

ATTACHMENT TO LER

Northeast Nuclear Energy Company
Millstone Nuclear Power Station - Unit 1
Provisional License Number DPR-21
Docket Number 50-245

Identification of Occurrence

Failure of ten primary containment isolation valves, to satisfy the Technical Specification limit for local leak rate testing.

Conditions Prior to Occurrence

The unit was shutdown on April 28, 1979 for a planned refueling outage.

Description of Occurrence

During the refueling outage, all testable primary containment isolation valves, cable penetrations and manways were local leak rate tested. Of the 83 isolation valves, penetrations and manways tested, a total of 10 were found to have leakage in excess of Technical Specification limits.

The Technical Specifications require that the combined total leak rate for all testable penetrations and isolation valves be less than 319.2 SCFH at 43 PSIG. In addition, no one penetration or isolation valve, except the main steam line isolation valves, may exceed a leak rate of 19.95 SCFH at 43 PSIG. No one main steam line isolation valve may exceed a leak rate of 11.5 SCFH at 25 PSIG.

The total "As Found" leak rate, assuming a worst single failure analysis, was on the order of 420 SCFH. Following repairs of the leaking valves, the total "As Left" leak rate was 108.65 SCFH at 43 PSIG.

The following is a list of the valves which initially failed to pass the leak rate test, their "As Found" and "As Left" leakages.

<u>Valve/Penetration</u>	<u>As Found</u>	<u>As Left</u>
A. Feedwater Check Valves		
1-FW-9A	> 50 SCFH	2.90 SCFH
1-FW-10A	> 50 SCFH	6.30 SCFH
B. Shutdown Cooling Isolation Valves		
1-SD-4A, B	80 SCFH	3.34 SCFH
1-SD-2B	33 SCFH	3.34 SCFH

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790 8100 452

C. Atmospheric Control Valves

1-AC-7, 9, 11 & 12	-	40 SCFH	2.55 SCFH
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D. Cleanup System Isolation Valves

1-CU-2	-	73 SCFH	2.71 SCFH
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Designation of the Apparent Cause of the Occurrence

A. Feedwater Check Valves

1-FW-9A	-	Worn seat and seal ring.
1-FW-10A	-	Worn seating surface.

B. Shutdown Cooling Isolation Valves

1-SD-4A, B	-	Worn seat rings and disc.
1-SD-2B	-	Worn seat ring.

C. Atmospheric Control Valves

1-AC-7, 9, 11 & 12	-	Dirt in air actuators.
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D. Cleanup System

1-CU-2	-	Worn disc and seat.
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Analysis of the Occurrence

A. Feedwater Check Valves

Although two feedwater check valves failed to pass the local leak rate, this is not considered to be significant since accident conditions would tend to seat these valves more firmly than test conditions, also the Feedwater System is part of the ECCS System and would normally be operating for small pipe breaks.

B. Shutdown Cooling Isolation Valves

The redundant isolation valve for 1-SD-4A & B was available and exhibited leakage within allowable limits. The redundant isolation valve for 1-SD-2B was also available and exhibited leakage within allowable limits.

C. Atmospheric Control Valves

The redundant isolation valves for 1-AC-7, 9, 11 & 12 were available and exhibited leakage within allowable limits.

D. Cleanup System

The redundant isolation valves for 1-CU-2 were available and exhibited leakage within allowable limits.

Corrective Action

A. Feedwater Check Valves

1-FW-9A - Replaced seat and seal ring.
Valve retested satisfactorily.

1-FW-10A - Stoned seating surface &
cleaned valve, retested
satisfactorily.

B. Shutdown Cooling Isolation Valves

1-SD-4A, B - Machined worn seat rings
and discs. Valves retested
satisfactorily.

1-SD-2B - Replaced worn seal ring.
Valve retested satisfactorily.

C. Atmospheric Control Valves

1-AC-7, 9, 11 & 12 - Rebuilt air actuators.
Valves retested satisfactorily.

D. Cleanup System

1-CU-2 - Lapped seat and replaced
disc. Valve retested
satisfactorily.

There are no recommendations for modification or replacement of any of the aforementioned components pending NRC review of Millstone's 10 CFR 50 App. J submittal.

Failure Data

A. Feedwater Check Valves

Manufacturer: Crane-Chapman Valve Company

Figure 1575 WE

18 inch check valve

Forged carbon steel

B. Shutdown Cooling Isolation Valves

1-SD-4A, B

Manufacturer: Crane-Chapman Valve Company

Figure L603

10 inch globe valve

Forged stainless steel

1-SD-2B

Manufacturer: Crane-Chapman Valve Company

Figure 76-1/2 U

12 inch gate valve

Forged stainless steel

C. Atmospheric Control Valves

Manufacturer: Allis-Chalmers

Model: 150R

18 inch butterfly valve

Cast carbon steel

D. Cleanup System Isolation Valve

Manufacturer: Crane-Chapman Valve Company

Figure 83-1/2 U

8 inch gate valve

Forged stainless steel

598350

LICENSEE EVENT REPORT

CONTROL BLOCK: ①

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 C T M N S 1 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 4 0 0 1 5
* R 9 LICENEE CODE 14 75 LICENSE NUMBER 25 26 LICENSE TYPE 37 37 CAT 57

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REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 1 | 2 | 4 | 5 | 7 | 0 | 6 | 2 | 8 | 7 | 9 | 8 | 0 | 8 | 1 | 3 | 1 | 2 | 9 | 0 |

60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | While performing local leak rate tests two feedwater system check valves, one
0 3 | inboard and one outboard (1-FW-9A, 10A) were found to be leaking in excess of the
0 4 | Technical Specification limit. See attached report.

0 6

O B

CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE
S | D | 11 | E | 12 | B | 13 | V | A | L | V | E | X | 14 | L | 15 | D | 16

8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45
LER RO REPORT NUMBER		EVENT YEAR 7 9 21 22		SEQUENTIAL REPORT NO. 0 1 9 23 24 25			OCCURRENCE CODE 0 3 26 27 28 29			REPORT TYPE L 30 31			REVISION NO. 0 32																								
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS 0 0 0 0 37 38 39 40			ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER C 2 5 6 44 45																											
B 33	Z 34	Z 35	Z 36				Y 41	Y 42	A 43																												
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (22)																																					

Crane-Chapman 18 inch forged carbon steel check valve figure number 1575. The leakage was attributable to normal deterioration of seating surfaces and seal ring wear. Valves were reworked and retested satisfactorily.

15 H 28 0 0 0 29 NA B 31 Local leak rate testing

7 8 9 10 11 12 13 44 45 46 80
ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY 35
1 6 Z 33 Z 34 NA LOCATION OF RELEASE 36
7 8 9 10 11 44 45 46 80

7 8 9 11 12 13 PERSONNEL INJURIES
NUMBER DESCRIPTION (41)
 NA

1 5 0 0 0 49 NA 80
7 8 9 10 11 12
LOSS OF OR DAMAGE TO FACILITY 49

1 3 TYPE Z 42 DESCRIPTION NA 79Ø8LØØ4S1 82

PUBLICITY
ISSUED 7/44 NA 528251 NRC USE ONLY

P. J. Przekop 63203-447-1791

LICENSEE EVENT REPORT

CONTROL BLOCK:

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1	C	I	T	M	I	N	S	1	2	0	0	+	0	0	0	0	10	1	-	0	0	3	4	1	1	1	1	1	4	0	1	1	5
2	LICENSEE CODE	14	15	LICENSE NUMBER	25	26	LICENSE TYPE	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55

DON'T	REPORT SOURCE	60	61	DOCKET NUMBER	65	66	EVENT DATE	74	75	REPORT DATE	80	81	82	83	84	85	86	87	88	89	90	91	92	93	94	95	96	97	98	99	100
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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

While performing local leak rate tests one cleanup system inboard isolation valve

(1-CU-2) was found to have leakage in excess of the Technical Specification limit.

The redundant isolation valves exhibited leakage within the desired limit. See attached report.

8	9	SYSTEM CODE	10	CAUSE CODE	11	CAUSE SUBCODE	12	COMPONENT CODE	13	COMP. SUBCODE	14	VALVE SUBCODE	15	16
17	18	S	D	E	F	B	L	A	V	L	V	F	I	X

17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46			
LER RO	REPORT NUMBER	EVENT YEAR	7	9			0	1	9		0	1	3			0	1	0	1	0	1	0	1	0	1	0	1	0	1	0	1	
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	B	Z	Z	Z	0	0	0	0	Y	Y	Y	Y	Y	A	A	C	C	C	C	C	C	C	C	C	C	C	C	C	C	C
SHUTDOWN METHOD	HOURS	Z	Z	Z	Z	Z	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

Crane-Chapman 8 inch forged stainless steel gate valve figure number 83 - 1/2 U.

Leakage was attributable to worn seating surfaces. Valve was reworked and retested satisfactorily.

8	9	FACILITY STATUS	10	% POWER	11	OTHER STATUS	12	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62			
15	16	H	0	0	0	NA		B	31	Local	leak	rate	testing																	

8	9	ACTIVITY CONTENT	10	RELEASED OF RELEASE	11	AMOUNT OF ACTIVITY	12	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65			
16	17	Z	Z	Z	Z	NA																										

8	9	PERSONNEL EXPOSURES	10	NUMBER	11	TYPE	12	DESCRIPTION	13	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64			
17	18	0	0	0	0	Z	Z	NA																									

8	9	PERSONNEL INJURIES	10	NUMBER	11	DESCRIPTION	12	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66			
17	18	0	0	0	0	NA																											

8	9	LOSS OF OR DAMAGE TO FACILITY	10	TYPE	11	DESCRIPTION	12	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66			
17	18	Z	Z	Z	Z	NA																											

8	9	PUBLICITY	10	ISSUED	11	DESCRIPTION	12	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66			
17	18	Z	Z	Z	Z	NA																											

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NRC USE ONLY

203-447-1791

NAME OF PREPARED P. J. Przekop

PHONE:

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