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Portland General Electric Company

Charles Goodwin, Jr. Assistant Vice President

May 18, 1979

Docket No. 50-344

Mr. R. H. Engelken, Director  
US Nuclear Regulatory Commission, Region V  
1990 North California Blvd.  
Walnut Creek, California 94596



Dear Sir:

Attached is statistical data concerning the requalification training program at the Trojan Nuclear Plant. This information was requested in IE Bulletin No. 79-10 which was transmitted to us on May 16, 1979.

Sincerely,

C. Goodwin, Jr.  
Assistant Vice President  
Thermal Operations & Maintenance

CG/DFK:cb

Attachment

- c: J. L. Williams
- D. J. Broehl
- R. L. Sullivan
- S. L. Loy
- S. R. Christensen
- Nuclear Operations Board
- Trojan Operating Committee
- Plant Review Board
- L. W. Erickson
- L. Frank, Director, DOE/EFSC
- Office of Management Information and Program Control
- NRC Resident Inspector
- File

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TABLE 1

Utility: Portland General Electric

Date: 5/18/79

Facility: Trojan Nuclear Plant

	Operators				Senior Operators			
	Exam 1	Exam 2	Exam 3	Exam 4	Exam 1	Exam 2	Exam 3	Exam 4
1. Action 1: Responses								
A. Total number failed annual examination	0	0	0		0	0	0	
B. Percent that failed annual examination	0	0	0		0	0	0	
2. Action 2: Responses								
*A. Percent required to attend additional lectures	57.1	50	36.4		0	27.3	16.7	
*B. Number of operators attending specified number of lectures (no. of operators/no. of lectures)	2/1 1/2 1/4	3/1	2/1 1/2 1/3			3/1	2/1	
3. Action 3: Responses								
A. Total number attending accelerated training because 70 percent on annual written examination	0	0	0		0	0	0	
B. Percent attending accelerated training lesson because 70 percent on annual written examination	0	0	0		0	0	0	
C. Total number attending accelerated training because unsatisfactory oral examination	0	0	0		0	0	0	

\* Because <80 percent on any one category on annual written examination.

304  
161

Operators				Senior Operators			
Exam 1	Exam 2	Exam 3	Exam 4	Exam 1	Exam 2	Exam 3	Exam 4

D. Percent attending accelerated training because unsatisfactory oral examination

.0	0	0		0	0	0	
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Date of Exam 1

1/15/77

1/15/77

2

1/15/78

1/15/78

3

1/15/79

1/15/79

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT  
WASHINGTON, D.C. 20555

May 11, 1979

IE Bulletin No. 79-10

REQUALIFICATION TRAINING PROGRAM STATISTICS

Description of Circumstances:

Because of the apparent failure of the operators at Three Mile Island to recognize certain plant conditions and take appropriate action to effectively cool the core and contain fission products after release from the core, the NRC Commissioners are evaluating licensee's requalification training programs. As one element in this evaluation, the NRC is interested in obtaining statistics about the failure rate on the annual requalification examinations. The information requested below along with other information will then be used to evaluate the effectiveness of the operator requalification training program.

Action to be Taken by Licensees:

For all power reactor facilities with an operating license.

1. Provide both the total number and percentage of operators who have failed the annual requalification examination.
2. Provide the percentage of those operators who take the annual requalification examination and are required to attend lectures on categories of material for which they received a grade of less than 80 percent. Also provide the total number of supplemental lectures attended (e.g., 3 operators had to attend 2 lectures, 1 operator had to attend 3 lectures, etc.).
3. Provide both the total number and percentage of operators under the requalification program that participated in accelerated training because they either scored less than 70 percent overall on the annual written examination or had an unsatisfactory performance on the oral examination.
4. Provide the same information required by 1 thru 3 on Senior Operators.

5. Provide the above information from four annual examinations (written and oral). If the requalification program has not been implemented for that period of time or records are not available, provide the maximum information that is available.
6. Record information in format shown in Table 1.
7. Within 14 calendar days of the date of issue of this Bulletin, report in writing to the Director of the appropriate Regional Office the results of your review. A copy of your report should be sent to the United States Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, Division of Project Management, Operator Licensing Branch, Washington, D.C. 20555.

Approved by GAO, B180225 (R0072), clearance expires 7/31/80. Approval was given under a blanket clearance specifically for identified generic problems.

Enclosures:

1. Table 1
2. List of IE Bulletins  
Issued in Last  
Twelve Months

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165

LISTING OF IE BULLETINS  
ISSUED IN LAST TWELVE MONTHS

Bulletin No.	Subject	Date Issued	Issued To
79-09	Failures of GE Type AK-2 Circuit Breaker in Safety Related Systems	4/17/79	All Power Reactor Facilities with an OL or CP
79-08	Events Relevant to BWR Reactors Identified During Three Mile Island Incident	4/14/79	All BWR Power Reactor Facilities with an OL
79-07	Seismic Stress Analysis of Safety-Related Piping	4/14/79	All Power Reactor Facilities with an OL or CP
79-06B	Review of Operational Errors and System Misalignments identified During the Three Mile Island Incident	4/14/79	All Combustion Engineering Designed Pressurized Water Power Reactor Facilities with an Operating Licensee
79-06A	Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	4/14/79	All Pressurized Water Power Reactor Facilities of Westinghouse Design with an OL
79-06	Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	4/11/79	All Pressurized Water Power Reactors with an OL except B&W facilities
79-05A	Nuclear Incident at Three Mile Island	4/5/79	All B&W Power Reactor Facilities with an OL
79-05	Nuclear Incident at Three Mile Island	4/2/79	All Power Reactor Facilities with an OL and CP

LISTING OF IE BULLETINS  
ISSUED IN LAST TWELVE MONTHS

Bulletin No.	Subject	Date Issued	Issued To
79-04	Incorrect Weights for Swing Check Valves Manufactured by Velan Engineering Corporation	3/30/79	All Power Reactor Facilities with an OL or CP
79-03	Longitudinal Weld Defects In ASME SA-312 Type 304 Stainless Steel Pipe Spools Manufactured By Youngstown Welding and Engineering Company	3/12/79	All Power Reactor Facilities with an OL or CP
79-02	Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts	3/02/79	All Power Reactor Facilities with an OL or CP
79-01	Environmental Qualification of Class IE Equipment	2/08/79	All Power Reactor Facilities with an OL or CP
78-14	Deterioration of Buna-N Components In ASCO Solenoids	12/19/78	All GE BWR Facilities with an OL or CP
78-13	Failures In Source Heads of Kay-Ray, Inc., Gauges Models 7050, 7050B, 7051, 7051B, 7050, 7050B, 7061 and 7061B	10/27/78	All General and Specific Licensees with the subject Kay-Ray, Inc. gauges
78-12B	Atypical Weld Material in Reactor Pressure Vessel Welds	3/19/79	All Power Reactor Facilities with an OL or CP
78-12A	Atypical Weld Material in Reactor Pressure Vessel Welds	11/24/78	All Power Reactor Facilities with an OL or CP
78-12	Atypical Weld Material in Reactor Pressure Vessel Welds	9/29/78	All Power Reactor Facilities with an OL or CP

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167



LISTING OF IE BULLETINS  
ISSUED IN LAST TWELVE MONTHS

Bulletin No.	Subject	Date Issued	Issued To
78-11	Examination of Mark I Containment Torus Welds	7/21/78	BWR Power Reactor Facilities for action: Peach Bottom 2 and 3, Quad Cities 1 and 2, Hatch 1, Monticello and Vermont Yankee
78-10	Bergen-Paterson Hydraulic Shock Suppressor Accumulator Spring Coils	6/27/78	All BWR Power Reactor Facilities with an OL or CP
78-09	BWR Drywell Leakage Paths Associated with Inadequate Drywell Closures	6/14/79	All BWR Power Reactor Facilities with an OL or CP
78-08	Radiation Levels from Fuel Element Transfer Tubes	6/12/78	All Power and Research Reactor Facilities with a Fuel Element transfer tube and an OL
78-07	Protection afforded by Air-Line Respirators and Supplied-Air Hoods	6/12/78	All Power Reactor Facilities with an OL, all class E and F Research Reactors with an OL, all Fuel Cycle Facilities with an OL, and all Priority 1 Material Licensees
78-06 78-12	Defective Cutler-Hammer Type M Relays with DC Coils	5/31/78	All Power Reactor Facilities with an OL or CP
78-11			

304 168  
168