

OPERATING DATA REPORT

DOCKET NO. 50-267
 DATE 790801
 COMPLETED BY J. W. Gahm
 TELEPHONE (303) 785-2253

OPERATING STATUS

1. Unit Name: Fort St. Vrain, Unit No. 1
2. Reporting Period: 790701 to 790731
3. Licensed Thermal Power (Mwt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330

NOTES

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

9. Power Level To Which Restricted, If Any (Net MWe): 231

10. Reasons for Restrictions, If Any: Nuclear Regulatory Commission restriction (70%) pending resolution of certain Final Safety Analysis Report and Technical Specification bases discrepancies. This unit is in startup phase following refueling and turbine generator overhaul.

| | This Month | Year to Date | Cumulative |
|---|---------------|----------------|------------------|
| 11. Hours in Reporting Period | <u>744</u> | <u>5,087</u> | <u>744</u> |
| 12. Number of Hours Reactor Was Critical | <u>696.9</u> | <u>1,843.3</u> | <u>15,317.0</u> |
| 13. Reactor Reserve Shutdown Hours | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 14. Hours Generator On-Line | <u>85.6</u> | <u>750.9</u> | <u>8,593.5</u> |
| 15. Unit Reserve Shutdown Hours | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 16. Gross Thermal Energy Generated (MWH) | <u>59,139</u> | <u>374,884</u> | <u>3,577,882</u> |
| 17. Gross Electrical Energy Generated (MWH) | <u>4,522</u> | <u>114,374</u> | <u>1,062,644</u> |
| 18. Net Electrical Energy Generated (MWH) | <u>0</u> | <u>101,177</u> | <u>952,628</u> |
| 19. Unit Service Factor | <u>11.5%</u> | <u>11.5%</u> | <u>11.5%</u> |
| 20. Unit Availability Factor | <u>11.5%</u> | <u>11.5%</u> | <u>11.5%</u> |
| 21. Unit Capacity Factor (Using MDC Net) | <u>0.0%</u> | <u>0.0%</u> | <u>0.0%</u> |
| 22. Unit Capacity Factor (Using DER Net) | <u>0.0%</u> | <u>0.0%</u> | <u>0.0%</u> |
| 23. Unit Forced Outage Rate | <u>88.5%</u> | <u>88.5%</u> | <u>88.5%</u> |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Maintenance Shutdown, October 20, 1979, 30 days

25. If Shut Down at End of Report Period, Estimated Date of Startup: 8-3-79

| Units In Test Status (Prior to Commercial Operation): | Forecast | Achieved |
|---|---------------|---------------|
| INITIAL CRITICALITY | <u>740201</u> | <u>740131</u> |
| INITIAL ELECTRICITY | <u>7612</u> | <u>761211</u> |
| COMMERCIAL OPERATION | <u>790701</u> | <u>790701</u> |

7908140504

626 294

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-267
 UNIT NAME Fort St. Vrain, Unit No. 1
 DATE 790801
 COMPLETED BY J. W. Gahm
 TELEPHONE (303) 785-2253

REPORT MONTH July, 1979

| NO. | DATE | TYPE | DURATION | REASON | METHOD OF SHUTTING DOWN REACTOR | LER # | SYSTEM CODE | COMPONENT CODE | CAUSE AND CORRECTIVE ACTION TO PREVENT REURREPCE |
|-------|--------|------|----------|--------|---------------------------------|--------------|-------------|----------------|---|
| 79-04 | 790201 | F | 542.3 | A | 4 | 79-03/03-L-0 | HH | PUMP XX | Excessive gland leakage on "B" boiler feed pump concurrent with inoperable "C" boiler feed pump necessitated controlled reactor shutdown on February 1, 1979, per LCO 4.3.2. Scheduled reactor shutdown for refueling was changed from March 1, 1979, to February 1, 1979. Thus shutdown number 79-04 has extended through February, March, April, May, and June, 1979. Upon completion of turbine generator overhaul and achievement of sufficient reactor power, first generation following shutdown 79-04 occurred on 7-23-79. |
| 79-05 | 790724 | F | 47.5 | H | N/A | N/A | N/A | N/A | Turbine generator taken off line due to field ground relay problems. Reactor was not shutdown. |
| 79-06 | 790726 | S | 1.9 | B | N/A | N/A | N/A | N/A | Turbine generator taken off line in order to perform overspeed tests. Reactor not shutdown. |
| 79-07 | 790728 | F | 53.0 | H | N/A | N/A | N/A | N/A | Turbine tripped from high vibration. Loop 2 shutdown occurred. Reactor power reduced to approximately 2% for Loop 2 recovery. |

626
295

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-267
 UNIT NAME Fort St. Vrain, Unit No. 1
 DATE 790801
 COMPLETED BY J. W. Gahm
 TELEPHONE (303) 785-2253

REPORT MONTH July, 1979 (continued)

| NO. | DATE | TYPE | DURATION | REASON | METHOD OF SHUTTING DOWN REACTOR | LER # | SYSTEM CODE | COMPONENT CODE | CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE |
|-------|--------|------|----------|--------|---------------------------------|-------|-------------|----------------|--|
| 79-08 | 790731 | F | 13.7 | H | 3 | N/A | N/A | N/A | While transferring to partial arc on the main turbine generator, throttle pressure dropped and load decreased 20 MW. During recovery, three circulators tripped, Loop 2 shutdown, reactor scrambled, and turbine tripped. Generator remained off line for balance of reporting period. |
| | | | | | | | | | |

SUMMARY: Plan to continue operation for electric production and fluctuation testing until release to continue power ascension testing above 70% received from Nuclear Regulatory Commission.

626 296

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267

Unit Fort St. Vrain, Unit No. 1

Date 790801

Completed By J. W. Gahm

Telephone (303) 785-2253

Month July, 1979

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net),

| | |
|----|---|
| 1 | 0 |
| 2 | 0 |
| 3 | 0 |
| 4 | 0 |
| 5 | 0 |
| 6 | 0 |
| 7 | 0 |
| 8 | 0 |
| 9 | 0 |
| 10 | 0 |
| 11 | 0 |
| 12 | 0 |
| 13 | 0 |
| 14 | 0 |
| 15 | 0 |
| 16 | 0 |

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

| | |
|----|-------|
| 17 | 0 |
| 18 | 0 |
| 19 | 0 |
| 20 | 0 |
| 21 | 0 |
| 22 | 0 |
| 23 | 60 |
| 24 | 0* |
| 25 | 0 |
| 26 | 661 |
| 27 | 1,095 |
| 28 | 292 |
| 29 | 0 |
| 30 | 385 |
| 31 | 563 |

*Generator on line but no net generation.

626 297

REFUELING INFORMATION

| | |
|--|---|
| 1. Name of Facility. | Fort St. Vrain, Unit No. 1 |
| 2. Scheduled date for next refueling shutdown. | September 1, 1980 |
| 3. Scheduled date for restart following refueling. | November 1, 1980 |
| 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? | No |
| If answer is yes, what, in general, will these be? | |
| If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10CFR Section 50.59)? | The Plant Operations Review Committee will review any questions associated with the core reload. |
| If no such review has taken place, when is it scheduled? | March 1, 1980 |
| 5. Scheduled date(s) for submitting proposed licensing action and supporting information. | ----- |
| 6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. | ----- |
| 7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. | a) 1482 HTGR fuel elements. b) 244 spent HTGR fuel elements. |
| 8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies. | Capacity is limited in size to about one third of core (approximately 500 HTGR elements). No change is planned. |

REFUELING INFORMATION (CONTINUED)

| | |
|---|--|
| 9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. | 1/86 under the Three Party Agreement (Contract AT (04-3)-633) between DOE, Public Service Company of Colorado (PSCo), and General Atomic Company.* |
|---|--|

*The 1986 date is based on the understanding that spent fuel discharged during the term of the Three Party Agreement will be shipped to the Idaho National Engineering Laboratory for storage by DOE at the Idaho Chemical Processing Plant (ICPP). The storage capacity has evidently been sized to accommodate fuel which is expected to be discharged during the eight year period covered by the Three Party Agreement.

626 299