance
e) Personnel monitoring

POOR ORIGINAL 102 s 103 RELATIVE AXIAL CONCENTRATION  $X \overline{u}/Q$  (meters<sup>-2</sup>) 10 105 100 107 ð 10°2 103 104 10

DISTANCE DOWNWIND (meters)

Relative axial concentration versus distance downwind by stability class --15 to 60 minute release time (Markee). 362 114 Fuel fabrication facilities may be required to manufacture plutonium as well as uranium based fuels.

-6-

Point Value Discuss:

. . .

- The specific changes in the routine and emergency environmental monitoring programs, and
- b) The specific changes in facility design philosophy as it relates to the off-site environment

required at a nuclear fuels fabrication facility in order to fabricate plutonium fuel in addition to uranium fuel.

#### Question #10

The concentration of  $^{226}$ Ra in the atmosphere at a particular location has been measured to be  $10^{-16}$  µCi/cc on the average. The average concentration of  $^{226}$ Ra in the soil at this location is 2.2 dpm/g and it is approximately uniformly distributed in the soil. The density of the soil is 2 g/cc.

Point Value

2

5

7

3

- a) Assuming a resuspension factor of  $5 \times 10^{-7}$  m<sup>-1</sup>, what is the resuspendable 226Ra activity per m<sup>2</sup>?
- b) What is the effective thickness of the resuspendable layer of soil?
- c) Assuming an adult inhales <sup>225</sup>Ra at the concentration measured at this location for 30 years, what would be the total integrated dose equivalent to the bone at the end of the 30 year period?
- Given: Breathing rate = 2x107 cc/day

Fraction of 226Ra inhaled reaching the bone is 0.2  $T_E$  for removal of 226Ra from bone = 1.6x10<sup>4</sup> days Dose equivalent rate to bone from 226Ra = 0.8 rem/day-pCi

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#### Ouestion 3

A composite whole milk sample collected on April 10 from cows on pasture at a dairy farm revealed the following:

Nuclide	Measured Concentration
40K	1100 pCi/l
89 <sub>Sr</sub>	< 2 pCi/l
90 <sub>Sr</sub>	12 pCi/l
131 <sub>I</sub>	< 0.2 pCi/l
137 <sub>Cs</sub>	1.7 pCi/l
Ca	1030 mg/l

The dairy farm is located 14.7 miles ENE from a 860 MW boiling water power reactor. Prevailing winds are from the south, and X/Q values at 15 miles from the plant reveal the maximum X/Q to be in the NNW sector, decreasing radially in a counterclockwise direction. X/Q in the ENE sector is approximately 10% of the maximum X/Q. Measured rainfall in the area during the week of April 3 - 10 was 0.88 inches. Since the start of the year, reactor operations have been normal and 24 kCi of gaseous radioactivity including 26 mCi of  $13^{11}$  I have been discharged to the atmosphere; approximately 8% of this amount was discharged the week of April 3 - 10.

- Point Value
  - 6

2

2

- a) Discuss the significance of the radionuclide concentrations given above, and suggest an explanation for any anomalies.
- b) What sort of radionuclide distributions would you expect in milk from a dairy farm located 6 miles south, in the sector of lowest X/Q? A goat milk dairy farm 3 miles NNW? Why?
- c) Suppose that a small (100 KT) nuclear weapons test (atmospheric) had been set off in Siberia 2 weeks prior to collection of the sample. Would you expect any change? Why?

-2-

#### Question 10

A geology research associate is studying brine solubilization of transuranics as part of a bedded salt waste disposal program proposal. 238 Pu, <sup>243</sup> Am, <sup>244</sup> Cm, <sup>245</sup> Cf. You, as University RSO, receive at 4 p.m. a telephone call that there has been a small fire in the glovebox from a solvent extraction process. On arriving at the scene, you observe through the lab door window that the glovebox apppears undamaged except for possible leaks around the gloves where they mate with the box. It is ascertained that the glovebox filters are intact. The lab room exhaust ventilation system is still working because there is air flow under the door into the lab. Previous studies with smoke tubes have shown the mixing factor in the room to be 0.1. From your experience, you estimate that a maximum of 10% of the material has been released from the glovebox. The room is 20 feet by 20 feet by 10 feet high, and the exhaust flow rate is 4000 ft<sup>2</sup>/min.

Point Value

3

. .

- a) The experimenter is quite concerned and wants to reenter the room immediately (30 minutes after the incident) to shut down a valuable piece of equipment. Calculate the room air concentrations and state the appropriate protective clothing and equipment necessary to do this job.
- b) Comment on the overall reentry problem from the standpoint of timing and preplanning.

Radionuclide	MPC <sub>a</sub> (µCi/cm <sup>3</sup> )
Pu-238	2 x 10 <sup>-12</sup>
Am-243	6 x 10 <sup>-12</sup>
Cm-244	9 x 10 <sup>-12</sup>
Cf-240	2 x 10 <sup>-12</sup>

on

Ventilation Equation

C = C.	$e^{-kQt}V$	where	
	k = m Q = f	nixing factor low rate	
	V = v t = ti	olume me	

#### Respiratory protection factors

- a) Half mask 10
- b) Full-face mask 50
- c) Airline respirator

half mask	1000
full-face	2000
	-71-

362 11

-9-

d) Self-contained breathing apparatus

pressure demand 10,000 demand 50

Permissible emergency excursion factor above MPC, is 5

#### Question 11

You are responsible for monitoring radioactive shipments at a waste management facility. A large shielded iron cask containing stainless steel scrap from a reactor storage basin was positioned at 50 feet from a high resolution gamma monitor and a 10-minute count showed photopeaks characteristic of <sup>6</sup>Co.

1173.2 KeV: 2960 counts (net)

1332.5 KeV: 5150 counts (net)

Point Value

a) Determine the apparent thickness of the cask assuming a point source and no significant buildup factor.

Linear Absorption

5

b) Estimate <sup>60</sup>Co activity in the cask.

Nuclide	Energy (KeV)	Photon Yield	Detector Efficiency*	Coefficient-iron (cm <sup>-1</sup> )
54 <sub>Mn</sub>	835	100	5.6 x 10 <sup>-9</sup>	0.5112
60 <sub>Co</sub>	1173	100	$4.1 \times 10^{-9}$	0.4335
60 <sub>Co</sub>	1332	100	3.5 x 10 <sup>-9</sup>	0.4058

\*Absolute efficiency at 50 feet in air (photons counted/photons emitted)

### Question 12

Point Value

You have been hired by a large nuclear facility as a consultant to develope a respiratory protection program. Assume that the design of the facility has been reviewed and approved by a panel of certified health physicists and that this panel has assured that engineering controls have been instituted wherever practicable to minimize the inhalation of radioactive material. However, they recommend that the use of respiratory protective devices will be necessary in addition for certain routine, non-routine and emergency operations. Discuss the elements of an acceptable respiratory protection program.

#### Question 13

A worker was found to have spent an entire 8-hour shift in an area behind a wall adjacent to a radiography operation. The X-ray machine in use was beamed down at the floor and operated at 220 kVp and 22 mA; the worker was about 10 feet from the tube head with only a plaster wall as a shield. The measured exposure rate at the worker's location was obtained by the radiographer with a thin metal (30 mg/cm<sup>2</sup> Fe) wall GM tube used in open window mode and was 27 mR/hr. The worker's TLD (LiF) badge was immediately processed and interpreted as reading 30 mrem; a pocket chamber (200 mR full scale) on the wall behind the worker was off-scale, and a TLD in the same location read 50 mrem.

- Point Value
  - 3

4

- a) What dose would you assign to the worker? Explain your basis.
- b) What additional data would you gather, if any, to establish or verify the dose?
- c) Explain the discrepancy between the TLD and other readings; would you recommend investigation of the TLD badge processor for accuracy, and if so, how might this be done?

#### Question 14

An air sampler with a flow rate of 2 cfm, operated for 24 hours sampling the discharge from a stack. The stack gases contain both Fe and I, and the stack discharge rate is 6000 cfm.

The filter was counted immediately after removal from the sampler and the gross count was 11,280 cpm. Two days later the gross count was 10,666 cpm. The counter background is 100 counts per hour, counting efficiency is 12%, and filter collection efficiency is 85% for <sup>9</sup>Fe and 50% for <sup>13</sup>I.

- a) Calculate the concentration of <sup>59</sup>Fe and <sup>131</sup>I in the stack effluent.
- b) Given that the iodine is in the elemental form and that the iron is attached to particulates, what type of filter media would you use in this air sampler?
- c) Given the MPC for <sup>59</sup>Fe =  $5 \times 10^{-9} \mu$ Ci/cm<sup>3</sup> and for <sup>131</sup>I =  $1 \times 10^{-10} \mu$ Ci/cm<sup>3</sup>, what recommendations would you make concerning this stack effluent?
- 2

Point Value

5

1

2

d) Do you think your answer is "statistically significant"? Why?

Data

 $T_{y_2}$  for <sup>59</sup>Fe = 45 days  $T_{y_2}$  for <sup>131</sup>I = 8.05 days 1 cu. ft. = 28,000 cm<sup>3</sup>

#### Question 15

A special maintenance job at a 1230 MWe unit BWR power plant involves rebuilding a reactor water cleanup pump. The procedure calls for removal of the pump from its isolation room to a temporary plastic service tent where most of the work will be performed. Initial surveys indicate no detectable airborne radioactivity, surface contamination levels below the minimum value for a contamination zone, and gamma fields of 200 mR/hr at one meter and 60 mR/hr in the general working area.

Removal from the isolation room can be performed by teams of three men in four hours or six men in two hours.

- a) What is the approximate dose equivalent (individual and integrated) that the men could receive?
- b) Based on health physics principles which alternative is preferable? Why?

The service tent is located 15 feet from a crud trap (a length of piping 4' long by 10" diameter) three feet above the floor.

- c) What thickness would a concrete shield wall need to be to reduce the exposure rate at 15 feet from 0.25 R/hr to 5 mR/hr? (Neglect scatter acound the wall. Assume that the predominant nuclide in the crud is Co. Show all calculations.)
- d) Approximately how many Curies are contained in the crud assuming it is all Co?
- e) As plant health physicist what would be your recommendation concerning this source of radiation exposure?

	Concret	e Build	up Fact	ors for a	Point Isotopic Source
			μx		
MeV	1	2	4	7	
1.0	1.97	3.18	6.22	12.3	
2.0	1.75	2.59	4.49	7.74	Dose Buildup Factors
1.0	2.09	3.50	7.02	13.9	
2.0	1.79	2.69	4.71	8.16	Energy Absorption Buildup Factors

Broad-Beam <sup>60</sup> Co Gamma Dose Transmission	Concrete Shield Thickness (inches)
0.1	11
0.01	19
0.001	27

262,20

-74-

-11-

.

1

Point Value

2

- 2

Concrete
0.354
0.337
0.321
0.306

Half-life of Co = 5.26 years

<sup>60</sup>Co gamma energies 1.17 MeV

1.33 MeV

#### Question 16

You are the health physicist in a nuclear power station. A certified welder has received an unknown, unplanned exposure. His 200 mR pocket chamber is off-scale and it will take two hours to have his TLD badge read. In addition, he received skin contamination over parts of his body, the most significant of which is 6,700 dpm on his face. The welder is a transient worker and you have not received written confirmation of exposure from previous employers as shown on the worker's NRC-4. However, it is the first week in the quarter and you are certain that he has recived no radition exposure during this period at a facility other than yours. His accumulated pocket chamber readings at your facility for the month are 600 mR prior to this exposure.

Point Value

4

1.01

- a) This welder is critical to the repair of the system. Consequently, you are being pressed by plant management to permit him to return to work prior to receiving the TLD badge results. Would you allow the welder to return to work in radiation and/or contamination zones? Why?
- b) Assume that you have made the decision to make an in vivo count of the worker. The results of this count are:

40

1.4.1

The activity is to be considered as the total activity in the worker's body.

The data for these radionuclides are:

131,

Critical organ = thyroid  $\epsilon = \Sigma EF (RBE) n = 0.23 MeV$  for thyroid Maximum permissible burden in total body = 0.7 µCi Physical  $T_{y_1} = 8$  days Biological  $T_{y_1} = 138$  days Thyroid weight = 20 grams Thyroid size = 3 cm  $f_2 = 0.2$ 60 Co: Critical organ = whole body  $\epsilon = \Sigma EF (RBE) n = 1.5 MeV$ Maximum permissible burden in total body = 10  $\mu$ Ci Physical  $T_{y_1} = 1.9 \times 10^3$  days Biological  $T_{y_2} = 9.5$  days Body weight = 7 x 10<sup>-7</sup> grams Body size = 30 cm  $f_2 = 1$ 

Possibly useful formulas for calculating doses:

Dose (rem) = 
$$\frac{51.2 \quad \epsilon f_2}{m} \int q(t) dt$$
  
Dose (rem) =  $\frac{51.2 \quad \epsilon q f_2}{m \lambda}$ 

Dose (rem) = 
$$\frac{1}{m}$$

Calculate the dose commitment to the critical organ. How much of this dose would you assign to the first quarter?

## Point Value

2

2

c)

What additional things would you recommend concerning this internal exposure?

362 122

The TLD badge for the worker read 2.0 rems. Do you have an d) overexposure that must be reported to the NRC as specified in 10CFR 20?

#### Section 6

#### Part II - Answers to Typical Questions

Seven questions have been selected from Part II of recent exams and an acceptable answer for each question is given. It must be recognized that other answers or modified versions of the answers given may be equally acceptable. In grading questions, the Examining Panel is looking for professional attitude, technical approach, organization, justification of assumptions and logical reasoning. Thus, variations on answers are acceptable as long as they are well supported; however, correct numerical answers to calculational problems are expected to obtain a perfect score.

- 1. Accelerator Exam 18, Question 8
- Radiation sources could be any material the beam could strike such as:
  - Collimators Magnets Beam pipe
  - a. Under normal operating conditions there could be beam loss caused by the special in the electron momentum causing a portion of the beam strike material. This would be a small continuous loss generating high energy photons and photoneutrons.

There would also be a continuous radiation of photons because of synchrotron radiation in the bend. These would be low energy.

- A failure in the beam transport system such as a magnet failure will dump the entire beam into some material (listed above). This would generate a point source of high energy photons and neutrons.
- II. The shielding should be high Z material around the beam pipe to reduce the photon and high energy neutron intensity as quickly as possible. This must be followed by low Z material (concrete) to absorb the moderated neutrons.
- III. a. Calculate beam power

 $P = 2.5 \times 10^{7} \text{ (eV) } \times 1 \text{ (amp) } \times 2 \times 10^{-6} \text{ (sec) } \times 3.6 \times 10^{2} \text{ (sec}^{-1)}$ = 1.8 × 10<sup>4</sup> watts

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Continuous beam loss of 0.1 % gives 18 watts. Thus, the rate of energy lost = 18 joule/sec. At 360 pulses/second, this amounts to 0.05 joule/pulse.

A single failure dumping the entire beam at one point results in  $1.8 \times 10^4$  watts or 50 joule/pulse. If the beam is turned off within 2 pulses the total energy lost is 100 joules. Thus, if there is less than one failure every 2000 pulses, the continuous beam loss dominates. It is reasonable to expect that a failure every 2000 pulses is intolerable from an operational standpoint.

Activation of machine parts is generally not a significant shielding ь. problem in electron accelerators compared with the shielding reguired for the machine operation. Radiation from activated parts is of much lower energy and more readily shielded than beamproduced radiation. However, local shielding of activated parts may be necessary for personnel access during maintenance periods.

> Environmental - Exam 17, Question 2 2.

#### 1. Air Monitoring

<sup>40</sup>K and cosmic radiation are not significant factors in air monitoring. Thoron and radon and their daughters may contribute significantly to the activity observed by an air monitor, and their associated alpha activities make it quite difficult to monitor air at MPC levels or below for more hazardous alpha emitters.

#### Sample Counting 2.

Cosmic radiation contributes significantly to the background counting rate of low-level beta and gamma counting equipment, even though they may be well shielded from effects of terrestial radiation.

#### In Vivo Counting 3.

The human subjects contain significant quantities of <sup>40</sup>K and may have in or on their bodies some radon daughters. The equipment in the counting chamber may contain <sup>40</sup>K, U and Th. The air in the chamber may contain radon and thoron plus their daughters. Some cosmic radiation will penetrate into the counting chamber.

#### Radiation Background Measurements 4.

362 .124 All four will contribute to background measurements making it difficult to detect small contributions to background radiation from other activities.

5. Calibration of Low-Level Instruments

Because all four contribute in some degree to background radiation, one cannot obtain "zero" background for calibrating instruments.

6. Materials for Construction and Shielding

Almost all soils and masonry materials contain  $^{40}\mathrm{K}$  and U. Some contain Th.

7. Radiochemical Analyses and Materials and Equipment Used

Radon and thoron daughters may be contaminants in low-level laboratories.  $^{40}{\rm K}$  and isotopes of the U and Th series may derive from materials such as glassware, ceramics, etc., and some of the chemical reagents will likely contain  $^{40}{\rm K}$  and some of the isotopes of the U and Th series.

#### 3. Fuel Cycle - Exam 19, Question 15

Work on each of the three streams will require work area preparation, wearing of at least a "basic set" of protective clothing including rubber gloves, the use of a Radiation Work Permit, bagging of tools for decontamination at the end of the job and continuous health physics coverage during at least the opening of the pump. Other specific considerations for each stream are:

a. Uranium Stream

The main problems in working on a pump in this stream will be centered around the slight fission product contamination that may still be in the uranium stream. In addition, although the U-235 enrichment will be in the range of the few percent, there is a chance (under unusual circumstances) such as draining a long length of small diameter piping into a large container) that considerations will have to be given to potential criticality problems. The work area should be papered and absorbent paper placed under the pump to absorb any leakage when the pump is opened. Full-face filter masks should be worn until the pump is opened and air samples are taken. If air samples are < MPC, mask requirements can be removed. When pump is opened, beta and hand exposure rates should be evaluated. Job should require health physics coverage at the start of the job, at the time pump is opened, and after work area cleanup.

b. Plutonium Stream

The main problems in working on a pump in this stream will be centered around potential for rapid spread of contamination, potential for ingestion and criticality considerations. If pump is to be drained into an

exterior container, care must be taken that the container is critically safe for the solution to be drained. (Keep in mind that solids may have collected in pump which have the effect of making the solution more concentrated than the normal stream concentration.) A plastic hut or tent should be built around the work area to contain the contamination. The hut or tent should have a separately enclosed exit area for personnel to use for removal of their outermost protective clothing. The work area should be papered and cove. "d with plastic. Absorbant paper or absorbent pads should be used to collect any drips when pump is opened. If pump is not in a cabinet under negative pressure, a filtered exhaust system should exhaust air from the hut or tent in a direction from the pump and away from the workers. Workers should wear at least a double set of clothing and an air-supplied full-face mask with all joints taped. Serious consideration should be given to using an air-supplied plastic suit over one set of basic clothes. Special care should be given to protecting any cuts or breaks in the skin before protective clothing is put on. Air samples should be taken both inside and outside of the hut or tent at several times during the job (or CAM should be used to continuously monitor working zone air within hut). Once pump is opened, exposed surfaces should be decontaminated with absorbent pads to prevent contamination from drying out and becoming airborne. At the end of the job, all exterior surfaces of pump should be decontaminated before hut or tent is taken down. This job will require continuous health physics coverage.

#### c. Fission Product Stream

The main problems in working on a pump in this stream will be centered around high radiation levels and potential for spread of contamination. Depending on a comparison of man-rem dose estimates for installing and removing shielding with the reduction in exposure afforded the work crew by having shielding in place, temporary shielding should be installed on the pump suction and discharge piping and on other equipment affecting dose rates in the work area. The work area should be papered and covered with plastic. Consideration should be given to using a simple hut or tent if pump is in a large room and dose rates will permit construction of the hut without undue exposure. Absorbent paper should be placed under the pump to collect any leakage when the pump is opened. When the pump is opened, beta and hand exposure rates must be evaluated (exposure rates may well increase by a factor of 3 to 20). Once pump is opened, exposed surfaces should be decontaminated with absorbent pads held with tongs to minimize hand exposures. Personnel should wear an air-supplied full-face mask with at least one set of protective clothing and double rubber gloves with all joints taped. Consideration should be given to wearing a double set of coveralls depending on the actual work to be performed. Air samples should be taken during the pump opening, and at several times during the

subsequent work. Job should require health physics coverage at the start, at the time the pump is opened and after the work area cleanup. If contamination levels are very high, continuous coverage might be required once the pump is opened.

4. Medical - Exam 20, Question 11

a. Parameters:

$$K = \frac{P(dose)}{WUT}$$

P = 0.1 rem/wk maximum permissible dose

d = 2 m

W = (20 pts/day) (5 d/wk) (4 films/pt) (100 As/film) (1 m/60sec)

= 666 mAmin/wk

U = 1/16 for a radiographic installation

T = 1 for a controlled area

WUT = 41.6

$$K = \frac{(.1)(2)^2}{41.6} = 9.6 \times 10^{-3}$$

from the graph: 0.75 mm Pb

b. 200 mR/hr = 3.33 mR/min

= 0.056 mR/sec

Since the average is 100 mA sec per film, the average time at 200 mA is 0.5 sec.

If all 100 pts received 4 chest views, each requiring a 0.5 sec exposure, the beam-on time at wall A will be:

(100 pts)(4 views/pt.)(.5 sec/v.) = 200 sec/wk

(200 sec/wk)(0.056 mR/sec) = 11.2 mR/wk

The MPD for this area is 100 mR/wk; therefore, 200 mR/hr is not excessive.

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5. Power Reactors - Exam 20, Question 8

a. Total activity builtup on a demineralizer can be calculated using:

$$\mu Ci = \frac{\mu Ci/\min}{\lambda} (1 - e^{-\lambda t})$$

Where: µCi/min is the input activity rate =

uCi/ml x 600 1/min x 1000 ml/1

t = 90 days = 1.30(5) min

Ra	adionuclide	Halflife	$\lambda$ (min <sup>-1</sup> )	<u>uCi/min</u>	$1-e^{-\lambda t}$	90 days
	Co-60	$5.26y = 2.76(6) \min$	2.51 (-7)	2.58 (2)	3.21 (-2)	3.30(7)
	Mn-54	313d = 4.51(5) min	1.54 (-6)	2.28 (2)	1.81 (-1)	2.68(7)
	Cs-137	$30.2y = 1.59(7) \min$	4.36 (-8)	1.92 (4)	5.65 (-3)	2,49(9)
				Total		2.54(9)

b. The radiation level at 3 meters after a four week decay period can be calculated by calculating the activity in Ci after 4 weeks and then using the formula:

$$R = \frac{6 CE}{d^2}$$

Where: R = radiation level in R/hr

C = Ci

E = MeV/d

d = distance in <u>feet</u> = 9.84

The beta radiation does not enter the picture because of the steel demineralizer vessel and the distance from the source

t = 4 weeks = 4.0? min.

		Activity 4 We	Gamma	$R/hr = \frac{6 CE}{0 C}$	
Radionuclide	<u>e-</u> \t	µСі	Ci	MeV/d	
Co-60	9.90(-1)	3.27(7)	3.27(1)	2.50	5.07
Mn-54	9.40(-1)	2.52(7)	2.52(1)	8.35(-1)	1.30
Cs-137	9.98(-1)	2.48(9)	2.48(3)	5.61(-1)	8.62(1)
		F	Vhr at 3 m	=	9.26(1)

6. University - Exam 19, Question 10

#### a. Shielding and distance

Since the source is a beta emitter, clear plastic (to minimize bremsstrahlung production) shadow shielding can be set up to shield an experimenter's body, and short tongs can be used to reduce the dose to hands and forearms. This quantity of <sup>32</sup>P would not require permanent shielding on all sides nor would remote handling tools be required. Reducing exposure time is generally nct practical, since such labeling experiments generally require fairly long, complex experimental procedures.

#### b. Shielding

A gamma source of this magnitude would require massive, permanent shielding on all sides. Neither time nor distance would be appropriate for reducing personnel dose. Direct exposure to the source even for very short times could be fatal, and the distance necessary to reduce the dose to a permissible level would preclude any experimental work around the source.

#### c. Time and distance

This source produces a high neutron dose rate and the particular operation is to be done only once. Due to the neutrons, adequate shielding would be bulky and unwieldy. Such shielding could make the transfer so difficult that personnel dose might actually be increased due to the greatly lengthened time of erposure. Further, since the transfer is to be done only once, adequate shielding would greatly add to the cost of the operation. Therefore, several practice transfers should be done using a simulated source and long handling tools to increase the distance from the source. The practice transfer will serve to uncover any unexpected, particularly difficult steps and, thus help to reduce the time of exposure to the actual source. Thus, time in conjunction with distance (long handling tools) is probably the best solution to this radiation exposure problem.

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7. General - Exam 20, Question 3

a. Dosimeter



Given exposure of 0.1 rem at A. Calculate exposure at D AC =  $\sqrt{20^2 + 35^2} = 40.31$  cm

 $\theta = \operatorname{Arc} \operatorname{tan} \left( \frac{20}{35} \right) = \operatorname{arc} \operatorname{tan} (0.57) = 29.74^{\circ}$ 

BC = 10 cm x sec  $\theta$  = 11.5 cm, .\*. AB = 40.31 - 11.5 = 28.8 cm Neglecting attenuation, Dose (B) =  $\frac{(40.31)^2}{(11.5)^2}$  x 0.1 rem

$$= 1.23 \text{ rem}$$

$$\mu = \frac{0.693}{\text{HVL}} = \frac{0.693}{5 \text{ cm}} = 0.14 \text{ cm}^{-1}$$
Attn =  $\frac{1}{10}$  =  $e^{-\mu x}$  =  $e^{-0.14 \times 28.8}$  = 0.018  
Dose (B) =  $\frac{1.23 \text{ rem}}{0.018}$  = 68.26 rem  
Dose (D) =  $\frac{(11.5)^2}{(10)^2} \times 68.26$  = 30.27 rem

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- b. 1. Inspect radiographic equipment to rule out malfunction (radiographic "cameras" usually have a positive indication of source retraction to the "safe" position). Set up a regularly scheduled preventive maintenance and monitoring program for the equipment.
  - Have the radiographer wear a "chirper" or "beeper" alarming pocket radiation monitor, and/or monitor the work area before re-entry.
  - Develop standard operating procedures for radiographic equipment usage. Couple this with scheduled periodic refresher courses and equipment checkouts for all users.
  - 4. Use a portable, radiation activated warning light and/or audible alarm in the radiation area during all radiography sessions.

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#### Section 7

#### Suggested Study References

The following bibliography is intended to provide the candidate with reference material related to the general topics covered in the exam. The Board does not mean to imply that study of these references, only, will ensure successful performance on the examination. This listing is by no means complete, and the candidate may need to consult additional reports, journals, and text books for information not provided in the references below.

At the same time, the Board does not want to infer that study of all of these references is necessary to successfully complete the examination. The list is provided as a guide to the type of material which should be studied.

Selected Health Physics Bibliography

 National Council on Radiation Protection and Measurements Reports -Particularly the following:

NCRP Report No. 8 (NBS Handbook 48) Control and Removal of Radioactive Contamination in Laboratories (1951).

NCRP Report No. 22 (NBS Handbook 69) Maximum Permissible Body Burdens and Maximum Permissible Concentrations of Radionuclides in Air and Water for Occupational Exposure (1959).

NCRP Report No. 23 (NBS Handbook 72) Measurement of Neutron Flux and Spectra for Physical and Biological Applications (1960).

NCRP Report No. 25 (NBS Handbook 75) Measurement of Absorbed Dose of Neutrons and of Mixtures of Neutrons and Gamma Rays (1961).

NCRP Report No. 28 (NBS Handbook 80) A Manual of Radioactivity Procedures (1961).

NCRP Report No. 32, Radiation Protection in Educational Institutions (1966).

NCRP Report No. 33, Medical X-ray and Gamma Ray Protection for Energies Up to 10 MeV - Equipment Design and Use (1968).

NCRP Report No. 35, Dental X-ray Protection (1970).

NCRP Report No. 38, Protection Against Neutron Radiation (1971).

NCRP Report No. 39, Basic Radiation Protection Criteria (1971).

NCRP Report No. 43, Review of the Current State of Radiation Protection Philosophy (1975).

NCRP Report No. 48, Radiation Protection for Medical and Allied Health Personnel (1976).

NCRP Report No. 49, Structural Shielding Deisgn and Evaluation for Medical Use of X-rays and Gamma Rays of Energies Up to 10 MeV (1976).

NCRP Report No. 50, Environmental Radiation Measurements (1976).

NCRP Report No. 51, Radiation Protection Design Guidelines for 0.1 - 100 MeV Particle Accelerator Facilities (1977).

 International Commission on Radiation Units and Measurements Reports -Particularly the following:

ICRU Report 14, Radiation Dosimetry; X-Rays and Gamma Rays with Maximum Fhoton Energies Between 0.6 and 50 MeV.

ICRU Report 17, Radiation Dosimetry; X-Rays Generated at Potentials of 5 to 150 kV.

ICRU Report 19, Radiation Quantities and Units.

ICRU Report 20, Radiation Protection Instrumentation and its Application.

ICRU Report 21, Radiation Dosimetry; Electrons with Initial Energies Between 1 and 50 MeV.

ICRU Report 22, Measurement of Low Level Radioactivity.

ICRU Report 25, Conceptual Basis for the Determination of Dose Equivalent.

 International Commission on Radiation Protection Publications - Farticularly the following:

ICRP Publication No. 7, Principles of Environmental Monitoring Related to the Handling of Radioactive Materials.

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ICRP Publication No. 8, The Evaluation of Risks from Radiation.

ICRP Publication No. 9, Recommendations of the ICRP.

ICRP Publication No. 10, Evaluation of Radiation Doses to Body Tissues from Internal Contamination due to Occupational Exposure.

ICRP Publication No. 10a., The Assessment of Internal Contamination Resulting from Recurrent or Prolonged Uptakes.

ICRP Publication No. 12, General Principles of Monitoring for Radiation Protection of Workers.

ICRP Publication No. 15, Protection Against Ionizing Radiation from External Sources.

ICRP Publication No. 16, Protection of the Patient in X-ray Diagnosis.

ICRU Publication No. 17, Protection of the Patient in Radionuclide Investigations.

- Attix, F., et. al., <u>Radiation Dosimetry</u>, Vols. I III, Academic Press, 1968.
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- Friedlander, G., et. al., <u>Nuclear and Radiochemistry</u>, Wiley and Sons, New York, NY 1964.
- Health and Safety Laboratory Procedures Manual, (HASL 300), US ERDA, New York, NY 10014.
- Health Physics Journals, Pergamon Press.
- 14. International Atomic Energy Agency, IAEA Safety Series 1 30, UNIPUB, Inc. New York, NY.

- Johns, H.E., <u>The Physics of Radiology</u>, Charles C. Thomas Publisher, 1971.
- Lapp, R.E. and Andrews, H., <u>Nuclear Radiation Physics</u>, Prentice-Hall, 1972.
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- Morgan, K.Z. and Turner, J.E., <u>Principles of Radiation Protections</u>, John Wiley and Sons, Inc. (1967).
- 19. Overman and Clark, Radioisotope Techniques, McGraw-Hill, 1960.
- Patterson, H. and Thomas R., <u>Accelerator Health Physics</u>, Academic Press, 1973.
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- 22. Radiological Health Handbook, U.S. Dept. of HEW, Public Health Service, Rockville, MD.
- Report of the Advisory Committee on the Biological Effects of Ionizing Radiation, <u>The Effects on Populations of Exposure to Low Levels of</u> <u>Ionizing Radiation</u> (BEIR Report), National Academy of Sciences, National Research Council, Washington, D.C. 1972.
- Saenger, E., <u>Medical Aspects of Radiation Accidents</u>, GPO (USAEC), 1963.
- Slade, D. (Editor), <u>Meteorology and Atomic Energy</u>, USAEC, TID-24190, 1968.
- 26. Taylor, L., Radiation Protection Standards, CRC, 1971.
- Title 10, Chapter 1, Code of Federal Regulations, USNRC, Washington, D.C. 20555.
- United Nations Scientific Committee on the Effects of Atomic Radiation, Ionizing Radiation: Level and Effects, New York, NY 1972.
- 29. US NRC Regulatory Guides, Particularily the following: 8.0 Occupational Health series, 1.21, 1.101, 1.109 and 4.2.
- Wash 1400, Reactor Safety Study (Rasmussen Report) Particularily; Main Report and Appendix VI (Calculation of Reactor Accident Consequences).

#### Section 8

#### Exam Strategy

The Board believes that it is an advantage to develop a strategy for taking the certification examination. We have noted in the past that candidates have, through a number of oversights and errors, penalized themselves heavily, in some cases heavily enough to make a difference between success and failure on the examination.

While we do not believe that our suggestions, given below, are the only possible ones on examination strategy, we do believe that they are sound and that at least they should stimulate the development of a suitable plan of your own.

#### PART I

Part I is a multiple choice examination, lasting three hours and requiring the answers to 150 questions. Some of the answers require calculation.

1. Budget your time so that you are answering about 1/3 of the questions in each hour.

2. Begin at the beginning and go through the whole examination, answering the questions you are sure of, in order. Pass over the difficult, uncertain questions, saving them until the end. Do not lose time by getting bogged down on a few difficult questions.

3. There is no penalty for a wrong answer, and therefore, it is to your advantage to answer every question.

4. If you are uncertain about an answer, it is probably true that your first choice is the correct answer. Do not change an answer unless you are certain that the first answer is wrong.

#### PART II

Part II consists of 16 questions, of which you answer any 7 in 4 hours. These questions may call for both numerical answers in which substantial calculation may be involved, and short essay-type answers.

1. When you receive this part of the examination, read through it in its entirety, then begin to work the questions which are easiest for you. Save the difficult questions until the end.

2. As in Part I, make a conscious effort to budget your time.

3. Before beginning to answer a question, read it again carefully so that you can be certain you are answering the question that is asked.

4. Think carefully about numerical constants and assumptions that you use. Try to be sure that they are accurate and reasonable.

5. Do your best to demonstrate a professional approach to the problems.

6. Organize your answer in a logical outline form to use as a check list to assure efficient and complete subject treatment. A concise, well-organized answer is much nore impressive than a rambling ten page discertation.

7. Re-read the question after completing it to be sure you have answered all the portions of the question and have provided all the information requested.

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#### Section 9

#### Grading Criteria

Effective September 23, 1977, the Board has formalized the following grading criteria for the certification examination.

1.

Part I - Taken Alone

#### Passing Criteria

To pass Part I, the candidate must achieve a score of at least 67 percent on the total exam and on the Fundamentals Section.

2.

Part I and II - Taken Together

#### Passing Criteria

To pass the exam, the candidate must achieve a score of at least 67 percent on both Part I and Part II.

#### Failure Upgrading Criteria

Any grade less than 67 percent on either part will be considered to be a failure of that part. To provide candidates with the opportunity to raise a failing grade to a passing grade the Board uses the following guides:

a. Give candidates who have scored at least 57 percent on both Part I and Part II and whose average grade (Part I and II given equal weight) is at least 60, the option to take an oral exam or retake the part(s) failed. (If a candidate repeatedly fails one of the parts, the option may be removed and the candidate required to take an oral exam.)

b. The Board considers any grade less than 57 percent to be below the standards for oral upgrading.

Availability of Performance Information

Candidates may request their performance information to assist them in preparing for re-examination.

#### DEFINITION OF RADIATION PROTECTION TECHNOLOGIST

A Radiation Protection Technologist is an individual devoted to the protection of man and his environment from unwarranted radiation exposure. This person is engaged in the study of the problems and practices of providing radiation protection. He is concerned with an understanding of the mechanisms of radiation damage, with the development and implementation of methods and procedures necessary to evaluate radiation hazards and with providing protection to man and his environment from unwarranted radiation exposure.

MARCH 1978

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EXAMINATION

PREPARATION

GUIDE

NATIONAL REGISTRY RADIATION PROTECTION TECHNOLOGISTS

## DEFINITION OF A RADIATION PROTECTION TECHNOLOGIST

Health Physics is a profession devoted to the protection of man and his environment from unwarranted radiation exposure. A Radiation Protection Technologist is a person engaged in the operational aspects of providing radiation protection. This individual is concerned with the basic understanding of the mechanisms of radiation damage, with the development and implementation of methods and procedures necessary to evaluate hazards and with providing protection to man and his environment from unwarranted radiation exposure.

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#### AN IMPORTANT MESSAGE TO CANDIDATES

This guide will help you prepare for the National Registry of Radiation Protection Technologists (NRRPT) examination. The guide, however, will not be the key to a good grade on the examination. Successful candidates usually start their preparation months before the test. They cover the fundamental and applied aspects of radiation protection through use of textbooks and joining study groups. The Board warns against approaching the exam in a casual fashion.

We believe that all applicants declared eligible to take the examination, after careful review of credentials, have a good probability of passing. You can avoid the disappointment of poor performance by recognizing at the start that the exam will be a rigorous test of your knowledge. Your grade will represent, for the most part, the thoroughness of your preparation.

Now that you know the key to good performance on the examination, we wish you success in achieving registration.

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#### CONTENTS OF NRRPT EXAMINATION

The examination consists of 150 multiple choice questions divided into three general categories: fundamentals, measurements, and operational health physics. Four hours are allowed for the exam. Not all the questions in this part are replaced each year. Consequently, this part of the test is held in strict confidence, and copies of past exams are not distributed. Questions typical of those found in the examination are in this guide.

We believe that it will be to your advantage to develop a plan for taking the forthcoming NERPT examination. We have noted in the past that candidates have, through oversights and errors, penalized themselves heavily, in some cases heavily enough to make the difference between success and failure on the examination. While we do not believe that our suggestions given below are the only possible ones for examination strategy, we do believe they are sound, and that they should at least stimulate the development of a suitable plan of your own.

#### EXAMINATION

The examination is a multiple choice lasting four hours and requiring the answers to 150 questions. Some answers require calculation.

- Budget your time so you are answering about 1/3 of the questions each hour.
- Start at the beginning and go through the entire examination in numerical order answering the questions you are sure of. Pass over the difficult questions you are uncertain of, saving them until the end. Don't lose time by getting bogged down on a few difficult questions.
- There is no penalty for a wrong answer. Therefore, it is to your advantage to answer each question.
- If you are uncertain about an answer, it is probably true that your first choice is the correct answer. Do not change an answer unless you are certain your first answer is wrong.

Further information about the NRRPT program may be obtained from the Chairman of the National Registery of Radiation Protection Technologists.

> Mr. D. Marshall TRA 667 EG&E Idaho, Inc. P. O. Box 1625 Idaho Falls, Idaho 83401

#### TYPICAL EXAM QUESTIONS

- 1. An instrument that gives a direct exposure rate reading resulting from X or Gamma radiation is a:
  - 1. quartz fiber dosimeter.
  - ion chamber survey meter.
     film badge.

  - 4. chemical dosimeter.
  - 5. proportional counter.
- Typical counting efficiencies of an ordinary (2 π) internal proportional counter for weightless alpha and beta sources might be, respectively:

	Alpha		Beta
1.	10%	and	50%
2.	50%	and	50%
3.	50%	and	65%
4.	65%	and	75%
5.	75%	and	50%

- The maximum permissible dose to the whole body of any individual 3. in a restricted area as stated in the Nuclear Regulatory Commission rules and regulations is expressed as rem per:
  - 1. hour
  - 2. week
  - 3. month
  - 4. calendar quarter
  - 5. calendar year
- 4. In general, the body cells most susceptible to damage by radiation are those found in:

- 1. highly specialized tissues
- 2. rigid or semirigid tissues
- 3. muscular tissues
- 4. rapidly dividing tissues
- 5. endocrine tissues

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- 5. Which of the following combinations of instruments would make the best selection for determining the dose at the open beam port of a swimming pool reactor?
  - 1. <u>Air-equivalent-wall ionization chamber, boron tri-fluoride</u> counter, tissue-equivalent fast neutron counter.
  - Geiger counter, boron trifluoride counter, phosphate glass dosimeter.
  - shielded (50 mg/cm<sup>2</sup>) ion chamber, moderated boron trifluoride counter, zinc sulfide scintillation detector.
  - quartz fiber clectroscope, methane recoil proton counter, Geiger gounter.
  - counter, Geiger counter.
    5. shielded (50 Mg/Cm<sup>2</sup>) ion chamber, methane recoil proton counter, zinc sulfide scintillation counter.
- 6. Which one of the following instruments is least useful as a basis for detecting and measuring a neutron fluence?
  - 1. proportional counter
  - 2. ionization chamber
  - 3. fission chamber
  - 4. scintillation counter
  - 5. BF<sub>3</sub> Geiger counter
- Proportional survey meters are often preferred to Geiger-Mueller survey instruments in surveying for alpha contamination because they:
  - 1. are more sensitive.
  - 2. have no 'window' through which alphas must pass.
  - 3. need less voltage.
  - 4. are able to discriminate against beta and gamma radiation.
  - 5. are less energy dependent.
- Two atomic species are referred to as isotopes if they have the same:
  - 1. atomic number but a different mass number.
  - 2. atomic weight but a different atomic number.
  - mass number and atomic number but different radioactive properties.
  - 4. number of neutrons but a different mass number.
  - 5. mass number but a different atomic number.
- In picocurie of any radioactive substance, the disintegration rate is:

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1. 2.22 DPM 2. 2.22 x 10<sup>6</sup> DPM 3. 37,000,000 DPM 4. 3.7 x 1010 DPS 5. 3.7 x 101 DPS To aid in reviewing for the exam by either home study or organized training courses a General Outline of the three catagories, (Fundamentals, Measurements and Operational) is supplied. Also included a list of suggested reference books. This list does not by any means constitute the complete reference material outline.

# GENERAL OUTLINE

1. FUNDAMENTALS

Radiation Sources

Natural Background Man-Made

## Principles and Fundamentals of Radiation Sources

Atomic Structure Radiation and Radioactivity Radiation Generating Equipment

# Interaction with Matter (Ionization)

Particulate Electromagnetic (Scattering, Filtration)

Biological Effects

Units of Dose Acute Effects Chronic Effects

General (Broad Coverage Topics)

Fundamentals of Radiological Health Specialized Topics Beneficial Applications of Radiation

Basic Related Science

Chemistry Physics Biology (Bacteriology, Nursing Care, Physiology, Anatomy) Electronics Engineering, Mechanical and Civil

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# II. MEASUREMENTS

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Detection and Measurement Instruments and Devices

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Basic Theory - Detectors
Basic Theory - Readouts
Calibration
Maintenance
Use
Applications
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Exposure Evaluation

Techniques for External Sources Techniques for Internal Sources

Environmental Surveillance and Sample Analysis

Analysis of Gross Activity Analysis for Specific Radionuclides

# III. OPERATIONAL

Exposure Evaluation

Guidelines Effects and Behavior of Radionuclides

Protection and Shielding

Basic Principles Radiation Generating Equipment Radioactive Materials Nuclear Reactors

Reactor Safety and Hazard Evaluation

Reactor Theory and Control (Critical Mass) Fission Product Inventories Hazard Evaluation Routine Operations Transportation Fuel Production and Preparation

Radioactive Waste Management

Gases and Particulates Liquids Solids

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# Surveys and Inspections

Radiation Generating Equipment Nuclear Reactor Operations Radionuclides Users

# Environmental Surveillance and Sample Analysis

Sampling Procedures and Equipment Existing Levels and Data Interpretation Environmental Sources

# Nuclear Emergencies

Sources and Types of Accidents Emergency Exposure Guides Public Relation and Administrative Aspects Medical Aspects Hazard Liability Aerospace Nuclear Safety

Radiation Protection Guides, Regulations and Recommendations, Legislative Power

## APPENDIX A

## Home Study Aids

1. Radiation Monitoring, EDM 123.

1.0

ERDA Technical	Information Center		
Oak Ridge, TN	37830	Single copy - \$3.70	
(make checks pa	ayable to ERDA)	25 or more - \$2.50	

 <u>Radiation Safety Technician Training Course</u>, ANL 7291, Rev. I, H. J. Moe, S. R. Lasuk, et al.

National Technical Information Service U. S. Dept. of Commerce 5285 Fort Royal Road Springfield, VA 22161 (est. \$6.00/copy)

3. <u>Atomic Radiation</u>, Theory, biological hazards, safety measures, treatment of injury. 110 pages

R.C.A. Service Company Government Services Bldg. 204-1, Cherry Hill Camden, NJ 08101 (post paid \$3.00)

Atomic Radiation, Part II, Monitoring, radiation protection, radioactive shipment, waste disposal. 110 pages

R.C.A. Service Company Government Services Bldg. 204-1, Cherry Hill Camden, NJ 08101 (post paid \$5.00)

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# APPENDIX B

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# Reference List

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# LAWRENCE LIVERMORE LABORATORY

Hazards Control Department

April 6, 1979

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David S. Myers

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Tabb, Frank G. Taschner, John C. Taylor, L. S. Terpillok, Michael S. Terrill, James G. Jr. Thiessen, J. W. Thomas, George E. Thomas, John W. Thomas, John W. Thomas, Ralph M. Thompson, James J. Thorburn, Robert C. Thornton, William T. Tiernan, Joseph A. Tochlin, Eugene

# NATIONAL REGISTRY OF RADIATION PROTECTION TECHNOLOGISTS

#### 1978

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NOTE: Term of office expires on January 1st of year indicated.

# Aubrey, Sichard F.

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Bakevich, George J. Bardecker, Jerald E Barton, John M. Bourquin, Martin W Branter, M. Keith Boyd, Robert M. Brown, Jeffrey M. Brown, Robert J Burbage, Woodrow R Burke, Lawrence L. Burstad, Duane R Butler, Alfred C

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Harrison, John D. Harvey, Paul W. Hatcher, Larry W. Helvie, James D., Jr Herron, David S

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# REGISTERED RADIATION PROTECTION TECHNOLOGISTS

# NATIONAL REGISTRY OF RADIATION PROTECTION TECHNOLOGISTS

### February 16, 1978

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Bakevich, George J. Bardecker, Jerald E. Barton, John M. Bourquin, Martin W. Branter, M. Keith Bovd, Robert M. Brown, Jeffrey M. Brown, Robert J. Burbage, Woodrow R. Burbage, Woodrow R. Burbage, Woodrow R. Burbage, Joane R. Burter, Alfred C.

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Harrison, John D. Harvey, Paul W. Harcher, Larry W. Helvie, James D., Jr. Herron, David S. Hodges, William R. Hooker, Claude D. Horman, R. Leon Horvath, Dennis A. Humm, Arthur F., Jr Hunt, Jerry B.

## Irvine, Dan L

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Jakubovski, Frank M. Johnson, Alvin C. Johnson, Joseph P. Johnson, Richard D. Jones, Charles C.

#### K

Kaulback, Donald Eric Klover, William J. Kitts, Ronnie J. Kobayshi, Richard S. Koeppen, Don L. Kroli, O. B.

#### L

Lamson, Myles L., III Lee, David W. Linville, John T. Lipp, Robert D. Loe, Ronald H. Loesch, Robert M. Lum, Jesse D. Lunsford, Marvin C.

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Mackliet, Dan L. Madden, Timothy W. Magyar, George Marshall, Don W. May, Bryce J. McCarthy, Marvin J. McClosky Debies, Mary Anne McNight, Charles D. Merkling, Terry D. Miller, Larry O. Morgin, R. L.

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Oliver, Harry A. Orrock, Donald L.

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#### Peterson, Henry K. Pringle, Charles D

Reynolds, Harold Wayne Sichards, Voncort R. Roesler, James J. Roter, Orew F. Rosenberry, Charles E. Rosenberry, Charles E.

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Tarpinian, James Ephrem Teague, Stephen V. Terpilak, M. S. Thomas, James L. Turner, Douglas V.

U Urness, David A.

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Van Parten, Allyn F. Vannoy, Tracy W.

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Waggoner, Larry O. Wallace, James E., Jr. Weetman, Richard A. Wartersau, Leonard W., Jr. Williams, Stephen L. Wilson, Ross L. Wilson, William A. Wright, Maynard R.

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# American Board Of Health Physics

March 1978

Dear Colleague:

Enclosed for your information is material concerning the Continuing Certification Program. The following information is enclosed:

- (1) General Policies and Procedures
- (2) Guidelines for the ABHP Continuing Certification Program
- (3) Application for Renewal of Certification
- (4) Application for ABHP-CEP Course Approval

Inquiries concerning the Continuing Education Program should be made to the following individuals:

(1) Inquiries concerning education credits for courses and related activities should be directed to:

> Lester Slaback AFRRI NNMC Bethesda, Maryland 20014

(2) Inquiries concerning continuing certification of individuals and applications should be directed to:

> Carlyle J. Roberts Division of Environmental Impact Studies Building 10 Argonne National Laboratory Argonne, Illinois 60439

In addition, information dealing with courses and supporting documentation submitted to L. Slaback should be summarized in concise language not to exceed 5 pages. If more than 5 pages are required, then the individual and/or organization should provide 8 additional copies.

A formal newsletter summarizing the previous year's activities will be transmitted in April.

Thank you for your continued support of ABHP activities.

Michael S. Terpilak Secretary-Treasurer

# American Board Of Health Physics

Meno to: Certified Health Physicists

From: C. J. Roberts, Vice Chairman, ABHP

Subject: Continuing Certification Program

This is a status report on the Continuing Certification (or recertification) Program of the American Board of Health Physics.

The program had a lengthy gestation period which included an open discussion of continuing certification at the Annual Health Physics Society meeting in 1975 and invitations\* to all CHP's for comments concerning the Board's evolving proposals. As a result of this extended dialogue the Board diplomates did reach a consensus on guidelines for a continuing certification program. These guidelines, formally adopted by the American Board of Health Physics at its meeting on June 27, 1976, are enclosed (see Attachment I).

At its San Francisco meeting last June, the Board also appointed the Continuing Education Panel called for by the guidelines. The panel is chaired by Roger J. Cloutier (see Attachment II for complete membership list). The responsibilities of the Panel include establishing standards for approval of courses to meet ABHP continuing education requirements.

Although the Panel is not ready to publish a general set of standards for use by potential sponsors in organizing acceptable courses, it is in the process of approving certain refresher courses to be given at the 22nd Annual HPS meeting in Atlanta, July 3-8, 1977. As soon as these arrangements are completed the details will be announced in the HPS Newsletter, and in the program for the Atlanta meeting. As many as 6 hours of lecture may be approved for credit. Since the refresher courses have been scheduled in pairs, anyone at the Atlanta meeting will be able to attend up to three hours of approved lectures. A total of 16 contact hours of lectures and demonstrations in advanced health physics topics is required during each renewal period, including the initial one which ends on December 31, 1981.

The Board expects to publish general standards for approval of continuing education courses soon after the Annual HPS meeting. Applications for renewal of certification also will be available at that time, although it will not be possible for CHP's to apply until they have accumulated the required credits for attendance of approved courses.

Please let me know if you have questions or comments concerning the continuing certification program.

C. J. Roberts Phone: CEA-15 312-739-7711 Argonne National Laboratory Ext. 2211

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\*From N. C. Reinig, Chairman of the ABHP at that time, dated October 10, 1975, and March 15, 1976.

AMERICAN BOARD OF HEALTH PHYSICS CONTINUING CERTIFICATION PROGRAM

# I. <u>Renewal</u> Period

In the five-year period beginning on January 1, 1977, and during each fouryear period thereafter, each Certified Health Physicist shall renew his certification. Individuals certified after January 1, 1977, shall renew their certification within each four-year period starting on January 1 in the year after certification is awarded.

Explanatory Note: Present Certified Health Physicists would be required to renew their certification before January 1, 1982. The next renewal deadline would be January 1, 1986. For example, an individual may choose to be recertified in 1977 and he may wait until 1985 before the next renewal.

# II. Extension of Renewal Period

The ABHP may extend the renewal interval, upon request when an individual cannot meet the requirements because of sickness, fore residence or other unusual circumstances.

Explanatory Note: This flexibility is provided to allow the Board to grant extensions when necessary. These cases should be infrequent.

# III. Requirements for Continuing Certification

To renew his certification a diplomate shall remain active in the profession of health physics and keep abreast of new developments in the profession. Demonstration of these requirements shall be provided through the following steps that shall be accomplished during the renewal period:

- a. Submission of an Application for Renewal of Certification.
- b. Attendance at ABHP-approved continuing education courses.
- c. Submission of further documentation to verify professional responsibilities and activities may be required by the Board.

# Explanatory Notes:

- a. The Application for Renewal of Certification will provide the Board with information about the diplomate's professional activities during the past four years. The form will be similar to the original application for certification. The application will also include a reaffirmation that the individual is fulfilling the Professional Responsibilities of a Certified Health Physicist.
- b. The continuing education requirement will be met by attending professionallevel courses approved by the ABMP. During the renewal period, each diplomate shall attend a course or courses providing a total of at least 16 contact hours of lectures and demonstrations on advanced health physics topics. No course examinations will be required. Courses may be

sponsored by any organization. Each course must be approved by the ABHP prior to attendance. Approval may be requested by sponsors of courses or by individual Certified Health Physicists. The Board will establish a Panel on Continuing Education to arrange and accredit courses. Lecturers at an ABHP-approved course will receive appropriate credit depending on the extent of their participation. Whenever practical, the ABHP will announce the approved courses in advance through selected publications; however, the sponsoring organization will have the primary responsibility for course announcements.

c. If the Board cannot determine through a review of the Application for Renewal of Certification that the applicant is actively engaged in the profession of health physics at least 25% of his/her working time and fulfilling the Professional Responsibilities of a Certified Health Physicist, the Board may require the applicant to submit reports or other documentation and letters of reference to assist the Board in its review. These cases should be infrequent.

### IV. Classification of Certified Health Physicists

There shall be three classes of Certified Health Physicists:

<u>Certified Health Physicist</u>: This class shall consist of all diplomates who, in the judgment of the Board, meet the requirements for recertification. These individuals shall be included in published listings of Certified Health Physicists.

<u>Certified Health Physicist - Emeritus</u>: This class shall include Certified Health Physicists who have retired from full-time professional activity. These individuals shall be included in published listings of Certified Health Physicists with the Emeritus designation.

<u>Certified Health Physicist - Inactive</u>: This class shall consist of all individuals who, in the judgment of the Board, do not meet the requirements for continuing certification. These individuals shall not be included in the published listings of Certified Health Physicists. At any future time, an in ividual in this class may regain active status upon completion of the requirements for renewal of certification.

### Explanatory Notes:

- a. The Emeritus status will be awarded, upon request, to Certified Health Physicists who retire from full-time active participation in professional activities because of age or health requirements.
- b. The Inactive status will, in most cases, result from individuals changing disciplines. For several reasons, the Board chooses to place these individuals in an Inactive status rather than revoke certification. The most compelling reason is that legal action may be initiated to prevent revocation of certification. Although the Board is confident that its judgment would be upheld, the Board prefere to use its limited resources to further the certification program :ather than expend them in legal procedures.

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# V. Renewal Fee

The renewal fee shall be \$20.

Explanatory Note: The fee for renewal of certification will be paid at the time the Application for Renewal of Certification is submitted. In addition, organizations that sponsor continuing education courses may charge a registration fee for the courses. The Board will encourage these organizations to establish the registration fees at a reasonable level. Preferably, these fees should only reimburse the sponsoring organization for the expenses incurred in the administration of the course.

# VI. Appeals

Individuals shall have the right to meet with the Board and appeal any decision made by the Board that affects their certification status.

# CONTINUING EDUCATION PANEL

Chairman: Roger J. Cloutier Vice-Chairman: Robert L. Junkins

# Term Expires 1977

Donald E. Barber 815 22nd Avenue, NW New Brighton, MN 55112 (612)373-8080

### Term Expires 1978

Roger J. Cloutier Oak Ridge Associated Universities P. O. Box 117 Oak Ridge, TN 37830 (615)483-8411 X 263 FTS: 850-4642

# Term Expires 1979

Frazier Bronson 2647 North Prindle Arlingtor Heights, IL 60006 (312)266-8566 (work) (312)259-7076 (home)

### Term Expires 1980

Robert L. Jun ins Radiation Management Corporation Suite 400, Science Center Bldg. #2 3508 Market Street Philadelphia, PA 19104 (215)243-2964 T. Jordan Powell Mail Code L-518 Lawrence Live more Laboratory Livermore, CA 94550 (415)447-1100 X 3822 FTS: 457-3822

Jean St. Germain Department of Medical Physics Memorial Sloan-Kettering Cancer Center 1275 York Avenue New York, NY 10021 (212)794-7390

Francis J. Haughey Radiation Science Busch Campus Rutgers University New Brunswick. NJ 08903 (201)932-2551 or 2582

Lester A. Slaback, Jr. Ar ed Forces Radiobiology Research Institute Defense Nuclear Agency Bethesda, MD 20014 (202)295-1285

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# American Board Of Health Physics

CONTINUING EDUCATION PANEL

# General policies and procedures.

A. In order to qualify for credit toward meeting the continuing education requirements of the American Board of Health Physics (ABHP) all courses and other activities must be approved by the Continuing Education Panel (CEP). Application for approval may be made directly to the Chairman of the CEP by the course sponsor or a participating (artified Health Physicist (CHP). Applicants are urged to submit their requests far enough in advance that a decision can be made by the Panel and announced before the course begins; however, the Panel will accept without prejudice (applying their usual approval criteria) all applications must be in the form specified by the Panel and be complete in all respects.

B. In the context of this document, a "continuing education course" is a program that is formally organized, is offered within a specific time period, covers preselected topics and is given by  $s_{\mu}$  offied individuals. Only that portion of a program which relates rather directly to health physics and contributes to the technical competence of the CHP will be approved. Related subjects are those that are used directly in health physics but are not usually designate, as health physics courses. Examples of these might be statistics, meteorolog; as applied to environmental dose assessment, reactor coolant chemistry and relation genetics. The Panel will evaluate each course on the basis of content, instructors' qualifications, degree of student involvement and schedule. After weighting these factors according to an established formula, it will assign each course a number of continuing educations credits which may be less than the number of contact hours.

C. The following activities have been reviewed by the CEP and approved for continuing education credit without specific application by individual CHP's. These approvals are exclusive of any additional education credits that might be earned by attending specific events at these meetings.

(1) Attendance and participation at the annual Health Physics Society meeting shall receive one continuing education credit per day with a limit of three (3) credits per meeting.<sup>2</sup>

(2) Attendance and participation at the HPS Midyear Topical Symposium shall receive one continuing education credit per day with a limit of three (3) credits per meeting.<sup>2</sup>

D. Course sponsors or organizers are strongly encouraged to provide certificates of attendance or other forms of recognition to the attendees.

"As an exception, applications for approval of continuing education activities concluded any time during 1977 will be accepted through April 1, 1978.

<sup>2</sup>The ABHP will accept a maximum of 8 CEC's acquired through attendance at these meetings toward the required total of 16 credits.

# GUIDELINES FOR THE AMERICAN BOARD OF HEALTH PHYSICS CONTINUING CERTIFICATION PROGRAM

## I. Renewal Period

In the five-year period beginning on January 1, 1977, and during each four-year period thereafter, each Certified Health Physicist shall renew his certification. Individuals certified after January 1, 1977, shall renew their certification within each four-year period starting on January 1 in the year after certification is awarded.

Explanatory Note: Present Certified Health Physicists would be required to renew their certification before January 1, 1982. The next renewal deadline would be January 1, 1986. For example, an individual may choose to have his certification renewed in 1978 and he may wait until 1985 before the next renewal.

# II. Extension of Renewal Period

The ABHP may extend the renewal interval, upon request, when an individual cannot meet the requirements because of sickness, foreign residence or other unusual circumstances.

Explanatory Note: This flexibility is provided to allow the Board to grant extensions when necessary. These cases should be infrequent.

# III. Requirements for Continuing Certification

To renew his certification a diplomate shall remain active in the profession of health physics and keep abreast of new developments in the profession. Demonstration of these requirements shall be provided through the following steps that shall be accomplished during the renewal period:

- a. Submission of an Application for Renewal of Certification.
- Attendance at ABHP-approved continuing education courses, or other approved activities.
- c. Submission of additional information to describe and verify his continuing professional responsibilities and activities if requested by the Board.

#### Explanatory Notes:

a. The Application for Renewal of Certification will provide the Board with information about the diplomate's of ssional activities since his previous application was submit. The form will be similar to the original application for ce The application will also include a reaffirmation that the control of ce The application will also include Responsibilities of a Cert Control of Physicist.

\* Throughout this document the conventional masculine pronoum is used when collective members of both sexes are referred to. thus, his = his/her.

# IV. Classification of Certified Health Physicists

There shall be three classes of Certified Health Physicists:

Certified Health Physicist: This class shall consist of all diplomates who, in the judgment of the Board, meet the requirements for recertification. These individuals shall be included in published listings of Certified Health Physicists.

Certified Health Physicist - Emeritus: This class shall include Certified Health Physicists who have retired from active professional practice. These individuals shall be included in published listings of Certified Health Physicists with the Emeritus designation.

<u>Certified Health Physicist - Inactive</u>: This class shall consist of all individuals who, in the judgment of the Board, do not meet the requirements for continuing certification. These individuals shall not be included in the published listings of Certified Health Physicists. At any future time, an individual in this class may regain active status upon completion of the requirements for renewal of certification. The necessary 16 CEC's must have been earned within the span of four consecutive calendar years including the year in which application is made.

## Explanatory Notes:

- a. The Emeritus status will be awarded, upon request, to Certified Health Physicists who retire from active participation in professiona. activities because of age or health requirements.
- b. The Inactive status will, in most cases, result from individuals changing disciplines. For several reasons, the Board chooses to place these individuals in an Inactive status rather than revoke certification. The most compelling reason is that legal action may be initiated to prevent revocation of certification. Although the Board is confident that its judgment would be upheld, the Board prefers to use its limited resources to further the certification program rather than expend them in legal procedures.

# V. Renewal Fee

The renewal fee shall be \$20.

Explanatory Note: The fee for renewal of certification will be paid at the time the Application for Renewal of Certification is submitted. In addition, organizations that sponsor continuing education courses may charge a registration fee for the courses. The Board will encourage these organizations to establish the registration fees at a reasonably level. Preferably, these fees should only reimburse the sponsoring organization for the expenses incurred in the administration of the course.

VI. Appeals

Individuals shall have the right to meet with the Board and appeal any decision made by the Board that affects their certification status.

Application For Renewal of Certification

	INSTRUCTIONS Application for: 1. Type or print in block capitals. Initial Renewal 2. Submit only one copy. Later (specify 2nd, 3rd) 3. If space is inadequate for any answer, Emeritus Status - use extra sheet of paper and number items to correspond with items as listed.
	Date
1.	Name 2. Birth
3.	(last) (first) (middle) Home Address
4.	Business Address
5.	Send mail to: home address  Home Telephone Number
	business address 🗇 (incl. area code)
	Bus. Telephone Number
6.	Year of original certification by AEHP
ABHP/	CEP-approved continuing education courses attended during current renewal period.
	SponsorCourse TitleWhere OfferedDates From ToCEP Approval Certificate No.Cont. Ed.
a.	
Ъ.	
d.	
e.	
Other	r ABHP-approved continuing education activities during current renewal period.
	Description of Activity Where Offered From To Credits
a.	
b.	
c.	
d.	JUZ 1/1
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9. Academic Degrees Attained:

	Institution	Viutor	Minor	Years of Full Attend.	Degree	Year
		1000				
а.						
b.						
с.						

10. Additional education and training related to health physics (except as listed in 7 and 9) since you were certified or since last renewal.

	Institution	Title of Course	Length of Course	Dates From To
a.				
D				
c.				

11. Present position. Describe in your own words. Do not use official job descriptions. We are particularly interested in your health physics activities. Describe any previous positions with present employer in item 12.

Date Assigned to Position:	Name of Employer:	Place of Employment:	Name and Title of Immediate Supervisor:
Exact Title of Present Position:			
Description of work, Include major	responsibility and specific fields		

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Percent of time in health physics work

12. Previous positions held since you were certified or since last renewal. Start with most recent position and work back. Emphasize those portions of work that are health physics or closely related. Employer may or may not be same as in item 11.

	Name of Employer:	Place of Employment:	
rom: To:			
xact title of position:			
escription of work. Include m	najor responsibility and specific fields		
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Percent of time	in health physics work	이제 모양은 것은 것을 많이 많이 많이 많이 많이 많이 했다.	
Date of Employment:	Name of Employer:	Place of Employment:	
Date of Employment: From: To:	Name of Employer:	Place of Employment:	
Date of Employment: From: To:	Name of Employer:	Place of Employment:	
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Date of Employment: From: To: Exact title of position: Description of work . Include r	Name of Employer: major responsibility and specific tields	Place of Employment:	
Date of Employment: From: To: Exact title of position: Description of work . Include r	Name of Employer:	Place of Employment:	
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Date of Employment: From: To: Exact title of position: Description of work. Include r	Name of Employer: major responsibility and specific tields	Place of Employment:	
Date of Employment: From: To: Exact title of position: Description of work. Include r	Name of Employer: major responsibility and specific tields	Place of Employment:	
Date of Employment: From: To: Exact title of position: Description of work. Include r Percent of time	Name of Employer: major responsibility and specific fields	Place of Employment:	

13. Describe any other professional health physics activities, such as consulting, in which you have engaged in the past five years, or since your last renewal.

14. Current Professional and Technical Society Membership:

Name of Organization	Year Joined	Type of Membership	Office Held

15. Special Achievements:

a. Citations or other awards:

b. Committee Activities (past five years or since last renewal):

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16. Communication (within past five years or since last renewal);

a. Books and journal articles published

b. Technical papers read at meetings

c. Technical reports, memoranda or similar documents (include a small but representative sample if possible).

d. Other speeches and lectures related to health physics

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17. Categories of Competence:

Select the categories in the list below in which you feel you are competent at this time to function as a Certified Health Physicist. Rank these in the order of your proficiency. (1 for your first choice, 2 for your second, etc.).

 Industrial Radiographic Installations Medical Radiographic and Fluoroscopic Radiotherapy Installations Radionuclide Laboratories Enviornmental Monitoring	Installations	Nuclear Power Reactors Nuclear Fuel Cycle Facilities Accelerators Radiological Engineering Regulatory Programs
 Other (specify)		 Other (specify)

18. Professional References: name and address of at least two persons other than your supervisor who are qualified to evaluate your health physics competence. If possible, at least one reference should be a Certified Health Physicist; do not use a Board or Panel member as a reference. References will be consulted only in exceptional cases where the Board needs additional information.

I certify that the statements above (including any attachments I have submitted hereto) are, to the best of my knowledge, accurate, and I understand that any falsification of information in this application will be cause for rejection of the application or withdrawal of a certification already made.

Signature (in ink)

19. Statement Concerning Professional Responsibilities of Certified Health Physicists

In order to maintain his technical competence, the Certified Health Physicist has a commitment to remain active in the field of health physics and acquainted with the scientific, technical and regulatory developments in his field.

In order to uphold the professional integrity of health physics implied in this certification, his relations with others, including clients, colleagues, governmental agencies, and the general public shall always be based upon and reflect the highest standards of professional ethics and integrity. The Certified Health Physicist shall represent himself as an authority only in those areas in which he has extensive experience and in which he is considered expert by his peers.

By my signature, I verify that I am fulfilling the Professional Responsibilities of a Certified Health Physicist.

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Date:

Signature (in ink):

Note: This application is not complete unless you have signed your name twice above and have included a check, for the renewal of certification, made out to the American Board of Health Physics, in the amount of \$20.00. Send to C. J. Roberts, EIS-Bldg 10, Argonne Natl. Lab., Argonne, IL 60439 APPLICATION FOR ABHP-CEP COURSE APPROVAL

For Official Use Appl. No. Date Rec'd

Phone ()

- 1. Institution:
- 2. Address:
- 3. Person Responsible:

4. Course Title:

5. Course Date:

6. Instructor(s) (Attach resume(s)):

7. Course duration (lecture and lab time), schedule and outside prep requirements:

8. Course description or outline:

9. Texts/Suppl. Info:

10. Prerequistites:

11. Relationship to Health Physics

12. Certificate of course completion to be awarded: Yes No (Circle)

13. Certification: This is an accurate description of the above named course or program.

Signature Name Title

14. Requestor:

Name

Phone ()

Address

Purpose: \_\_\_\_ For approval for only my attendance For inclusion with publicity for this training Other specify:

15. Date by which approval is required

Notes on Application for Course Approval for Credit Towards ABHP Continuing Certification

1. Organization or institution sponsoring and/or organizing this training.

2. Address of the above institution.

3. Individual knowledgeable in the details of the makeup of this education (i.e., the information asked for on this form) and who can be contacted for further information.

4. Full title, including subtitles.

5. Starting and ending dates. Repetitions over a two (2) year time span may be specified.

6. Individual(s) presenting the training. The resume shoul emphasize his qualifications to present this course as well as any relevant health physics background. The instructor's involvement in the design and preparation of the course should also be detailed.

7. Total time (hrs) and time presentation scheme (e.g., 8 hrs. per day, 3 hours per day). Identify separately the number of hours of laboratory or field exercises and outside preparation expected to be performed by the student.

8. As detailed as possible with emphasis on relevance to health physics. As a minimum include major subject areas and the relative portion of the course or training devoted to each area.

9. Also identify areas of required supplemental reading.

10. Both with regard to entry to the course and with regard to the student background assumed in the design of the course. Specifically, is the course designed for experienced health physicists?

11. Show how this course relates to health physics if it is not obvious based on the course description or how it is of particular value to this applicant where specific approval is being requested.

12. Please indicate if a personalized certificate or some other notice of course completion or level of performance shall be given to the student or attendees. This is strongly encouraged as means of assisting the ABHP in its continuing certification review process.

13. By representative of organization identified in I if request is other than from an individual C.H.P.

14. This is the individual to whom the approval will be sent unless otherwise indicated. Please indicate the reason for submitting this application.

15. Please allow at least six (6) weeks for ABHP-CEP review. Send application o: Lester A. Slaback, Jr. Armed Forces Radiobiology Research Institute NNMC - Building 42 Bethesda, MD 20014

# American Board Of Health Physics

CONTINUING EDUCATION PANEL Armed Forces Radiobiology Research Institute National Naval Medical Center - Bldg. 42 Bethesda, Maryland 20014

Education Approved for Credit Towards ABHP Certification Renewal

1. The attached list includes all 1978 and 1979 courses approved through 1 February 1979.

2. The list of 1977 courses is available upon request.

3. The list does not include the mid-year topical symposium in the annual HPS conference for which approval has been separately granted.

4. Also not included is the approval for the 1977 IRPA meeting. Approval was granted for attendance at this meeting on the same basis as the HPS meetings and subject to the same % continuing education credits limitation. See the 1978 ABHP newslette. for additional details.

5. Assigned credits are based on the information provided by the applicant. Because of differences in detail provided a direct comparison of credits assigned cannot be made since the credits assigned a particular course may not reflect the absolute maximum achievable. This also infers that an individual may be able to obtain more credit for a particular course by submitting a more detailed application after attending the course (but within the 90 day rule). While this latter point may be true it is suggested that the Chairman of the CEP be contacted prior to such a submission in order to avoid unnecessary duplicate applications.

Allaland

LESTER A. SLABACK, Jr: Chairman

<u>Title</u> urse on Radiat hysics Certifi

Approval is restricted to individual named on application only.

the application to the CEP/ABHP unless otherwise stated on the approval certificate. Any other type of participation The above credits are based on the assumption of full participation in all aspects of the program as represented in requires separate application to the CEP. \*\*

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Satisfies continuing education requirements for certification renewal SR

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		COURSES AFTROVED BI INC	CONTINUED CONCEPTION FANGL		
Cert	ificate No.	Title	Sponsor/Location	Date Assigne	ed CEC **
	78-11	Summer School on Radiation Protec- tion Dosimetry	Health Physics Society	26-30 Jun 78	SR
	78-12	Safety Controls in Reactor Operations	Rensselaer Polytechnic Institute Troy, NY	Annually Spring- Semester 78-79	SR
	78-13	Sigma Xi Lecture	Rensselaer Polytechnic Institute Troy, NY	4 Apr 78	
	78-14	Radiological Engineering	Rensselaer Polytechnic Institute Troy, NY	Annually Fall Semester 77~78-79	3
	*78-15	Nuclear Engineering Seminar	Rensselaer Polytechnic Institute Troy, NY	Yearly, Fall & Spring Semester 1977-78	9
3	*78-16	Biomagnetic Effects Workshop	Lawrence Berkeley Laboratory Berkeley, CA	6-7 Apr 78	4
62 1	78-17	Annual Conference Meeting	National Conference of Radiation Control Program Directors Little Rock, AR	19-23 Jun 77	5
81	78-18	Symposium of Short Courses in the State of the Art of the Health Physics	Delaware Valley Health Physics Society Chapter, et al Philadelphia, PA	12 May 78	4
	*78-19	Seminar for Industrial Radiographers to Discuss Radiation Safety & NRC Requirements	Region V, USNRC, Walnut Creek, CA	6 Apr 78	•
	78-20	Workshop on TLD	East Tennessee Chapter, HPS Oak Ridge, TN	27 Apr 78	2
	78-21	Primary Management of Radiation Injury	Radiation Management Corp. Philadelphia, PA	.1 Apr 78	6

\* Approval is restricted to individual named on application only.

	SES APPROVED BY	THE CONTINUING EDUCATION PANEL		
Certificate N	Title	Sponsor/Location	Date	Assigned CEC
78-22	On-Line Sample Analysis by Gamma Spectrometry	Continuing Education Courses at Annual HPS Meeting Minneapolis, MN	20-23 Jun 78	-
78-23	Review of ICRP 26-Recommendations of the International Commission on Commission on Radiation Protection	Continuing Education Courses at Annual HPS Meeting Minneapolis, MN	20-23 Jun 78	-
78-24	Rudioactive Waste Disposal Classi- fications	Continuing Education Courses at Annual HPS Meeting Minneapolis, MN	20-23 Jun 78	-
78-25	Transportation of Radioactive Mate- rials - A. Review of Current Regulations	Continuing Education Courses ar Annual HPS Meeting Minneapolis, MN	20-23 Jun 78	1
78-26	Transportation of Radioactive Materials - B. Hazard Assessments in Urban Environments	Continuing Education Courses at Annual HPS Meeting Minneapolis, MN	20-23 Jun 78	1
78-27	Current Status of Personnel Dosimetry	Continuing Education Courses at Annual HPS Meeting Minneapolis, MN	20-23 Jun 78	-
78-28	Radiation Surveillance	Harvard School of Public Health Boston, MA	13-17 Jun 78	See 78-5
18-29	Radiation Shielding Course	Portland General Electric Co. Portland, OR	15-19 May 78	4
78-30	Laser/Microwave Hazards Course	Health Physics Society, Northern California, Livermore, CA	22 Mar 78	3
78-31	Personnel Monitoring	Greater New York Chapter HPS Columbia Univ., New York, NY	<sup>9</sup> 8 Mar 78	-

\* Approval is restricted to individual named on application only.

	COURSES APPROVED BY	THE CONTINUING EDUCATION PANEL		
Certificate No	Title	Sponsor/Location	Date	Assigned CEC
78-32	Health Physics & Radiation Protection	Nuclear Regulatory Commission/Oak Ridge Assoicated Universities Oak Ridge, TV	77-78	8
78-33	Orientation Course in Regulatory Practices & Procedures	Nuclear Regulatory Commission Bethesda, MD	77-78	3
78-34	Inspection Procedures	NRC, Region III	17-78	2
78-35	Course in Medical Use of Radio- nuclides for State Regulatory Personnel	NRC, Baylor College of Medicine The Methodist Hospital	77-78	£
78-36	Seminar On Calibration of Teletherapy Machines	NRC, Univ. of Texas System Cancer Center, Houston, TX	17-78	3
78-37	Course in Safety Aspects of Industrial Radiography	NRC, Louisiana State University	77-78	-
78-38	Gas & Oil Well Logging for State Regulatory Personnel	NRC, Schlumberger Well Services Houston, TX	77-78	-
68-81* 62	Envir. Radiation - Sources and Measurement Techniques	Greater New York Chapter, NPS	16 May 78	-
18-40	The Teaching of Medical Physics	AAPM Summer School Course for 78 Univ of California, Los Angeles, C	23-29 Jul 78	4
\$78-41	PWR Fundamentais	U.S.N.R.C. Washington, DC	17-21 Apr 78	4
78-42	Conference of Radiation Control Program Directors	NRC, BRH, State of Pa., etc. Marrisburg, PA	1-4 May 78	4
\$18-43	BWR Fundamentals	U.S.N.R.C. Washington, DC	9-13 Jan 78	4

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#### COURSES APPROVED BY THE CONTINUING EDUCATION PANEL

Certificate No.	Title	Sponsor/Location	Date	Assigned CEC
78-44	Preparation Course for the ABMP Certification Examination	Baltimore-Washington Health Physics Society	11 Jan-17 May 3	77 6
*78-45	Preparation Course for the ABHP Certification Examination	Baltimore-Washington Health Physics Society	11 Jan-17 May 7	78 4
78-46	Innovations in Practical Health Physics Technology & Methods	North Carolina Chapter, HPS Raleigh, NC	5 May 78	2
*78-47	Applications of Reliability & Risk Analysis with Emphasis on Nuclear Yower Plants	George Washington University Washington, DC	10-14 Apr 78	4
78-48	5th Intl Symposium on Packaging & Transp of Radioactive Waste	Sandia Corp. Albuquerque, NM	7-12 May 78	6
78-49	Environmental Protection Criteria for Radioactive Waste	EPA Waste Environmental Standards Washington, DC	3/30-4/1/78	1
78-50	BWR/PWR Radwaste	U.S. Nuclear Regulatory Commission Washington, DC	26-30 Jun 78	13
N 78-51	Not used			
→ 78-52 →	Medical Management of Radiation Casualties	Yankee Atomic Electric Co. & Peter Bent Brigham Nospital Westborough, MA	13 Oct 78	4
78-53	Basic Radiological Defense Officer Course	University of Lowell Lowell, MA	1978-79	2
78-54	Fall Meeting Nuclear Power	Alabama Chapt'r of the MPS Muscle Shoals, AL	13-14 Oct 78	2

	Assigned CEC	3	-	1	2	2	3	2	lec 78 11
	Date	a 3-5 Oct 78	13-14 Oct 78	13-14 Oct 78	on 18-20 Sep 78	4-5 Oct 78	12/8-15/77 10/16-20/78	13-17 Mar 78	5 Sep - 13 D
THE CONTINUING EDUCATION PANEL <sup>+</sup>	Sponsor/Location	U.S. Nuclear Regulatory Commissio Washington, DC	North Carolina Chapter, HPS Chapel Hill, NC	East Tennessee & Atlanta Chapters HPS, Oak Ridge, TN	U.S. Nuclear Regulatory Commissio Washington, DC	Roswell Park Memorial Institute Buffalo, NY	DOE Sys. Saf. Dev. Ctr Clearwater, FL	Staff College, Dcfense Civil Preparedness Agency Battle Creek, MI	Catholic University of America Washington, DC
COURSES APPROVED BY	Title	1978 Ail Agreement State Meeting	Radiation Emergency Planning	Nuclear Waste Management	Wast <sup>2</sup> Management Contractors *	Reduced Dose Mammography	Basic MORT Seminar	Advanced Radiological Defense Officer	Nuclear Fuel Cycle
	Certificate No.	78-55	78-56	78-57	78-58	*78-59	78-60	78-61	*78-62

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## COURSES APPROVED BY THE CONTINUING EDUCATION PANEL\*

Certification No.	Title	Sponsor/Location	Date	Assigned CEC
79-1	Spring Seminar AAPM	Southern California Chapter AAPM Loma Linda University Hospital Loma Linda, CA	23-25 Apr 79	2
79-2	Not used			
*79-3	Medical Oncology MEDI 604M	Foundation for Advanced Education in the Sciences, Inc., National Institutes of Health, Bethesda, MD	2/5-5/25/79	2
79-4	Recent Developments in Applied Health Physics	Health Physics Society Richland, WA	3-4 Feb 79	3
79-5	Short Course on Radiation Protection	The University of Michigan Ann Arbor, MI	1-12 May 78	3
79–6	Microwaves Laser & Ultraviolet Biophysical & Biological Basic Applications & Hazards in Medicine and Industry	Mass. Institute of Technology Cambridge, MA	1978-79 Presentations	9
79-7	Ionizing & Nonionizing Radiation in Mediciae Theory - Practice - Protection (Summer Presentation)	Health Physics Society Bethesda, MD	2-6 Jul 79	10
79-8	Application of Optical Instrumen- tation in Medicine VII	SPIE (BRH So-sponsor) Bellingham, WA	25-27 Mar 79	In review
79-9	Neutrons for Electron Medical Accelerators	NBS, BRH	9-10 Apr 79	In review

\* Approval is restricted to individual named on application only.

National Registry of Radiation Protection Technologists

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## DEFINITION OF A RADIATION PROTECTION TECHNOLOGIST

A Radiation Protection Technologist is a person engaged in the operational aspects of providing radiation protection. This individual is concerned with:

- 1. The basic understanding of the mechanisms of radiation damage.
- The development and implementation of methods and procedures necessary to evaluate hazards, and
- Providing protection to man and his environment from unwarranted radiation exposure.

#### HISTORY

## NATIONAL REGISTRY OF RADIATION PROTECTION TECHNOLOGISTS

The National Registry of Radiation Protection Technologists (NRRPT) came into existance as a small working committee, consisting of three Health Physics technicians selected by the Health Physics Society Board of Directors. The name, of Health Physics technicians were submitted by laboratories and industries from across the United States.

The committee, under the direction of the American Board of Health Physics, met in Miami. Florida. in June, 1973, and was appointed to study and make recommendations to the Health Physics Society Board of Directors as to the desirability, feasibility, and implementation of a Health Physics technician registry program.

From Knoxville, Tennessee, October 1974, the committee reported to the American Board of Health Physics the desirability of a registry program. The committ. was assigned the task of developing a proposed program to initiate the Registry. Additional Health Physics technician names were solicited from local chapters. The initial NRRPT Board of Dirr.tors was selected.

The Board (NRRPT) of Directors met November 20, 1975, in Rockville, Maryland. Officers of the Board were elected, an examination panel selected, and assignments were made to develop By-Laws for the Registry and to explore the funding possibilities.



The Board met sgain February 1976, in Denver, Colorado, to complete the registry program and stopt the By-Laws. This work was necessary prior to submitting an application for incorporation. The NRRPT was incorporated April 12, 1976, in the State of Tennessee.

The first examination was administered in November of 1976.

EXCERPTS FROM THE NERPT BY-LAWS

#### ARTICLE I NAME

The name of the organization shall be the National Registry of Radiation Protection Technologists, which hereinafter shall be designated the Registry. An elected Board of Directors will administer the registry program and will hereinafter be designated the Board.

ARTICLE II OBJECTIVES

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The objectives of the Board shall be to develop standards and procedures for the registration of Radiation Protection Technologists: to institute examinations leading to registration; and to issue written proof of registration to individuals who possess the required cualifications for registration.

#### ARTICLE III BOARD OF DIRECTORS

The present membership of the Board is ten (10) and can be no greater than twelve (12), who shall, themselves, be registered Radiation Protection Technologists or certified Health Physicists.

They shall act as individuals and not as representatives of any organization. The Board shall consist of not less than one (1) and not more than two (2) persons proposed and recommended by the American Board of Health Physics. Other members of the Registry may be proposed and sponsored by organizations other than the American Board of Health Physics or as invited by the Board.

An Examination Panel consisting of Registered Radiation Protection Technologists appointed by the Board prepares, administers, and grades the written examination with assistance from a Professional examination service and under the guidance and

## **POOR ORIGINAL**

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#### approval of the Board.

#### PURPOSES OF THE BOARD

 To evaluate the standards and advance the profession of applied Health Physics by encouraging its study and improving its practice.

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- To encourage and insist on the highest standards of professional ethics and integrity in the profile of radiation protection.
- To determine the competence of specialists in radiation protection and to arrange control and conduct investigations and examinations, to test the qualifications of voluntary candidates for registration by the Board.
- To grant and issue certificates in the field of applied Health Physics to voluntary applicants and maintain a registry of those who qualify.

#### MEANING OF REGISTRATION

The Certificate of Registration indicates that its holder has completed certain requirements of study and competence which the Board considers to constitute an adequate foundation in Radiation Protection and has passed an exam designed to test the individual's knowledge in this field.

It chould be recognized that the Certificate of Registration by the Board is not a license and, therefore, does not confer a legal qualification to practice Health Physics.

#### NRRPT - CODE OF ETHICS

In achieving registration, the Registered Radiation Protection Technologist recognizes and assumes responsibilites due the profession of radiation protection.

In order to maintain their technical competence, the Registered Technologist, while active in the field of radiation protection, has a cummitment to maintain their technical competence. This shall be accomplished by remaining acquainted with scientific, technical, and regulatory development in their field.

In order to uphold the integrity of the profession of radiation protection implied in this registry, their relations with others, including supervision.



colleagues, governmental agencies, and the general public shall be baked upon and reflect the highest standards of professional ethics and conduct.

Registration may be revoked for action cons ered by the Board to be in violation of these "Code of Ethics." Any pers for whom such action is contemplated shall have the right of appearance before the Board.

NATIONAL REGISTRY OF RADIATION PROTECTION TECHNOLOGIST PROGRAM

The Certificate of Registration certifies that its holder has met the general requirements and passed a multiple choice examination to test competence in fundamental concepts required as a Radiation Protection Technologist. A registered Radiation Protection Technologist shall be of good moral character and shall have met all the requirements listed herein.

I. Requirements for Registration

A. Education

The applicant shall have a high school diploma or equivalent. In addition, the applicant must submit to the Board evidence of operational ab 'ities as a Radiation Protection Technologist.

8. Age

Minimum age at time of application shall be 21 years.

C. Experience

An applicant must have a minimum of five (S) years experience. Training may be substituted for experience if the applicant will submit to the Board information about the program and proof of completion. This information should include curriculum, typical examinations, and passing requirements for radiation protection related subjects.



Training Program	Experience Credit
Four years - 85, Radiation Protection Technolog	ay 2 vears
iwo years - Radiation Protection Technology	l year
One year - Radiation Protection Technology	1/2 year
Company Training Program	1/2 year
• Military Training	

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Experience credit allowed for training programs is accumulative up to a maximum of two (2) years. An applicant may not claim work experience while in formal classroom study.

\*Military training programs will be evaluated by the Board on an individual basis up to a maximum of 1 year.

#### 9. Other Requirements

At the time of application, an applicant sust be engaged in radiation protection a substantial portion of the time.

#### E. Txamination

Upon investigation of the applicant and approval by the Board, the applicant will be given permission to take the National written exam. The National exam will consist of a '50 question, broad base multiple choice examination in radiation protection, (1) fundamentals, (2) measurements, and (3) applied operational concept:, with a four hour time limit.

#### II. Applications

Applications may be submitted individually or as a group by a Company or Facility. Group applications must provide a completed application form for each individual, however, only one copy of the Company's or Facility's Training Program need be enclosed if experience credit is desired for individual applicants within the group. Group applications must provide individual information as required by a single applicant as follows:

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é.

1. Name

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- 2. Address
- 3. Age
- 4. Training programs completed
- 5. Experience
- 5. Present assignment
- 7. Supervisor's signature

## III. Operational Knowledge

A major part of the Board's applicant "screening" will be the investigation of field competence and ability to perform radiation protection work. The screening includes examination of the applicant's work history, institutional and/or in-plant training programs completed, and an interview of persons knowledgeable of the applicant's abilities

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To facilitate this review, the applic hould include detailed information regarding the following:

1. Companies worked for, positions held and responsibilities nile in these positions.

- 2. Information regarding training programs completed; if possible, include course outlines, typical examinations and passing requirements.
- 3. Names of supervisors and knowledgeable persons whom the Board may
- interview to further establish competence of the applicant.

IV. National Examinations

A. National examinations shall be administered and controlled by an examination panel appointed by the Board. The examinations shall be proctored by Registered Radiation Protection Technologists, Certified Health P. sicist. or other knowledgeable person approved by the Board. The examinations will be given once or twice each year as demand and capability dictate. The frequency and locations will be determined by the Board attempting to keep

exam sites as close as possible to the applicants. The examination fee will be determined annually by the Board. Permits are required for entry into the examination room. No reference material, including programmable calculators or Health Physics slide rules, may be brought into the room.

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8. Re-examination

A candidate who fails the first examination may be admitted for a second examination after one year and payment of a reduced re-examination fee. If a candidate does not appear for re-examination within two years, he must submit a new application and full fees.

C. Grading and Passing Requirements

The grading and passing requirements will be determined by the Board. The Board's actions on any applicant's examination will be final. Requirements for registration may be changed by the Board without notice. Changes will not be retroactive.

CHANGE IN REQUIREMENT

Current requirements and procedures of the NRRPT are described in this brochure. These are subject to change without notice; however, changes will be published before their effective date whenever practical. Changes will not be retroactive.

CORRESPONDENCE

All correspondence to the NRRPT should be sent to:



#### AN INSIDE VIEW OF

#### AMERICAN BOARD OF HEALTH PHYSICS PROGRAMS

REPORT ON

AMERICAN BOARD OF HEALTH PHYSICS

EXAMINATION PROGRAM

PART I

1968 - 1975

by

Mr. Michael S. Terpilak

For Presentation at the Twentieth Anniversary Meeting

of the

Health Physics Society

July 16, 1975

Buffalo, New York

American Board of Health Physics Chairman, Panel of Examiners c/o Bureau of Radiological Health 12720 Twinbrook Parkway Rockville, Maryland 20852

#### Report on

American Board of Health Physics

Examination Program

1968 - 1975

Background - Nature of Program

In the Spring of 1968, the Panel of Examiners, American Board of Health Physics (ABHP), under Contract No. PH 128-68-1 with the U. S. Public Health Service, Department of Health, Education, and Welfare, in cooperation with the Professional Examination Service (PES) developed a multiplechoice examination for use in certifying health physicists. This contract was supported by the National Center for Radiological Health as a part of the Center's responsibility in assuring that those individuals that carry out radiation protection activities are adequately qualified. This was the first time the ABHP used multiple-choice questions in its qualifying examination which previously consisted entirely of essay questions, presented in two farts. The decision of the Panel of Examiners to use an objective, multiple-choice test was based upon the difficulty and unreliability of scoring essay examinations and the cumbersome and time-consuming procedures required. The Panel felt, however, that the essay form of examination should be retained, at least in part, to enable candidates to identify the specific steps followed in answering the questions. It was therefore decided that Part I, the multiple-choice part of the test, would cover the fundamentals of health physics and represent a general body of informational knowledge, and Part II, the essay test would involve more specialized problems in health physics.

#### Examination Development

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The first meeting of the ABHP Panel of Examiners and the staff of the Professional Examination Service was held on February 20 and 21, 1968. The task of the Panel was to prepare and outline the subject-matter areas to be included in Part I of the Written Examination and to select appropriate test questions from the PES file of questions in radiological health. It was decided that 150 test questions would prat effectively satisfy the limitations of time and the demands of adequate coverage. The Part I Written Examination developed consisted of questions assigned to the following areas:

	Content Area	Number of Que	stions
I.	Fundamentals	50	11.1
II.	Measurement	20	
III.	Occupational Health Physics Problems	25	
IV.	Non-occupational Health Physics Problems	23	
ν.	Health Physics Administration	20	

The examination was so constructed that four scores could be obtained: (a) a total score based on 150 questions; (b) a subscore based on the 50 questions in Section I; (c) a subscore based on the 30 questions in Section II; (d) a subscore based on the 70 questions in Section III, IV, and 7 combined.

A manual of instructions was prepared to guide proctors in administering the test.

The first examination was administered on Jur 2 17, 1968 to 68 candidates qualified for regular certification by the ABHP and to 15 candidates qualified under an associate program for persons lacking the experience requirement for regular certification.

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#### Item Development

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The ABHP established its own bank of examination questions which has been expanded by periodically soliciting new questions from health physicists previously certified by the Board. Question writers are provided with instructions describing the kinds of questions used in the examination. The new questions are screened and classified by subject-matter consultants and by test specialists on the PES staff to ensure content accuracy and relevancy and conformance and psychometric principles. They are then sent to panels of three experts in the field for independent subject-matter review. The reviewers' comments are used by the subject-matter consultants and test editors in the final review of each question. The questions are then submitted to the Panel of Examiners for final approval. Acceptable questions are included in the ABHP file from which the Panel of Examiners select questions for inclusion in each revision of the test.

Since the beginning of the ABHP program in 1968, 65 health physicists have contributed some 400 questions for the examination and these questions have been reviewed by 36 certified specialists in the field.

#### On-Going Examination Development

To assist the ABHP in uppdating and maintaining the initial examination, Federal funding continued through 1971. This support provided for the analysis of the test result; a correlation study between scores on Part I, the multiple-choice examination, and Part II, the essay examination, administered in 1968; solicitation of new questions and the establishment of the Board's own bank of questions; revision of the examination, and the administration of the examinations. Over the four-year period, (1968-1971) government contracts were awarded PES totaling \$29,073.00 for the development of ABHP

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examination materials as follows:

1968	\$ 4,918.00	(PH 128-68-1)
1969	8,355.00	(CPE-R-69-17)
1970	8,000.00	(CPE-R-70,0018)
1971	7,800.00	(68-05-0002)

Since 1971, the ABPH Part I-Written Examination program has been supported by the Board and voluntary subscriptions from certified health physicists. The examination is reviewed annually by members of the Panel of Examiners to maintain the quality and relevance of the Examination, and insure that it covers up-to-date concepts. Revisions are made in the examination by rewording questions or replacing questions on the basis of subject-matter considerations and in conjunction with item analyses. The examination subject-matter cutline was revised in 1973 to reflect more accurately the actual test content, and presently has the following content distribution:

Area	Number	of	Questions
Fundamentals		50	
Measurement		30	
Operational Health Physics		70	

#### Examination Scores

The candidates' marked answer sheets are scored by PES and a report of the results is forwarded to the Board. The score reports present (1) a listing of the raw scores obtained by each candidate for the total test and for each of the three subtests, (2) statistical data based on the group of candidates tested, and additional interpretive information to help the Board evaluate the candidates' performance, including a comparison of the results of the test from year to year and comparisor of the difficulty level of the

subtests and total test between each of the years that the examination has been administered.

#### Examination Usage

 $A_{ij}^{*} = A_{ij}^{*}$ 

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The examination has been administered to a total of 445 candidates . from 1968 through 1974.

## American Board of Health Physics

	Maximum Raw Scores	Range of Raw Scores	Standard Deviations	Average Raw Scores	Average Percent Scores
66 Candidates - 1974					
Total	150	48 - 128	18.86	94.03	62.69
Fundamentals	50	14 - 47	7.85	32.88	65.76
Measurement	30	6 - 27	4.70	17.74	59.14
Operational Healta Ph "cs	70	19 - 63	8.84	43.41	62.01
					0
24 Candidator - 1972					C.,
54 Candidates - 1975					
Total	150	46 - 130	18.94	97.18	64.78
Fundamentala	50	20 - 45	6.66	34.44	66.88
Mcasurement .	30	9 - 27	4.73	19.71	65.69
Operational Health Physics	70	16 - 60	9.33	43.03	61.47
78 Candidates - 1972					
Total	150	45 - 125	18.00	87.47	58.32
Fundamentals	50	8 - 42	7,33	29.01	58.03
Measurement	30	7 - 28	4.67	17.82	59.40
Problems and Admin.	70	20 - 60	8.51	40.64	58.06
45 Candidates - 1971					
Tetal	150	51 - 125	18.23	87.36	58.24
Fundamentals	50	17 - 44	7.13	30.13	60.27
Measurement	30	9 - 27	4.85	17.73	59.11
Problems and Admin.	70	20 - 60	9.19	39.49	56.41
74 Candidates - 1970					
Total	150	40 - 129	17.23	96.28	64.09
Fundamontala	50	13 - 46	6.94	33.76	67.52
Mancurament	30	7 - 28	4.32	18.68	62.27
Problems and Admin.	70	9 - 59	8.58	43.85	62.64

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## Professional Examination Service American Board of Health Physics

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	Maximum Raw Scores	Range of Raw Scores	Standard Deviations	Average Raw Scores	Average Percent Scores
<u>63 Candidates - 1969</u> Total Fundamentals Measurement Problems and Admin.	150 50 30 70	$50 - 128 \\ 15 - 46 \\ 9 - 28 \\ 10 - 60$	18.18 7.12 4.43 9.10	96.81 33.76 18.68 44,37	64.54 67.52 62.27 63.30
67 Candidates - 1968 Total Fundamentals Measurement Problems and Admin.	150 50 30 70	55 - 137 20 - 48 9 - 29 14 - 62	16.13 5.91 4.00 8.03	103.55 37.69 21.85 44.01	69.03 75.38 72.83 62.87

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