



SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, Box 15830, Sacramento, California 95813; (916) 452-3211

August 2, 1979

U. S. Nuclear Regulatory Commission  
ATTN: Mr. R. H. Engelken, Director  
Region V Office of Inspection & Enforcement  
1990 North California Boulevard  
Walnut Creek Plaza, Suite 202  
Walnut Creek, California 94596

Re: Operating License DPR-54  
Docket No. 50-312  
Reportable Occurrence No. 79-7

Dear Mr. Engelken:

In accordance with Technical Specifications for Rancho Seco Nuclear Generating Station 6.9.4.1b, and Regulatory Guide 1.16, Revision 4 Section C.2.a2, the Sacramento Municipal Utility District is hereby submitting a fourteen-day followup report to Reportable Occurrence 79-7, which was initially reported to your office July 20, 1979, via telephone and a confirmation letter July 23, 1979.

On July 20, 1979, information from analysis of the "Pipe Support Base Plate Design Using Concrete Expansion Anchor Bolts" in accordance with I.F. Bulletin 79-02 indicated that six safety-related anchors at Rancho Seco did not meet the Bulletin Criteria No. 2.

The six anchors of concern were:

	<u>Number I.D.</u>	<u>System</u>
1)	4U-26020-4	"A" Decay Heat
2)	4U-26021-10	"B" Decay Heat
3)	3A-48401-6	"B" Nuclear Service Raw Water
4)	4A-26120-1	"A" Decay Heat
5)	4U-26021-1	"B" Decay Heat
6)	5A-47361-1	"B" Nuclear Service Raw Water

Anchors on both "A" and "B" Decay heat Systems that did not meet the requirements of Bulletin 79-02 determined the Decay Heat System to be below the Limiting Condition for Operation and a reactor shutdown was commenced at 1615 on July 20. The generator was removed from service at 2245 and the reactor was tripped at 2247 the same day.

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Immediate actions were initiated to modify the specified hangers and, by working continuous rotating shifts, Rancho Seco strengthened the anchors to the standards required by the R. H. Engelken letter of July 20. The reactor was returned to operation and achieved full power on July 23 at 2107.

The transient related to this event required a reactor shutdown for 50.5 hours and 54 hours of lost power generation.

Respectfully submitted,



J. J. Mattimoe  
Assistant General Manager  
and Chief Engineer

JJM:RWC:jim

cc: Director, MIPC (3)  
Director, I&E (30)

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