



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NRC FOR

June 19, 1979

MEMORANDUM FOR: Chairman Hendrie  
Commissioner Gilinsky  
Commissioner Kennedy  
Commissioner Bradford  
Commissioner Ahearne

FROM: Harold R. Denton, Director  
Office of Nuclear Reactor Regulation

THRU: Lee V. Gossick  
Executive Director for Operations *[Signature]*

SUBJECT: REMOVAL OF SUSPENSION OF OPERATION OF RANCHO SECO

The Staff has completed its review of the immediate actions performed by Sacramento Municipal Utility District in response to the Commission's Order of May 7, 1979. I am satisfied that licensee has satisfactorily completed the prescribed actions, provided acceptable analyses, and specified appropriate implementing procedures. Accordingly, in accordance with the terms of the Commission's Order, I propose to inform the licensee that the suspension of operations of Rancho Seco previously imposed is no longer required. I will, however, not take this action pending the Commission's determination of the hearing requests of (1) Friends of the Earth, et al., and (2) Richard D. Castro and Gary Hursh, which are presently before you. Assuming the Commission does not require a hearing prior to the lifting of the suspension and does not raise any objection to my proposed action, it will then be promptly implemented.

Enclosed for Commission consideration are copies of the Staff Safety Evaluation and the proposed letter to licensee. Copies of this memorandum and enclosures are also being provided to petitioners Friends of the Earth, et al., and Messrs. Castro and Hursh, and to the licensee and the California Energy Commission.

*[Signature]*

Harold R. Denton, Director  
Office of Nuclear Reactor Regulation

Enclosures:  
As stated

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket No. 50-312

Mr. J. J. Mattimoe  
Assistant General Manager and  
Chief Engineer  
Sacramento Municipal Utility District.  
6201 S Street  
P. O. Box 15830  
Sacramento, California 95813

Dear Mr. Mattimoe:

By Order of May 7, 1979, the Commission confirmed your undertaking a series of actions, both immediate and long term, to increase the capability and reliability of the Rancho Seco Nuclear Generating Station to respond to various transient events. In addition, the Order confirmed that you would shut down Rancho Seco on April 28, 1979, and maintain the plant in a shut-down condition until the following actions had been satisfactorily completed:

- (a) Upgrade the timeliness and reliability of delivery from the Auxiliary Feedwater System by carrying out actions as identified in Enclosure 1 of your letter of April 27, 1979.
- (b) Develop and implement operating procedures for initiating and controlling auxiliary feedwater independent of Integrated Control System control.
- (c) Implement a hard-wired control-grade reactor trip that would be actuated on loss of main feedwater and/or turbine trip.
- (d) Complete analyses for potential small breaks and develop and implement operating instructions to define operator action.
- (e) Provide for one Senior Licensed Operator assigned to the control room who has had Three Mile Island Unit No. 2 (TMI-2) training on the B&W simulator.

By submittal of May 14, 1979, as supplemented by seven letters dated May 22, 24, 29, 30(3) and June 6, 1979, you have documented the actions taken in response to the May 7 Order. We have reviewed this submittal, and are satisfied that, with respect to Rancho Seco, you have satisfactorily completed the actions

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prescribed in items (a) through (e) of paragraph (1) of Section IV of the Order, the specified analyses are acceptable, and the specified implementing procedures are appropriate. The bases for these conclusions are set forth in the enclosed Safety Evaluation.

As noted on page 13 of the Safety Evaluation, you will be required to conduct a test during power operation to demonstrate operator capability to assume manual control of the Auxiliary Feedwater System independent of the Integrated Control System.

Appropriate Technical Specifications for Limiting Conditions for Operation and for surveillance requirements should be developed as soon as practicable and provided to the staff within seven days with regard to the design and procedural changes which have been completed in compliance with the provisions of the May 7, 1979 Commission Order. The revised Technical Specifications should cover:

- (1) Addition of flow indication to the Auxiliary Feedwater System;
- (2) Addition of the Anticipatory Reactor Trips; and
- (3) Changes in set points for high pressure reactor trip and PORV actuation.

Within 30 days of receipt of this letter, you should provide us with your schedule for completion of the long term modifications described in Section II of the May 7 Order, and you should submit for staff review the model used in the analysis for potential small breaks referenced in your letter of May 14, 1979.

My finding of satisfactory compliance with the requirements of items (a) through (e) of paragraph (1) of Section IV of the Order will permit resumption of operation in accordance with the terms of the Commission's Order; it in no way affects your duty to continue in effect all of the above provisions of the Order pending your submission and approval by the Commission of the Technical Specification changes necessary for each of the required modifications.

Sincerely,

Harold R. Denton, Director  
Office of Nuclear Reactor  
Regulation

Enclosures:

1. Safety Evaluation
2. Notice

cc w/enclosures: See next page

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Sacramento Municipal Utility  
District

cc w/enclosure(s):  
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cc w/enclosure(s) and incoming  
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