AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-312
UNIT Rancho Seco Unit 1
DATE 79-06-30
COMPLETED BY R. W. Colombo
TELEPHONE (916) 452-3211

MONTH July

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	861
2	0	18	862
3	0	19	859
4	0	20	727
5	88	21	0
6	532	22	0
7	819	23	467
8	866	24	852
9	867	25	860
10	867	26	866
11	867	27	868
12	620	28	865
13	613	29	869
14	715	30	856
15	857	31	856
16	860		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO. 50-312
DATE 79-07-31
COMPLETED BY R. W. Colombo
TELEPHONE 916-452-3211

OPERATING STATUS

1. Unit Name: Rancho Seco Unit
2. Reporting Period: July 1979
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes
Shut down July 20 for approximately 54 hours to modify several pipe supports to the criteria in I&E Bulletin 79-02.

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5087	37584
12. Number Of Hours Reactor Was Critical	601.9	3399.4	22470.2
13. Reactor Reserve Shutdown Hours	0	1519.2	3900.6
14. Hours Generator On-Line	573.8	3277.5	21440.7
15. Unit Reserve Shutdown Hours	13.8	1199.3	1210.2
16. Gross Thermal Energy Generated (MWH)	1,546,232	8,961,925	54,035,081
17. Gross Electrical Energy Generated (MWH)	480,799	2,932,120	18,194,920
18. Net Electrical Energy Generated (MWH)	450,527	2,780,155	17,179,045
19. Unit Service Factor	77.1	64.4	57.0
20. Unit Availability Factor	79.0	88.0	60.3
21. Unit Capacity Factor (Using MDC Net)	69.4	62.6	52.4
22. Unit Capacity Factor (Using DER Net)	66.0	59.5	49.8
23. Unit Forced Outage Rate	21.4	7.1	35.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-312
UNIT NAME Rancho Seco Unit 1
DATE 79-07-31
COMPLETED BY R. W. Colombo
TELEPHONE (916) 452-3211

REPORT MONTH: July

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	79-07-01	S	13.8	F	1	N/A	N/A	N/A	Continuation of last month's "Prerequisite testing of AFW and plant heatup in preparation for startup."
8	79-07-01	F	97.6	B	3	N/A	N/A	N/A	Rx trip due to high pressure resulting from STP-070 (Aux. Feedwater Flow Test). Repaired weld on Aux. System Line and performed other maintenance.
9	79-07-12	F	5.0	A	3	N/A	N/A	N/A	Rx trip due to pressure transmitter malfunction which was equated to turbine overspeed.
10	79-07-20	F	53.8	F	1	79-007/01-T-0	CF	SUPORT	Pipe supports having safety factor of <2 according to I.E. Bulletin 79-02 analysis.

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1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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NARRATIVE SUMMARY OF PLANT OPERATIONS

DATE	
7/1/79	
(0000)	Reactor at 15% Full Power
(0855)	Lowered reactor to 13% power
(1348)	Reactor tripped (high pressure)
7/4/79	
(1030)	Reactor in heatup mode
7/5/79	
(0730)	Reactor critical
(0750)	Reactor at 2.5% power
(1239)	Completed STP-070, "Aux. Feedwater Flow Test"
(1500)	Reactor at 15% FP
(1525)	Generator on line (closed OCB's)
(2340)	Reactor at 50% power increasing to 70% (420 MWe)
7/6/79	
(1126)	Reactor at 70% power, commenced 5-hour hold
(1600)	Increasing to 92% power
7/7/79	
(1738)	Started increasing to 100% power
(1946)	Unit at 100%
7/8/79	
(0530)	Completed Auxiliary Feedwater Pump P-319 portion of STP-071, Auxiliary Feedwater Endurance Test
7/10/79	
(1235)	Completed Auxiliary Feedwater Pump P-318 portion of STP-071, Auxiliary Feedwater Endurance Test
7/12/79	
(1717)	Reactor trip on turbine-generator trip
(1909)	Reactor critical
(2216)	Generator on line (Closed OCB's)

7/13/79	
(2400)	Reactor at 32% power
(0115)	Started increasing reactor to 50% power
(0255)	Reactor at 50%
(0712)	Increasing reactor to 92% power
(1230)	Reactor at 90%, commenced 2-hour hold
(1510)	Started increasing to 100% power
(2115)	Reactor at 100% power
7/14/79	
(1740)	Overspeed Protection Control failed. All intercept valves closed. Lowered reactor power to 30%. Took manual control of turbine
(2205)	Increasing to 92% power
7/15/79	
(0000)	Reactor at 87% commenced 2-hour hold
(0159)	Reactor at 100% power
7/20/79	
(1615)	Started reactor power decrease at 1 MW/minute
(2210)	Transferred aux. trans. loads to SU transformers
(2245)	Opened OCB's 220/230
(2247)	Pushed Turbine trip button
(2320)	Reactor in HSD
7/23/79	
(0117)	Reactor critical
(0431)	Generator on line (closed OCB's)
(0542)	Reactor at 50% power
(0650)	Started increasing to 72% power
(1510)	Started increasing to 92% power

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7/23/79 (Continued)

(1805) Reactor at 87% power commenced 2-hour hold

(2107) Reactor at 100% power

PERSONNEL CHANGES REQUIRING REPORTING

No personnel changes that require reporting in accordance with Technical Specifications Figure 6.9-2 were made in July, 1979.

MAJOR ITEMS OF SAFETY-RELATED MAINTENANCE

1) Conducted inspection test of all pipe support anchors in Class 1 systems. Made modifications to those anchors that did not meet the new criteria covered in Bulletin 79-02. Reference: LER 79-07.

2) Replaced "B" Decay Heat Pump (P-261B) outboard seal. Reference: LER 79-08.

SUMMARY OF CHANGES MADE IN ACCORDANCE WITH 10 CFR 50.59(b)

No changes, tests, or experiments were completed in July, 1979, which constituted a change in a safety analysis report description.

REFUELING INFORMATION REQUEST

1. Name of Facility: Rancho Seco Unit 1

2. Scheduled date for next refueling shutdown: February 1980

3. Scheduled date for restart following refueling: April 1980

4. Technical Specification change or other license amendment required:

   a) Change to Rod Index vs. Power Level Curve (TS 3.5.2)

   b) Change to Core Imbalance vs. Power Level Curve (TS 3.5.2)

   c) Tilt Limits (TS 3.5.2)

   d) Safety Equipment Testing (TS 3.3.3)

5. Scheduled date(s) for submitting proposed licensing action: December 1979

6. Important licensing considerations associated with refueling: None

7. Number of fuel assemblies:

   a) In the core: 177

   b) In the Spent Fuel Pool: 112*

8. Present licensed spent fuel capacity: 579

9. Projected date of the last refueling that can be discharged to the Spent Fuel Pool: 1987

*Previous reports of 122 were in error.