

TMI DOCUMENTS

DOCUMENT NO: TM-0649

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PEA
Supervisor, Document Control, NRC

7906180695

229 092

SURVEILLANCE PROCEDURE NAME 2301-M1 Post Accident Monitoring Int

SURVEILLANCE PROCEDURE NUMBER 2301-M1

DATE DATA TAKEN 3-26-79. THIS SURVEILLANCE IS REQUIRED FOR MODE(S) 1, 2, 6

MODE(S) DATA TAKEN IN 1

NOTE: Use a schedule form for each E or D recorded.

COMPONENT DESIGNATOR				LOCATION UNIT	TYPE TASK	TASK I.D.	SCHEDULE NO.
SYS	TYPE	I.D.	LP				
5			15 16 17		23 24 25		32 33 34
TMI	2301M	11		036002	TSS	2301-M	1790870

E/D NUMBER/PARAGRAPH OR SECT. / Description (If Appl.)

403A	01	Pressure	2	Zero	Level	Inst	RC	1	LT3	15
404C	01	more	than	3	inches	5	from	LT2	and	15
403A	02	Ne	4	7	4					
404C										
403A										
404C										
403A										
404C										

RESOLUTION DESCRIPTION (If Appl.)

TAN No.	SEQ No.	Description
403A	03	Low
404C	03	at
403A		
404C		
403A		
404C		

SHIFT SUPERVISOR COMPLETE SECTION BELOW:

TEST COORDINATOR COMPLETE BEL

YES	NO	QUESTION
	✓	1. This E or D placed the Unit into an action statement.
	✓	2. This E or D was caused by Equipment Failure.
	✓	3. This E or D has caused the performance of this Surveillance to be unsatisfactory for Mode(s) <u>ALL</u> with regard to satisfying the intent of the Tech Spec.

The Resolution column has fully explained what has been done to close out this item. (If App.)

This E or D is closed:

Shift Supervisor Initials & Date:

229 094

Signature of the Test Coord.

3an 3-26-79
Initials Date

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WORK

COPY THREE MILE ISLAND NUCLEAR STATION
UNIT #2 SURVEILLANCE PROCEDURE 2301-M1
POST ACCIDENT MONITORING INSTRUMENTATION CHANNEL CHECK

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2.0	10/29/77	0	27.0			52.0		
3.0	04/27/78	1	28.0			53.0		
4.0	04/27/78	1	29.0			54.0		
5.0	10/29/77	0	30.0			55.0		
6.0	10/29/77	0	31.0			56.0		
7.0	10/29/77	0	32.0			57.0		
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10.0			35.0			60.0		
11.0			36.0			61.0		
12.0			37.0			62.0		
13.0			38.0			63.0		
14.0			39.0			64.0		
15.0			40.0			65.0		
16.0			41.0			66.0		
17.0			42.0			67.0		
18.0			43.0			68.0		
19.0			44.0			69.0		
20.0			45.0			70.0		
21.0			46.0			71.0		
22.0			47.0			72.0		
23.0			48.0			73.0		
24.0			49.0			74.0		
25.0			50.0			75.0		

Unit 1 Staff Recommends Approval Approval <u>NA</u> Date _____ Cognizant Dept. Head	Unit 2 Staff Recommends Approval Approval <u>NA</u> Date _____ Cognizant Dept. Head
Unit 1 PORC Recommends Approval <u>NA</u> Date _____ Chairman of PORC	Unit 2 PORC Recommends Approval <u>J. F. Sullivan</u> Date <u>4/26/78</u> Chairman of PORC
Unit 1 Superintendent Approval <u>NA</u> Date _____	Unit 2 Superintendent Approval <u>J. F. Sullivan</u> Date <u>4/27/78</u>
Manager Generation Quality Assurance Approval <u>NA</u> Date _____	

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
Power Range Nuclear Flux		
NI-5 & Amplifier	<u>97</u>	Power Range Indication Within $\pm 2\%$ Flux of each other
NI-6 & Amplifier	<u>97</u>	
NI-7 & Amplifier	<u>95</u>	
NI-8 & Amplifier	<u>97</u>	
Reactor Building Pressure		
BS-PT-4383-1	<u>0</u>	W.R. Press. ± 1.0 Psi of each other
BS-PT-1412-1	<u>0</u>	
BS-PT-4383-2	<u>0</u>	N.R. Press. ± 0.15 psi of each other
BS-PT-1412-2	<u>0</u>	
Core Flood Tank Level		
CF-2-LI1	<u>13.1</u>	Level for Tank A ± 4.5 in. of each other
CF-2-LI2	<u>13.1</u>	
CF-2-LI3	<u>13.0</u>	Level for Tank B ± 4.5 in. of each other
CF-2-LI4	<u>13.1</u>	
RC Outlet Temperature		
RC-4A-TT1	<u>604</u>	Temp. $\pm 1.5^{\circ}\text{F}$ of each other
RC-4A-TT4	<u>604</u>	
RC-4B-TT1	<u>605</u>	
RC-4B-TT4	<u>605</u>	
RB Dome Rad. Monitor		
HP-R-214	<u>5</u> Backg.	Definite increase in CPM
	<u>8</u> C.S.	

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
RC Loop Press		
RC-3A-PT1	<u>2160</u>	Press. ± 24 psig of each other
RC-3B-PT1	<u>2158</u>	
RC-3A-PT2	<u>2140</u>	
RC-3B-PT2	<u>2139</u>	
Pressurizer Level		
RC-1-LT1	<u>226</u>	Level ± 8 inches of each other
RC-1-LT2	<u>223</u>	
RC-1-LT3	<u>232</u> E-1	
Steam Generator Startup Level		
SP-1A-LT4	<u>160</u>	Level for OTSG A ± 1 of each other
SP-1A-LT5	<u>160</u>	
SP-1B-LT4	<u>164</u>	Level for OTSG B ± 1 of each other
SP-1B-LT5	<u>164</u>	
Steam Generator Operating Level		
SP-1A-LT2	<u>56</u>	Level for OTSG A $\pm 2\%$ of each other
SP-1A-LT3	<u>56</u>	
SP-1B-LT2	<u>55</u>	Level for OTSG B $\pm 2\%$ of each other
SP-1B-LT3	<u>55</u>	
BWST Level		
DH-3-LT1	<u>54.69</u>	Level ± 1.1 ft. of each other
DH-3-LT2	<u>54.62</u>	

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
High Press. Inj. Flow		
MU-23-dpt1	<u>0</u>	Flow ± 4.5 GPM of each other
MU-23-dpt2	<u>0</u>	
MU-23-dpt3	<u>0</u>	
MU-23-dpt4	<u>0</u>	
Low Press. Inj. Flow		
DH-1-dpT1	<u>0</u>	N/A
DH-1-dpT2	<u>0</u>	N/A
RB Spray Pump Flow		
BS-1-dpT1	<u>0</u>	Flow ± 27 GPM of each other
BS-1-dpT2	<u>0</u>	
Steam Generator Pressure		
SP-6A-PT1	<u>900</u>	Press. for OTSG A ± 18 psig of each other
SP-6A-PT2	<u>900</u>	
SP-6B-PT1	<u>900</u>	Press. for OTSG B ± 18 psig of each other
SP-6B-PT2	<u>900</u>	

NOTE: If acceptance criteria is not met proceed with ACTION statement 3.3.3.6 of the Technical Specifications.

Performed By: *John W. Quinn* Date: 3-26-79
Approved By: *C. Guthrie* Date: 3-26-79

CONTROLLED COPY THREE MILE ISLAND NUCLEAR STATION
UNIT #2 SURVEILLANCE PROCEDURE 2301-M1
POST ACCIDENT MONITORING INSTRUMENTATION CHANNEL CHECK

WORK

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4.0	04/27/78	1	29.0			54.0		
5.0	10/29/77	0	30.0			55.0		
6.0	10/29/77	0	31.0			56.0		
7.0	10/29/77	0	32.0			57.0		
8.0	04/27/78	1	33.0			58.0		
9.0			34.0			59.0		
10.0			35.0			60.0		
11.0			36.0			61.0		
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18.0			43.0			68.0		
19.0			44.0			69.0		
20.0			45.0			70.0		
21.0			46.0			71.0		
22.0			47.0			72.0		
23.0			48.0			73.0		
24.0			49.0			74.0		
25.0			50.0			75.0		

Unit 1 Staff Recommends Approval

Approval NA Date _____
Cognizant Dept. Head

Unit 2 Staff Recommends Approval

Approval NA Date _____
Cognizant Dept. Head

Unit 1 PORC Recommends Approval

NA Date _____
Chairman of PORC

Unit 2 PORC Recommends Approval

J. J. [Signature] Date 4/26/78
Chairman of PORC

Unit 1 Superintendent Approval

NA Date _____

Unit 2 Superintendent Approval

J. J. [Signature] Date 4/27/78

Manager Generation Quality Assurance Approval

NA Date _____

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
Power Range Nuclear Flux		
NI-5 & Amplifier	<u>98.5</u>	Power Range Indication Within $\pm 2\%$ Flux of each other
NI-6 & Amplifier	<u>98.5</u>	
NI-7 & Amplifier	<u>99.</u>	
NI-8 & Amplifier	<u>100</u>	
Reactor Building Pressure		
BS-PT-4383-1	<u>0</u>	W.R. Press. ± 1.0 Psi of each other
BS-PT-1412-1	<u>0</u>	
BS-PT-4383-2	<u>-0.1</u>	N.R. Press. ± 0.15 psi of each other
BS-PT-1412-2	<u>-0.2</u>	
Core Flood Tank Level		
CF-2-LI1	<u>13'</u>	Level for Tank A ± 4.5 in. of each other
CF-2-LI2	<u>12' 11"</u>	
CF-2-LI3	13' 2" 13'	Level for Tank B ± 4.5 in. of each other
CF-2-LI4	<u>13' 2"</u>	
RC Outlet Temperature		
RC-4A-TT1	<u>605</u>	Temp. $\pm 1.5^\circ\text{F}$ of each other
RC-4A-TT4	<u>605.5</u>	
RC-4B-TT1	<u>605.5</u>	
RC-4B-TT4	<u>605</u>	
RB Dome Rad. Monitor		
HP-R-214	<u>32 mr/hr</u> Backg.	Definite increase in CPM
	<u>41 mr/hr</u> C.S.	

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
RC Loop Press		
RC-3A-PT1	<u>2140</u>	Press. ± 24 psig of each other
RC-3B-PT1	<u>2140</u>	
RC-3A-PT2	<u>2125</u>	
RC-3B-PT2	<u>2125</u>	
Pressurizer Level		
RC-1-LT1	<u>238</u>	Level ± 8 inches of each other
RC-1-LT2	<u>220</u>	
RC-1-LT3	<u>227</u>	
Steam Generator Startup Level		
SP-1A-LT4	<u>161</u>	Level for OTSG A ± 5 in. of each other
SP-1A-LT5	<u>162</u>	
SP-1B-LT4	<u>165</u>	Level for OTSG B of each other
SP-1B-LT5	<u>165</u>	
Steam Generator Operating Level		
SP-1A-LT2	<u>57</u>	Level for OTSG A $\pm 2\%$ of each other
SP-1A-LT3	<u>56.5</u>	
SP-1B-LT2	<u>58</u>	Level for OTSG B $\pm 2\%$ of each other
SP-1B-LT3	<u>56.5</u>	
BWST Level		
DH-3-LT1	<u>56</u>	Level ± 1.1 ft. of each other
DH-3-LT2	<u>56</u>	

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
High Press. Inj. Flow		
MU-23-dpt1	<u>0</u>	Flow ± 4.5 GPM of each other
MU-23-dpt2	<u>0</u>	
MU-23-dpt 3	<u>0</u>	
MU-23-dpt4	<u>0</u>	
Low Press. Inj. Flow		
DH-1-dpT1	<u>0</u>	N/A
DH-1-dpT2	<u>0</u>	N/A
RB Spray Pump Flow		
BS-1-dpT1	<u>0</u>	Flow ± 27 GPM of each other
BS-1-dpT2	<u>0</u>	
Steam Generator Pressure		
SP-6A-PT1	<u>903</u>	Press. for OTSG A ± 18 psig of each other
SP-6A-PT2	<u>908</u>	
SP-6B-PT1	<u>905</u>	Press. for OTSG B ± 18 psig of each other
SP-6B-PT2	<u>910</u>	

NOTE: If acceptance criteria is not met proceed with ACTION statement 3.3.3.6 of the Technical Specifications.

Performed By: *J. B. Ellis* Date: 2/28/79
Approved By: *C. S. Hume* Date: 2/28/79

229 103

DATE PRINTED 01/10/79 SURVEILLANCE PERFORMANCE FORM MET-ED T M I UNIT 2 EARLY DATE 01-21-79 SCHED DATE 01-28-79** LATE DATE 02-04-79

PROCEDURE NO 2301-M 1 TECH/SPEC REF 4.3.3.6 DEPT RESP = OPERATIONS TASK NO = 2301-M 1 POST ACCIDENT MONITORING INST CH CK

DEPENDENT TASK WORK ORDER NO. = 036000326 ACCOUNT NO. = 520.1 GC CODE = 29 COMPONENT NO = T-1-2301-M 1- COMPONENT DESC = STANDARD TECH SPEC ITEM COMP LOCATION = ROD LVL GRID

SPECIAL COMMENT = PLANT CONDITION 1-1-0-0-0-0-1

ASSIST DEPT FREQUENCY M QUALITY CONTROL 1 SPECIFIC DAY CONTRACTOR 0 INTERFERENCE 0 PRIORITY 1 COMPONENT STATUS 1

***** COMPLETE THIS SECTION *****

(1) (5) (38) 401CT 12301M 1 036002TS2301-M 1740280 * COMP NO ** CSU* * TASK ** SCHED*

RESULTS (51) DATE PERFORMED (39) 10 2 1979 MONTH DAY YEAR CHECK ONE ONLY PERFORMED EXCEPTIONS DEFICIENCIES (14 BOTH E S AND D S) (15 NOT PERFORMED) ACTUAL HOURS (45) 000011.00 ACTION TAKEN CODE (52) L L I REASON NOT PERFORMED (54) L L I ABNORMAL OCC REPT (56) L L L L I

PERFORMED BY EMPLOYEE NUMBER (60) 054911 SIGNATURE - [Signature] APPROVED BY EMPLOYEE NUMBER (65) 058441 SIGNATURE - J. Scherman WITNESSED BY EMPLOYEE NUMBER (70) L L L L L I SIGNATURE - CORRECTIVE MAINTENANCE JOB TICKET NUMBER (75) L L L L L I

403A (1) DUPLICATE AS ABOVE (5-38) 402A (1) DUPLICATE AS ABOVE (5-38)

RESULTS DESCRIPTION ASSISTING DEPARTMENTS L O L I L L L L L L L L L L L L L L L L L L L I (39) CODE (39) L L L L L I L I (51) HOURS (44) L L L L L I . L I

404A (1) DUPLICATE AS ABOVE (5-38) CODE (50) L L L L L I L O L I L L L L L L L L L L L L L L L L L I (39) HOURS (55) L L L L L I . L I L I (61)

**UNIT 2 AND STATION
TECH SPEC SURVEILLANCE
Exception and Deficiency List**

SURVEILLANCE PROCEDURE NAME Post Accident Monitoring Inst. Ch Check

SURVEILLANCE PROCEDURE NUMBER 2301-M1

DATE DATA TAKEN 2/2/79 THIS SURVEILLANCE IS REQUIRED FOR MODE(S) 1, 2, + 3

MODE(S) DATA TAKEN IN 1

Note: Use a separate form for each E or D recorded.

COMPONENT DESIGNATOR				LOCATION UNIT	TASK I.D.	SCHEDULE NO.
SYS	TYPE	I.D.	LD			
5		18 16	17	23 24 25	32 33	38
TMI	2301-M1			03000	2301-M1	TECH SPEC
E/D NUMBER / PARAGRAPH OR SECT. / Description (If Appl.)						
403A						01E-1? PAGE 7: RC LOOP PRESSURES GREATER THAN
404C						01HAW = 24 PSIP APART. RC36-PT1 AND RC38-PT
403A						022 RE-40 = 25 PSIP HIGHER THAN RC-34-PT1 AND
404C						020 RC-34-PT2. COMPARISON WITH PANEL 4
403A						03WARRANTY RANGE LOOP PRESS RECORDERS INDICATE
404C						03TEST THAT RC-38-PT1 AND PT2 ARE IN ERROR
403A						
404C						

RESOLUTION DESCRIPTION (If Appl.)

TXN No.	SEC#	Description
403A		04... IN RPS CIRCUITS WERE ATTEMPTED TO
404C		04... WITH COMPUTER
403A		
404C		
403A		
404C		

SHIFT SUPERVISOR COMPLETE SECTION BELOW:

TEST COORDINATOR COMPLETE BELOW

YES	NO	QUESTION
<input checked="" type="checkbox"/>	<input checked="" type="checkbox"/>	1. This E or D placed the Unit into an action statement.
<input checked="" type="checkbox"/>	<input type="checkbox"/>	2. This E or D was caused by Equipment Failure.
<input type="checkbox"/>	<input type="checkbox"/>	3. This E or D has caused the performance of this Surveillance to be unsatisfactory for Mode(s) _____ with regard to satisfying the intent of the Tech Spec.
		Shift Supervisor Initials & Date: <u>[Signature]</u> <u>2/2/79</u>
		Initials Date

The Resolution column has fully explained what has been done to close out this item. (If Appl.)

This E or D is closed:

[Signature] 02-06-79
Signature of the Test Coord. Date

229 105

CONTROLLED COPY THREE MILE ISLAND NUCLEAR STATION
UNIT #2 SURVEILLANCE PROCEDURE 2301-M1
CONTROL POST ACCIDENT MONITORING INSTRUMENTATION CHANNEL CHECK

WORKING

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3.0	04/27/78	1	28.0			53.0		
4.0	04/27/78	1	29.0			54.0		
5.0	10/29/77	0	30.0			55.0		
6.0	10/29/77	0	31.0			56.0		
7.0	10/29/77	0	32.0			57.0		
8.0	04/27/78	1	33.0			58.0		
9.0			34.0			59.0		
10.0			35.0			60.0		
11.0			36.0			61.0		
12.0			37.0			62.0		
13.0			38.0			63.0		
14.0			39.0			64.0		
15.0			40.0			65.0		
16.0			41.0			66.0		
17.0			42.0			67.0		
18.0			43.0			68.0		
19.0			44.0			69.0		
20.0			45.0			70.0		
21.0			46.0			71.0		
22.0			47.0			72.0		
23.0			48.0			73.0		
24.0			49.0			74.0		
25.0			50.0			75.0		

Unit 1 Staff Recommends Approval Approval <u>NA</u> Date _____ Cognizant Dept. Head	Unit 2 Staff Recommends Approval Approval <u>NA</u> Date _____ Cognizant Dept. Head
Unit 1 PORC Recommends Approval <u>NA</u> Date _____ Chairman of PORC	Unit 2 PORC Recommends Approval <u>J. F. Schubert</u> Date <u>4/26/78</u> Chairman of PORC
Unit 1 Superintendent Approval <u>NA</u> Date _____	Unit 2 Superintendent Approval <u>J. L. Schmitz</u> Date <u>4/27/78</u>
Manager Generation Quality Assurance Approval <u>NA</u> Date _____	

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
Power Range Nuclear Flux		
NI-5 & Amplifier	<u>94</u>	Power Range Indication Within <u>+2%</u> Flux of each other
NI-6 & Amplifier	<u>95</u>	
NI-7 & Amplifier	<u>95</u>	
NI-8 & Amplifier	<u>95</u>	
Reactor Building Pressure		
BS-PT-4388-1	<u>+7</u>	W.R. Press. <u>+1.0</u> Psi of each other
BS-PT-1412-1	<u>+7</u>	
BS-PT-4388-2	<u>+5</u>	N.R. Press. <u>+0.15</u> psi of each other
BS-PT-1412-2	<u>+5</u>	
Core Flood Tank Level		
CF-2-LI1	<u>13.1</u>	Level for Tank A <u>+4.5</u> in. of each other
CF-2-LI2	<u>13.0</u>	
CF-2-LI3	<u>13.0</u>	Level for Tank of each other
CF-2-LI4	<u>13.2</u>	
RC Outlet Temperature		
RC-4A-TT1	<u>605</u>	Temp. <u>+1.5</u> ⁰ F of each other
RC-4A-TT4	<u>604</u>	
RC-4B-TT1	<u>605</u>	
RC-4B-TT4	<u>604</u>	
RB Dome Rad. Monitor		
HP-R-214	<u>2 × 10⁴</u> ^{10² scale} Backg. <u>7 × 10⁴</u> C.S.	Definite increase in CPM

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
RC Loop Press		
RC-3A-PT1	<u>2160</u>	E-1 D-Tower Press. ± 24 psig of each other
RC-3B-PT1	<u>2190</u>	
RC-3A-PT2	<u>2150</u>	
RC-3B-PT2	<u>2185</u>	
Pressurizer Level		
RC-1-LT1	<u>220</u>	Level ± 8 inches of each other
RC-1-LT2	<u>224</u>	
RC-1-LT3	<u>224</u>	
Steam Generator Startup Level		
SP-1A-LT4	<u>150</u>	Level for OTSG A ± 5 in. of each other
SP-1A-LT5	<u>150</u>	
SP-1B-LT4	<u>152.5</u>	Level for OTSG B ± 5 in. of each other
SP-1B-LT5	<u>152.5</u>	
Steam Generator Operating Level		
SP-1A-LT2	<u>51</u>	Level for OTSG A $\pm 2\%$ of each other
SP-1A-LT3	<u>51</u>	
SP-1B-LT2	<u>52.5</u>	Level for OTSG B $\pm 2\%$ of each other
SP-1B-LT3	<u>51</u>	
BWST Level		
DH-3-LT1	<u>55.5</u>	Level ± 1.1 ft. of each other
DH-3-LT2	<u>55.4</u>	

DATA SHEET 1

DATA DESCRIPTION	READING	ACCEPTANCE CRITERIA
High Press. Inj. Flow		
MU-23-dpt1	<u>0</u>	Flow ± 4.5 GPM of each other
MU-23-dpt2	<u>0</u>	
MU-23-dpt 3	<u>0</u>	
MU-23-dpt4	<u>0</u>	
Low Press. Inj. Flow		
DH-1-dpT1	<u>0</u>	N/A
DH-1-dpT2	<u>0</u>	N/A
RB Spray Pump Flow		
BS-1-dpT1	<u>0</u>	Flow ± 27 GPM of each other
BS-1-dpT2	<u>0</u>	
Steam Generator Pressure		
SF-6A-PT1	<u>790</u>	Press. for OTSG A ± 18 psig of each other
SP-6A-PT2	<u>702</u>	
SP-6B-PT1	<u>900</u>	Press. for OTSG B ± 18 psig of each other
SP-6B-PT2	<u>910</u>	

NOTE: If acceptance criteria is not met proceed with ACTION statement 3.3.3.6 of the Technical Specifications.

Performed By: Z. P. [Signature]

Date: 2-2-79

Approved By: [Signature]

Date: 2/2/79