

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 I L Z I S 1 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 37 CAT 58

CON'T
01 REPORT SOURCE X 6 0 5 0 0 0 2 9 5 7 0 7 1 3 7 8 0 7 2 6 7 9 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
02 Westinghouse identified a potential problem involving inaccuracies in
03 steam generator level indication following a feedwater line break inside
04 the containment. The reactor trip and auxiliary feedwater initiation
05 could be delayed or prevented.
06
07
08

09 SYSTEM CODE I A 11 CAUSE CODE B 12 CAUSE SUBCODE A 13 COMPONENT CODE Z Z Z Z Z Z 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 14 15 16 17
17 LER/RO REPORT NUMBER 7 9 21 22 EVENT YEAR 23 SHUTDOWN METHOD 24 SEQUENTIAL REPORT NO. 0 5 0 26 OCCURRENCE CODE 27 0 1 28 REPORT TYPE 29 T 30 REVISION NO. 31 0 32
ACTION TAKEN X 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 22 0 0 0 0 40 ATTACHMENT SUBMITTED Y 23 NRRD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER W 1 2 0 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10 This potential error was not previously considered. Until this problem
11 can be thoroughly reviewed, the low-low steam Generator Reactor Trip
12 and Auxiliary Feedwater Initiation setpoint has been conservatively
13 raised to 15%.
14

15 FACILITY STATUS E 28 % POWER 1 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY D 31 DISCOVERY DESCRIPTION Westinghouse 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

16 ACTIVITY CONTENT Z 33 RELEASED OF RELEASE Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

17 PERSONNEL EXPOSURES NUMBER 0 9 37 TYPE Z 38 DESCRIPTION NA 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

18 PERSONNEL INJURIES NUMBER 0 40 DESCRIPTION NA 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

19 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

20 PUBLICITY ISSUED N 44 DESCRIPTION NA 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

NAME OF PREPARER Thomas M. Parker PHONE 312-746-2084

7908140 555 S
627312
NRC USE ONLY

ATTACHMENT TO LER
NO. 79-050/01 T-0
COMMONWEALTH EDISON CO.
ZION GENERATING STATION
50-295

On June 25, 1979 Westinghouse Electric Corporation notified Commonwealth Edison of a potential problem involving inaccuracies in steam generator level indication that could develop during a feedwater line break inside the containment. The energy injected into the containment atmosphere from the break increases the containment temperature. The increased containment temperature causes a 10% (maximum) bias prior to reactor trip or safety injection. This bias could delay or prevent the initiation of auxiliary feedwater and reactor trip. This bias had not been previously considered.

Westinghouse recommended raising the setpoint for Low-Low Steam Generator Level Reactor Trip and Auxiliary Feedwater initiation by 10%. Since the current setpoint is 10% above the setpoint used in the analysis, (FSAR 14.1.9) no immediate action was taken.

While continuing to investigate the potential problem, a setpoint change from 10% to 15% was made on Low-Low Steam Generator Reactor Trip and Auxiliary Feedwater setpoint. The safety analysis assumed a trip at 0%. The 5% inaccuracies plus the 10% Westinghouse recommended lead to a 15% setpoint. This is a very conservative response to the level bias problem. Further discussions within Commonwealth Edison and with Westinghouse will continue to resolve the exact effect of feedwater breaks inside the containment on reference leg heating.

This accident and its consequences will continue to be studied to determine if the setpoint needs to be permanently changed. A technical specification change will be submitted at that time if necessary.