NEDO-21052-A 75NED53 CLASS I MAY 1979

# MAXIMUM DISCHARGE RATE OF LIQUID-VAPOR MIXTURES FROM VESSELS

F. J. MOODY



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NEDO-21052-A 75NED53 Class I May 1979

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## PREFACE

This approved version of report NEDO-21052 (September 1975) is being provided in accordance with the NRC's topical report program. This report NEDO-21052-A consists of the original text of NEDO-21052, the NRC staff letter accepting this report as a reference, the NRC staff Topical Report Evaluation and the supplementary information General Electric provided to the NRC staff during the review of NEDO-21052.

NEDO-21052-A

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555 RECEIVED

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L.J. SCRON

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General Electric Company ATTN: Mr. L. J. Sobon, Manager BWR Containment Licensing MC905 175 Curtner Avenue San Jose, CA 95125

Gentlemen:

SUBJECT: REVIEW OF GENERAL ELECTRIC TOPICAL REPORT NEDO-21052, "MAXIMUM DISCHARGE RATE OF LIQUID-VAPOR MIXTURES FROM VESSELS"

We have completed our review of the General Electric Topical Report NEDO-21052, "Maximum Discharge Rate of Liquid-Vapor Mixtures from Vessels," as it is to be applied to determine the mass and energy release rate resulting from a design basis accident for Mark I containment response analyses. Based on our review, we conclude that the model described in NEDO-21052, in conjunction with its method of application for Mark I containment response analyses, are acceptable for reference as specified in the enclosure.

During the course of our review, we determined that additional justification would be necessary to support application of this model to break sizes and types other than the double-ended rupture of a recirculation line in a Mark I plant. We understand that you wish to pursue the application of this model for other sizes and types of breaks. Therefore, when you provide the additional information required, as discussed in our letter dated January 30, 1978, the staff will continue its review of the subject topical report. Such information should be submitted to the Division of Project Management.

The staff does not intend to repeat its review of this topical report when it appears as a reference for a Mark I containment response analysis, except to assure that the model is applicable to the specific plants involved. Should the regulatory criteria or regulations change such that our conclusions concerning this topical report become invalid, you will be notified and will be given the opportunity to revise and resubmit your topical report for review, should you so desire. In accordance with established procedure, we request that General Electric issue a revised version of the topical report to include any supplementary information provided for our review, this acceptance letter, and the NRC staff evaluation.

Sincerely, D. Eisenhut Acting Assistan

Director for Systems and Projects, Division of Operating Reactors

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Enclosure: Topical Report Evaluation

cc: Mr. L. Gifford General Electric Company 4720 Montgomery Lane Bethesda, MD 20014 MAXIMUM DISCHARGE RATE OF LIQUID-VAPOR MIXTURES FROM VESSELS

F.J. Moody

#### ABSTRACT

A discrepancy exists in theoretical predictions of the two-phase equilibrium discharge rate from pipes attached to vessels. Theory which predicts critical flow data in terms of rips exit pressure and quality severely overpredicts flow rates in terms of vessel fluid properties. This study shows that the discrepancy is explained by the flow pattern. Due to decompression and flashing as fluid accelerates into the pipe entrance, the maximum discharge rate from a vessel is limited by choking of a homogeneous bubbly mixture. The mixture tends towar a slip flow pattern as it travels through the pipe, finally reaching a different choked condition at the pipe exit.

NOMENCLATURE

		_	

fL/D Pipe Friction Parameter

- G Mass Flux
- g\_ Newton's Constant
- h Enthalpy
- K Slip Ratio

 $\hat{m}_{\mbox{ij}}$  / Vaporization or Condensation Rates  $\hat{m}_{\mbox{ij}}$ 

- p Pressure
- s Entropy
- T Temperature
- u Speed
- V Speed of Disturbance Propagation
- v Specific Volume
- v\* Specific Volume, Defined by Equation (18)
- v\_ Specific Volume, Defined by Equation (24)
- ve Specific Volume, Defined by Equation (25)
- x Flowing Quality
- Z Function Defined by Equation (42)
- a Void Fraction
- 8 Entropy Production Function, Defined by Equation (39)

#### Subscripts

- c Critical Flow Rate
- f Saturated Liquid
- g Saturated Vapor

fg Vaporization o Stagnation REF Reference Value

Superscript Derivative with Respect to Pressure

```
Other Notation
      inch(es)
in.
      centimeter(s)
cm.
      meter(s)
10
     square
Da
     Pound(s) Force
1bf
      Pound(s) Mass
1bm
kN
      Kilonewton(s)
     Kilogram(s)
kem
      British thermal unit(s)
Btu
```

#### INTRODUCTION

When high pressure fluid near the saturation state is discharged to a low pressure receiver, the mass flow rate will be limited to a maximum value which depends on conditions in the vessel. The discharge rate largely determines vessel decompression rate, receiver pressurization rate, vessel thrust reaction, and impingement forces on nearby objects. Prediction of the critical flow rate plays a major role in determining core cooling requirements associated with safety analyses of nuclear power plants.

Experimental programs conducted by numerous workers have provided a broad range of two-phase critical flow data (e.g., References 1 through 7). Geometries tested include orifices, nozzles, tubes, and pipes of various lengths and sizes. Liquidvapor critical flow data fall into either equilibrium or non-equilibrium classifications; subdivisions include geometry and the flow pattern. The non-equilibrium classification has been studied in part by various workers including Henry (8, 9) and Edwards (10). However, Boure' (11) suggested that further progress in two-phase flow depends on additional study of phenomena which govern interfacial forces and transfer rates of heat and mass. Idealized equilibrium flows are less complex because they do not require theoretical models for interfacial heat and mass transfer.

A discrepancy associated with equilibrium flows is addressed in this study. It has been shown by Fauske ( $\frac{4}{2}$ ) and Henry ( $\frac{6}{2}$ ) that critical flow data is seriously underpredicted by the homogeneous model in terms of pipe exit static pressure and quality. On the other hand, slip flow models, including those by Fauske ( $\frac{4}{2}$ ), Levy ( $\frac{12}{2}$ ), and Moody ( $\frac{13}{2}$ ) reasonably pre-

dict much of the pipe exit data, in spite of the fact that they satisfy either momentum or energy conservation, but not both, and hence do not completely describe two-phase critical flow. In serming contradiction, others, including Allemann (14) and Moody (15), showed that slip flow models greatly overpredict two-phase blowdown rates in terms of vessel stagnation properties. This represents a fundamental discrepancy in theoretical prediction of two-phase critical flow. Whether slip models are inadequate for relating critical flow and stagnation properties, or whether the discrepancy is caused by other model difficulties needs to be understood. It is the purpose of this study to explain the discrepancy and provide a basis for accurately predicting the maximum discharge rate of equilibriu: liquid-vapor mixtures from vessels.

#### EQUILIBRIUM TWO-PHASE FLOWS

Since this study is for equilibrium flows, it is necessary to identify physical constraints which determine saturated equilibrium for a two-phase critical flow. When liquid at equilibrium is decompressed below its saturation pressure, vapor is formed at a finite rate un.il a new equilibrium state is reached. S. eral studies have been made which help determine the suration of non-equilibrium states. Edwards and O'Brien (16) conducted experiments in which a 3.96 m (13 ft) long, 7.32 cm (2.88 in.) i.d. pipe was ruptured at one end after pressurization to 70 atmospheres. High-speed pressure recordings along the pipe showed that pressure dropped below the saturation value and rose again within 0.5 millisecond. Zaker and Wiedermann (17) employed a 5.08 cm (2 in.) i.d., 1.93 m (6 ft) long tube and found that nonequilibrium states lasted for less than 1 millisecond. The same duration of non-equilibrium states was found by Gallagher (18) during decompression experiments on 5.08 cm (2 in.) i.d. hot water driver tubes of 1.83, 3.66, and 5.18 m (6, 12, and 17 ft) length, pressurized up to 140 atmospheres and temperatures up to saturation. These water depressuri-zation studies indicate that non-equilibrium states survive for a millisecond or less.

A fluid particle which is accelerated from stagnation in a vessel into a pipe will undergo rapid decompression. Based on the decompression studies men-:ioned, it is reasonable to expect that when moving in a decreasing pressure field, a fluid particle lags behind equilibrium about 1 millisecond in time. Once inside a uniform pipe, fluid acceleration and associated pressure reduction decrease, permitting equilibrium states to be closely approached if the pipe is sufficiently long. For example, the maximum water speed achievable from a system at 70 atmospheres is about 120 mps (400 fps). Therefore, the farthest a water particle can travel in 1 millisecond is about 12 cm (0.4 ft). It follows that if the flow path is less than 12 cm, non-equilibrium states are expected whereas in longer flow passages, equilibrium statare expected.

Fauske (19), Uchida (20), Simon (21), and Sutherland (7) have presented water bldata for pipes of various lengths with dia er to 1.27 cm (0.5 in.). When the data for fix\_' conditions are plotted as a function of pipe ler\_ac\_, a characteristic knee is observed near a length of 10 cm (4 in.). Pipes approaching zero length tend to have non-flashing orifice discharge rates. As pipe length is increased, the discharge rate decreases sharply. For pipe lengths greater chan about 10 cm, the discharge rate is unaffected by further increase in pipe length. This observation is consistent with the time required for a steam-water system to approach saturated equilibrium. Data of Sozzi and Sutherland (7) for 1.27 cm (0.5 in.) i.d. pipes shows that even subcooled water in the vessel requires no more than 12.7 cm (5 in.) to approach equilibrium.

This study applies only to flow passages long enough to assure that equilibrium is approached. For hot water, that length is less than 12.7 cm (5 in.).

#### THE FLOW PATTERN

Since equations which govern two-phase flow differ in modeling relative motion between vapor and liquid, theoretical values of the critical flow rate strongly depend on the flow pattern. Various correlations have been proposed for predicting flow pattern in steady two-phase flow, some of which are summarized by Lahey (22) and Wallis (23). However, flow pattern correlations are not available for steep pressure gradients or unsteady decompression. When a body of liquid is decompressed below its saturation pressure, homogeneous boiling occurs. For example, photographs by Kober (24) show the formation of thoroughly mixed bubbles in liquid during the depressurization of hot water in glass vessels, initially pressurized to about 2 atmospheres. Furthermore, at high pressure, both Edwards (16) and Gallagher (18) found that transient water depressurization in long pipes was predicted by models based on homogeneous bubbles in liquid. Therefore, it is expected that when a particle of liquid is accelerated into the entrance of a discharge pipe, decompression below its saturation pressure will cause homogeneous boiling. If the flow path into a discharge pipe is short so that vapor formation is not complete, continued homogeneous boiling is expected until equilibrium is approached. Even though discrete bubbles move relative to liquid during acceleration, the homogeneous model is expected to describe pipe entrance conditions during blowdown. Moreover, homogeneous choked flow near the entrance may better represent blowdown data in terms of stagnation properties.

Inside the pipe, vapor formation rate is reduced, and transition to other flow patterns can occur. For steam-water critical flow in straight pipes, Fauske ( $\frac{4}{2}$ ) determined from steady-state correlations that the annular flow pattern was most likely. Flow pattern studies by Henry ( $\frac{25}{25}$ ) near the discharge of a straight pipe showed that the slip ratios were less than required for annular flow, but definitely indicated a slip flow. Even though the blowdown flow may be limited by homogeneous choking at the pipe entrance, transition to a slip flow could produce a second choked condition near the pipe exit. These two possible choked conditions are considered next.

#### WKED FLOW

Critical flow occurs when further reduction in rec\_\_ver pressure does not change the mass discharge tate from a flow passage which is attached to a vessel. Mathematically, this condition corresponds to

$$\frac{dG}{dp} = 0$$
(1)

The sonic state of a fluid occurs when further reduction in receiver pressure does not propagate into the flow. If V is the propagation speed of a dist () turbance sainst the direction of flow, conditions for the scale state are

$$V = \frac{dV}{dp} = 0$$
 (2)

When equations (1) and (2) are satisfied simultaneously, the flow is choked. Equation (2) also characterizes steady flow conditions when no disturbances are present.

#### THE HOMOGENEOUS MODEL

Although homogeneous theory is well established, several important features of choked flow are summarized here. The one-dimensional conservation rquations written for a small disturbance moving at absoluto speed, V, against a homogeneous, frictionless, adiabatic flow are,

Mass: 
$$d(GA) + d(AV/v) = 0$$
 (3)

Momentum: 
$$d(G^2Av) + d(AV/v)$$
  
+ g Adp = 0 (4)

Energy:  $d(GAh_o) + d(AVh_o/v)$ - AVdp = 0

Employing the definition of stagnation enthalpy,

$$h_0 = h + G^2 v^2 / 2g_c$$
 (6)

equations (3), (4), and (5) are combined with the Gibbs equation to show that

$$Tds = dh - vdp = 0.$$
(7)

It follows that state changes occur isentropically across the disturbance. For steady flow it is similarly shown that state changes are isentropic.

If the sonic conditions of equation (2) exist somewhere in the flow, and steady flow occurs upstream, equations (3) and either (4) or (5) are integrated at constant entropy from stagnation to a condition where equation (1) is satisfied, for which

$$S = \sqrt{-g_c \left(\frac{\partial p}{\partial v}\right)_s}$$
(8)

Equation (8) is the well known expression for critical flow of a homogeneous fluid, which can be shown to occur simultaneously with the sonic state either in the throat of a converging-diverging nozzle or in a uniform area flow passage.

Computations giving the critical mass flux and critical flow pressure for a homogeneous steam-water mixture are graphed in Figures 1 and 2 in terms of stagnation pressure and enthalpy. Both subcooled and saturated states are shown.

#### A CONSISTENT SLIP MODEL

Whether or not critical flow and sonic states occur simultaneously for flow patterns other than homogeneous has been questioned. Faletti (1) noted that at maximum steam-water flows, further reduction of receiver pressure slightly affected pressures inside the flow passage. Isbin (3) suggested that in annular flow, pressure disturbances might propagate upstream in the liquid film. However, Henry (25) concluded that two-dimensional effects near the exit of two-phase critical discharge explained apparent receiver pressure influence for short distances upstream. He found that these effects could be reduced with gradual rather than abrupt expansion at the pipe exit. It is shown in this section that for



#### STAGNATION ENTHALPY, ho/hREF

## Fig.1 Critical mass flux - homogeneous, equilibrium steam-water

an idealized one-dimensional slip model, critical flow and sonic states can occur simultaneously.

Several two-phase slip models for critical flow by Fauske (4), Levy (12), and Moody (13) are based on either momentum or energy conservation, but not both, and hence are not consistent. Since the slip ratio introduced one more degree of freedom, additional assumptions were imposed, such as isentropic or isenthalpic flow. These assumptions do not necessarily describe the physical behavior.

This study includes all the conservation laws, and hence is consistent. The slip model developed here is described by separated streams of liquid and vapor in saturated equilibrium undergoing one-dimensional flow at average, unequal speeds. Momentum and kinetic energy transport due to phase change at the interface is based on speed of the initial phase, whereas transport of enthalpy and entropy is based on the final phase. These idealizations primarily affect vaporization and condensation rates. Heat transfer and shear stress at the wall, and interfacial shear stress are considered neglizible.

Following a procedure similar to that for homogeneous flow, mass, momentum, and energy conservation equations for each phase are written for a small disturbance moving leftward at absolute speed V in a passage of variable area, into steady slip flow to the right. Subscripts i and j refer to either the liquid or vapor. When i and j both appear in an expression, i  $\neq$  j. Derivations are given in Appendix I.

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Mass: 
$$d[A_{i}(u_{i}+V)/v_{i}] + \hat{m}_{ij}$$
  
-  $\hat{m}_{ji} = 0$  (9)

Momentum:  $d[A_i(u_i+V)u_i/v_i] + \hat{m}_{ij}u_i$ 

$$-\dot{m}_{ji}u_{j} + g_{c}A_{i}dp = 0$$
 (10)

Energy: 
$$d[A_{i}(u_{i}+V)h_{oi}/v_{i}] + \hat{m}_{ij}h_{oij}$$
  
-  $\hat{m}_{ji}h_{oji} - A_{i}Vdp = 0$  (11)

Stagnation enthalpies are defined by

$$h_{oi} = h_i + u_i^2 / 2R_c$$
 (12)

$$h_{oij} = h_j + u_i^2 / 2g_c$$
 (13)

$$h_{oji} = h_i + u_j^2 / 2g_c$$
 (14)

The second law is not required for integration of the conservation equations, but it is useful later for identifying the stable range of slip ratio:

acond law: 
$$d[A_i(u_i+V)s_i/v_i]$$
  
+  $\dot{m}_{ij}s_j - \dot{m}_{ji}a_i \ge 0$  (15)

Next, the conservation equations for both phases are added and simplified with the substitutions,

$$u_{g} = v^{*}G$$
 (10)

$$K = u_g / u_f$$
(17)

$$v^{*} = xv_{y} + (1-x)kv_{f} = v^{*}(p,x,k)$$
 (18)

$$A_{\rho}/A = xv_{\rho}/v^{*}$$
(19)

$$GA = u_g A_g / v_g + u_f A_f / v_f$$
(20)

$$\Lambda = \Lambda_{g} + \Lambda_{f}$$
(21)

$$x = u_g A_g / v_g G A$$
 (22)

$$\alpha = A_g/A$$
 (23)

$$v_m = v^{\pm} \left( x + \frac{1-x}{K} \right) = v_m(p, x, K)$$
 (24)

$$v_e^2 = v^2 \left(x + \frac{1-x}{K^2}\right) = v_e^2(p, x, K)$$
 (25)

The resulting combined equations are given by,

Mass: 
$$d(GA) + d\left[AV\left(\frac{\alpha}{v_g} + \frac{1-\alpha}{v_f}\right)\right] = 0$$
 (26)

Momentum: d(G\*Av<sub>m</sub>) + d(VGA)

$$g_c Adp = 0$$
 (27)

nergy: 
$$d(GAh_o) + d\left[V\left(\frac{A_f}{v_f}h_{of}\right) + \frac{A_g}{v_g}h_{og}\right)\right] - AVdp = 0$$
 (28)

Second law:  $d(GAS) + d \left[ v \left( \frac{t}{v_f} s_f \right) \right]$ 

$$+\frac{A_g}{v_g}s_g
ight) \ge 0$$
 (29)

in which

$$h_{o} = h(p,x) + G^{2} v_{e}^{2}(p,x,K) / 2g_{c}$$
  
= h\_{o}(p,x,G,K) (30)

$$h(p,x) = xh_{\sigma} + (1-x)h_{f}$$
 (31)

$$s(p,x) = xs_{\alpha} + (1-x)s_{f}$$
(32)

Equations (26) through (29) express differential changes across a small disturbance which moves against the flow. If conditions of equation (2) are satisfied, an appropriate interpretation for a general flow passage with variable area is that no disturbance is present. Another interpretation which applies to either a uniform area flow passage or a nozzle throat is that a disturbance does exist but is stationary (see Appendix II). For either case, imposing the conditions of equation (2) reduces equations (26) through (29) to the steady equations for slip flow [see for example, Wallis (23) or Lahey (22)]:

Mass: 
$$d(GA) = 0$$
 (33)

mentum: 
$$d\left[G^2Av_m(p,x,K)\right] + g_eAdp = 0$$
 (34)

nergy: 
$$d[GAh_{(p,x,G,K)}] = 0$$
 (35)

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(36)

### Second law: $d[GAs(p,x)] \ge 0$

Suppose that steady flow occurs, and that the critical flow condition of equation (1) is satisfied either at a nozzle throat or in a uniform area section. Expansion of equations (26) through (29) shows that if equation (1) is satisfied, conditions of equation (2) also would be satisfied if a disturbance occurred. It follows that - even in slif flow - the sonic and critical flow states can occur simultaneously.

It is desirable to integrate the flow properties from some known state, say stagnation conditions in the vessel, to the critical flow state of equation (1). If p is considered independent, equations (26), (27), and (28) include four dependent variables, namely G, A, x, and K. Obviously one more equation is required for integration Ogasawara (<u>26</u>) and Giot and Frite (<u>27</u>) showed that momentum or energy conservation for either phase is sufficient for one more independent equation. However, an approach which involves entropy production is equivalent, and is employed in this study. The Gibbs equation for either phase,

$$Tds_{ij} = dh_{ij} - v_{ij}dp \qquad (37)$$

and the Clapeyron equation,

$$s_{fo} = h_{fo}$$
 (38)

were employed with equations (9) through (14), (20), (31), and (32) to express the entropy differential as,

$$ds(p,x) = \frac{1}{T} \left( \frac{\dot{m}_{fg} + \dot{m}_{gf}}{(dp)GA} \right) \left( \frac{u_g - u_f}{2g_g} \right)^2 dp$$
$$= \partial(p,x,G,K) dp$$
(39)

The vaporization and condensation rates per unit pressure reduction for phase equilibrium are given by

$$\frac{\dot{m}_{fg}}{(dp)GA} = \frac{T}{\dot{m}_{fg}} \left[ \frac{-(1-x)s_{f}' + Zxs_{g}}{1 + Z^{2}} \right]$$
(40)

$$\frac{\dot{m}_{gf}}{(up)GA} = \frac{T}{h_{fg}} \frac{\left[xs_{g}' + Z(1-x)s_{f}'\right]}{1 + Z^{2}}$$
(41)

where

$$z = \frac{(u_g - u_f)^2}{2g_c h_{fg}}$$
(42)

Equation (39) displays the entropy production due to phase change with relative motion between the phases, discussed by Wallis (23). Although interfacial friction is neglected, the entropy-producing mechanism is an effective shear stress, generated by momentum transfer between the phases which are moving at unequal velocities. Equation (39) provides one more independent equation for use with equations (26), (27), and (28).

Equation (26) is useful in eliminating flow area, A, from (27), (28), and (39), which can be written in matrix form as

$$\begin{bmatrix} a_{11} & a_{12} & a_{13} \\ a_{21} & a_{22} & a_{23} \\ a_{31} & a_{32} & a_{33} \end{bmatrix} \begin{bmatrix} dG \\ dx \\ dK \end{bmatrix} = \begin{bmatrix} b_1 \\ b_2 \\ b_3 \end{bmatrix} dp$$
(43)

where



Equation (43) can be integrated from stagnation to the point where equation (1) is satisfied. For this procedure, a starting value of K is required. Equations (9) and (10) show that at the stagnation limit where G approaches zero, the value of K must be  $(v_g/v_f)^{1/2}$ . Resulting values of the slip critical mass flux always are higher than the homogeneous flow values, and are within about 6 percent of earlier results presented by Moody (13), which are based on mass and energy conservation, and the assumption of isentropic flow. However, it is seen from equation (39) that isentropic flow cannot occur unless there is either no phase change or relative motion between the phases.

Homogeneous critical flow is expected to control mass flux in the pipe entrance region during blowdown. Therefore, it is necessary to consider slip critical flow in terms of local properties. Applying equation (1) to (43), the critical flow condition becomes,

$$\frac{dG}{dp} = \begin{pmatrix} b_1 & a_{12} & a_{13} \\ b_2 & a_{22} & a_{23} \\ b_3 & a_{32} & a_{33} \end{pmatrix} = 0$$
(45)

which results in a solution for the slip critical mass flux  $G_{\mu}(\mathbf{p},\mathbf{x},K)$ .

#### STABLE RANGE OF SLIP RATIO

A reasonable lower limit for K is 1.0 for phases traveling at equal speeds. An upper limit can be determined from the second law. Eliminating GA from equations (33) and (36) and combining with (39), it follows that

From equations (43) and (46), it can be shown that

$$\theta \frac{dp}{dK} dK = \theta \frac{a_{11} a_{12} a_{13}}{a_{21} a_{22} a_{23}} dK \ge 0$$
(47)  
$$\theta \frac{dp}{dK} dK = \theta \frac{a_{11} a_{12} b_{1}}{a_{21} a_{22} b_{2}} dK \ge 0$$
(47)

A detailed study of the above inequality shows that for fixed values of p, x, and G, K will always be changing in a direction which makes

The value of K which satisfies equation (48) is considered the stable value, and also the upper limit. Simultaneous solutions of equations (45) and (48) were obtained numerically to express the critical mass flux and the maximum slip ratio in terms of local pressure and quality. Results are graphed in Figure 3. Also shown are results for a slip ratio of 1.0.



Fig.3 Critical flow states - slip model

#### COMPARISONS WITH DATA

Equilibrium blowdown data is employed in this section to show that two-phase blowdown rates are controlled by homogeneous choking near the pipe entrance, followed by a second choked condition for slip flow near the exit.

### Two Critical Flow States During Blowdown

Figure 4 presents blowdown mass flux and pipe exit pressure data of Sozzi and Sutherland (7) for 1.27 cm (0.5 in.) i.d. pipes of various lengths and vessel stagnation pressures between 65 and 70 atmospheres. The data is graphed as a function of stagnation quality, defined as

$$x_{0} = \frac{v - v_{f}(p_{0})}{v_{fg}(p_{0})}$$
(49)



Blowdown rates and exit pressures of Sozzi Fig.4 and Sutherland (7) at 70 atmospheres

Calculations for the critical flow rate and pipe discharge pressure are shown by the solid and dotted lines, respectively, for homogeneous and slip theory. Homogeneous theory better predicts the mass flux data, indicating that blowdown rates were limited by homogeneous critical flow near the pipe entrance. Slip model predictions of the blowdown rate in terms of stagnation properties are too high. For the pipe exit, calculated homogeneous critical flow rates were employed in the slip model to predict pipe exit pressures from equations (45) and (48). The slip theory prediction better represents exit pressure data. A consistent interpretation of Figure 4 is that biowdown rates are limited by homogeneous equilibrium

critical flow near the pipe entrance, and a second critical flow state occurs at the pipe exit which corresponds to a slip flow pattern.

#### Saturated Water Blowdown

Figure 5 shows the blowdown mass flux obtained by different workers for saturated water at various stagnation pressures. Uchida (24), Fauske (19), Henry (6), and Sozzi and Sutherland (7) employed uniform pipes from 0.4 to 1.3 cm (0.16 to 0.513 in.) i.d. Allemann (14) conducted vessel blowdowns through pipes up to 17.3 cm (6.8 in.) i.d. Data is shown only for pipes 10 cm (4 in.) or longer so that the flow rates are close to equilibrium. The solid curve shows critical mass flux calculated from homogeneous equilibrium theory. Agreement is sufficiently close to verify that the homogeneous model predicts saturated water blowdown rates for given vessel pressures. This supports an eaclier conclusion of Simon (21).



STAGNATION PRESSURE, PO/PREF

#### Fig.5 Blowdown rate of saturated water

Calculated homogeneous critical mass flows were compared with pipe blowdown rates of Moody (15). Results showed that when the pipe friction parameter, fL/D, is greater than about 3.0, saturated water discharge rates are limited by friction and choking at the pipe exit, and therefore are less than values determined by homogeneous critical flow at the entrance.

#### Subcooled and Saturated Blowdown

Figure 6 presents the blowdown mass flux data of Henry (6) for steam-water discharge through a 0.9 m (3 ft) long, 0.8 cm (0.313 in.) i.d. pipe. Static pressure taps were located on the pipe to give a description of the pressure profile. Stagnation pressure was estimated from the most upstream pressure tap, located 76 cm (30 in.) from the pipe exit. Blowdown rates calculated from homogeneous theory are shown by solid lines. Close agreement was obtained showing that for both saturated and subcooled stagnation states, blowdown rates are limited by homogeneous choking near the pipe entrance.



Fig.6 Blowdown rates of Henry (6)

Figure 7 shows pipe exit pressures for the same data of Henry (6) presented in Figure 6. Blowdown rates calculated from the homogeneous model in terms of stagnation properties were employed to determine pipe exit pressure from the slip model. Calculated results compare sufficiently well to verify the existence of a choked slip condition at the exit.

#### Blowdown Flow and Pipe Discharge Properties

Data for steam-water blowdowns in terms of pipe exit properties has been obtained by numerous workers and reported in References 1 through 5. In addition, data of Henry ( $\underline{6}$ ), and Sozzi and Sutherland ( $\underline{7}$ ), already presented in Figures 4, 6, and 7 in terms of stagnation properties, includes the pipe exit properties. The pipe exit data is presented in Figure 8 as a function of static pressure and quality. All data is for flow passages longer than 10 cm (4 in.) so that equilibrium is assured. The solid curves were calculated from the slip model at the maximum slip ratio. Agreement further indicates that near the pipe exit, a choked slip flow occurs.



Fig.7 Pipe exit pressures of Henry (6)

Low quality data of Henry (6) is shown in Figure 9. For quality below 0.02, predictions with the slip model at the maximum slip ratio are too low, and the data appear to be bounded by a slip ratio of 1.0 and the maximum value.

#### CONCLUSIONS

The following conclusions from this study are restricted to flow passages sufficiently long for saturated equilibrium to be closely approached between the liquid and vapor. For steam-water flows, the flow length should be 10 cm (4 in.) or longer.

1 During two-phase blowdown from vessels, flow pattern in the pipe entrance is a homogeneous mixture of vapor bubbles in liquid.

2 Two-phase blowdown rates from vessels are predicted by the homogeneous, equilibrium, choked state in terms of stagnation properties in the vessel (Fig.1).

3 Downstream from the pipe entrance, the twophase blowdown probably tends toward a slip flow pattern.

4 Pipe discharge rates at the exit are predicted by the choked state of a slip flow model.

5 The pipe exit choked flow state does not influence homogeneous choking at the entrance for pipe friction parameters of fL/D < 3.0.

6 For values of the friction parameter fL/D > 3.0, pipe entrance flow will unchoke, and blowdown rates will be less than that predicted by homogeneous choking in terms of vessel stagnation properties.

7 The slip ratio in an equilibrium choked flow is bounded by 1.0 and an upper limit determined by second law requirements.



Fig.8 Critical flow properties at pipe exit

REFERENCES

1 Faletti, D. W., "Two-Phase Critical Flow of Steam/Water Mixtures," Dissertation, 1959, University of Washington.

2 Moy, J. E., "Critical Discharges of Steam/ Water Mixtures," M.S. Thesis, 1955, University of Minnesota.

3 Isbin, H. S., Moy, J. E., and DaCruz, A. J. R., "Two Phase Steam/Water Critical Flow," Trans. AIChE, Vol. 3, No. 3, 1957, pp. 361-365.

4 Fauske, H. K., "Contribution to the Theory of Two-Phase, One-Component Critical Flow," ANL-6633, 1962, Argonne National Laboratory, LeMont, III.

5 Zaloudek, F. R., "The Low Pressure Critical Discharge of Steam/Water Mixtures from Pipes," HW-68934 Rev., 1961, Hanford Atomic Products Opera-

tion, Richland, Wash. 6 Henry, R. E., "An Experimental Study of Low-Quality, Steam-Water Critical Flow at Moderate Pres-sures," ANL-7740, 1970, Argonne National Laboratory, LeMont, 111.

7 Sozzi, G. L., and Sutherland, W. A., "Criti-cal Flow of Saturated and Subcooled Water at High Pressure," NEDO-13418, May 1975, General Electric Company, San Jose, Calif.

8 Henry, R. E., "A Study of One- and Two-Component, Two-Phase Critical Flows at Low Qualities." ANL-7430, 1968, Argonne National Laboratory, LeMont, 111.

8



#### Fig.9 Low quality data of Henry (6)

9 Henry, R. E., "The Two-Phase Critical Discharge of Initially Saturated or Subcooled Liquid," Nuclear Science and Engineering, Vol. 41, 1970, pp. 336-342.

10 Edwards, A. R., "Conduction Controlled Flashing of a Fluid, and the Prediction of Critical Flow Rates in a One-Dimensional System," AHSB(S)R-147, 1968, U.K.A.E.A., A.W.R.E., Foulness Island, South End on Sea, Essex, England.

11 Boure', J. A., "Two-Phase Flows with Application to Nuclear Reactor Design Problems," vonKarman Institute for Fluid Dynamics Lecture Series, Dec. 9-13, 1974, Service des Transferts Thermiques, Centre d'Etudes Nucleaires de Grenoble, France.

12 Levy, S., "Prediction of Two-Phase Critical Flow Rate," Trans. ASME, JHT, Ser. C, Vol. 87, No. 1, 1965, p. 53.

13 Moody, F. J., "Maximum Flow Rate of a Single Component, Two-Phase Mixture," Trans. ASME, JHT, Ser. C, Vol. 87, No. 1, 1965, p. 134.

14 Allemann, R. T., et al., "Experimental High Enthalpy Water Blowdown from a Simple Vessel Through a Bottom Outlet," BNWL-1411, 1970, Battelle Northwest Laboratory, Richland, Wash.

15 Moody, F. J., "Maximum Two-Phase Vessel Blowdown from Pipes," <u>Trans. ASME</u>, JHT, Ser. C, Vol. 88, 1966, p. 285.

16 Edwards, A. R., and O'Brien, T. P., "Studies of Phenomena Connected with the Depressurization of Water Reactors," Journal of the <u>British Nuclear</u> Energy Society, Vol. 9, 1970. 17 Zaker, T. A., and Wiedermann, A. H., "Water Depressurization Studies," IITRI-578-p-21-26, 1966,

Illinois Institute of Technology, Chicago, Ill.

18 Gallagher, E. V., "Water Decompression Ex-periments and Analysis for Blowdown of Nuclear Reactors," IITRI-578-p-21-39, 1970, Illinois Institute

of Technology, Chicago, Ill. 19 Fauske, H. K., "The Discharge of Saturated Water Through Tubes," Symposium Series 61, Chemical Engineering Progress, 1965, p. 210.

20 Uchida, H., and Nariai, H., "Discharge of Saturated Water Through Pipes and Orifices, Proceedings of the Third International Heat Transfer Conference, Vol. 5, 1966. 21 Simon, U., "Blowdown Flow Rates of Initially

Saturated Water," Paper presented at the European

Two-Phase Flow Meeting, Risø, Denmark, June 1971. 22 Lahey, R. T., "Two-Phase Flow in Boiling Water Nuclear Reactors," NEDO-13388, July 1974, General Electric Company, San Jose, Calif.

23 Wallis, G. B., <u>One Dimensional Two-Phase</u> Flow, McGraw-Hill, New York, 1969.

24 Kober, K. J., et al., "Untersuchungen zur Simulation der Drukentlastung von Siedwasserreactoren," Forschungsauftrag Nr. RS 16, 1968, Battelle Institute, Frankfurt, Germany.

25 Henry, R. E., Fauske, H. K., and McComas, S. T., "Two-Phase Critical Flow at Low Qualities, Part 1: Experimental," Nuclear Science and Engineering, Vol. 41, 1970, pp. 79-91.

26 Ogasawara, H., "A Theoretical Prediction of Two-Phase Critical Flow," Japan Society of Mechanical Engineers Bulletin, Vol. 10, No. 38, 1967. 27 Giot, M., and Fritte, A., "Two-Phase, Two-

and One-Component Critical Flows with the Variable Slip Model," Progress in Heat and Mass Transfer, Vol. 6, Pergamon Press, New York, 1972, p. 651.

APPENDIX I CONSERVATION EQUATIONS FOR MOVING DISTURBANCE

Figure A-1 shows two separated phases, designated i and j, undergoing one-dimensional flow to the right in a flow passage of variable area, with a disturbance moving to the left at velocity V. Wall and interface friction are assumed to be negligible.

Static pressure is uniform through each phase at any flow cross section. Moving with the disturbance in phase i is a control volume which is sufficiently thin that mass, momentum, and energy storage inside can be neglected. Mass conservation for the control volume is written as

$$\dot{m}_{i} + \dot{m}_{i4} - \dot{m}_{i4} = 0$$
 (50)

where  $\dot{m}_1$  is the mass flow rate relative to the control volume, given by

> (51)  $\dot{\mathbf{m}}_{i} = \mathbf{A}_{i} \left( \mathbf{u}_{i} + \mathbf{V} \right) / \mathbf{v}_{i}$

Equations (50) and (51) can be combined to give equation (9).

Momentum conservation is written assuming that mass rate mij leaves the control ' lume with velocity ui and mii enters with velocity wi:

$$d(\hat{m}_{i}u_{i}) + \hat{m}_{ij}u_{i} - \hat{m}_{ji}u_{i} + g_{c}A_{i}dp = 0$$
 (52)

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Equations (51) and (52) yield (10).

It is assumed that mass rate fij leaves the control volume as phase j with static enthalpy hy but with velocity ui, and that miji enters as phase i with hi and uj. Appropriate stagnation enthalpies are given by equations (12), (13), and (14). Work is done as the control volume moves at rates pAiV,



MOVING DISTURBANCE



Fig.A-1 Control volume moving with disturbance

 $pVdA_{\rm i},$  and  $(p+dp)(A_{\rm i}+dA_{\rm i})V.$  Therefore, energy conservation for the moving control volume is written as

$$d(\hat{m}_{i}h_{oi}) + \hat{m}_{ij}h_{oij} - \hat{m}_{ji}h_{oji} - A_{v}Vdp = 0$$
(53)

Equations (51) and (53) yield equation (11). Finally, mass rate  $\hat{m}_{1j}$  leaves the control volume at specific entropy  $s_j$ , and  $\hat{m}_{j1}$  enters at  $s_i$ . Therefore, the second law is written as

$$d(\hat{m}_{i}s_{j}) + \hat{m}_{ij}s_{j} - \hat{m}_{ji}s_{i} \ge 0$$
 (54)

Equations (51) and (54) yield (15).

APPENDIX II SIMULTANEOUS CRITICAL FLOW AND SONIC STATES

For the critical flow state, equation (26) is expanded, and the condition of equation (1) is imposed to give

$$GdA + AVd \left(\frac{\alpha}{v_g} + \frac{1-\alpha}{v_f}\right) + \left(\frac{\alpha}{v_g} + \frac{1-\alpha}{v_f}\right) (AdV + VdA) = 0$$
(55)

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If the sonic state, given by conditions of equation (2) is to apply simultaneously with the critical flow state, equation (55) req ires that dA = 0. It follows that one-dimensional two-phase critical flow and the sonic states can occur simultaneously in a uniform area flow passage or in a nozzle throat. If only the sonic conditions of equation (2) are imposed, the expanded form of equations (26) through (29) reduce to equations (33) through (36), which are identical to the steady-flow equations in a flow passage of variable area. Therefore, when the steady-flow equation of equation (1) in a section where dA = 0, the sonic and critical flow states occur simultaneously.

## SUPPLEMENTARY INFORMATION

NRC request for this information was contained in a letter from D. G. Eisenhut (NRC) to L. J. Sobon (GE) dated August 29, 1977

GE response with the following information was provided in a letter from L. J. Sobon (GE) to D. G. Eisenhut (NRC) dated November 8, 1977

## REPLY TO NRC REQUEST FOR ADDITIONAL INFORMATION ON NEDO 21052

## SECTION I. SUMMARY POINTS OF NEDO-21052

The following summary of NEDO-21052 will be helpful in the discussion which presents additional information requested by the NRC.

<u>Point 1.</u> Results apply to cases where two-phase saturated equilibrium is closely approached.

<u>Point 2.</u> Data indicates that saturated equilibrium is closely approached during mixture travel of approximately 10 cm into the discharging flow passage.

<u>Point 3.</u> Two choked flow conditions can occur in a blowdown flow passage. One choked condition occurs near the entrance, and another choked condition at the exit. See Figure AA.

<u>Point 4.</u> Bulk boiling of fluid particles undergoing rapid depressurization in the pipe entrance region first produces a homogeneous flow pattern of vapor bubbles in liquid. Therefore, equilibrium choked flow rates are limited by the homogeneous equilibrium model (HEM) near the entrance region. Once in the flow passage, decompression of a fluid particle is slower and a slip flow pattern forms, leading to a second slip-choked condition at the exit.

<u>Point 5.</u> The entrance choked condition is predicted by the HEM in a form which relates vessel pressure  $P_0$ , entrance stagnation enthalpy  $h_0$ , and choked mass flux  $G_c$ . The flow rate proceeding through the passage is limited by entrance homogeneous choking.

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<u>Point 6.</u> The exit flow rate already is determined by entrance choking. Some degree of phase separation occurs in the flow passage between entrance and exit. The discharge condition corresponds to a choked slip equilibrium model (SEM) for which  $G_c$  and  $h_o$  are determined from entrance conditions. Exit static pressure  $P_e$ adjusts to accommodate  $G_c$  and  $h_o$  (or equivalently the exit quality  $x_e$ ) for the SEM choked flow rate.

<u>Point 7.</u> Most of the early equilibrium choked flow data is based on discharge exit properties, best correlating  $P_e$ ,  $x_e$ , and  $G_c$  at the exit by a SEM.

Point 8. Equilibrium data in terms of vessel properties is best correlated by the HEM.

<u>Point 9.</u> A counterpart of the two choked states in two-phase flow is found in gas dynamics. The restriction of gas flow by a nozzle in which the throat Mach number is 1.0, limits mass flow in terms of reservoir properties.

However, downstream shocking and pipe friction may produce an exit condition also at Mach 1.0, with the same flow rate and stagnation enthalpy occuring at the nozzle, but at pressure, density, and temperature different from nozzle properties.

<u>Point 10.</u> It is not possible to take choked flow data presented in terms of discharge properties and directly obtain stagnation properties appropriate for upstream homogeneous choking. The reason is that although stagnation enthalpies at entrance and exit are equal, the entropy change associated with transition from homogeneous to slip flow reduces the stagnation pressure in the direction of flow. A counterpart in gas dynamics would be to take discharge choked flow properties and attempt to predict entrance stagnation properties without knowing the area ratio of an upstream Mach 1.0 nozzle which limits the flow rate.

## SECTION II. RESPONSE TO NRC REQUESTS

## REQUEST 1

Critical flow measurements have been made by a number of experimenters. In several cases the data have been compared with the predictions of the homogeneous equilibrium model (HEM). In each of the references listed below the experimentally determined critical flows were found to exceed the values predicted by the HEM model.

These conclusions indicate that use of HEM to predict break flow is not conservative for containment analysis. For each of the following data sets provide a comparison of the experimental critical flows with the homogeneous equilibrium model in NEDO-21052 and justify any differences in your conclusions with those of the author.

- a. M. W. Benjamin and J. G. Miller, "The Flow of Saturated Water Through Throttling Orifices," Trans. ASME, Vol. 63, p. 419 (1941).
- b. D. W. Faletti and R. W. Moulton, "Two-Phase Critical Flow of Stream-Water Mixtures," <u>AIChE Journal, Vol. 9</u>, p. 247 (1963).
- c. F. R. Zaloudek, "The Critical Flow of Hot Water Through Short Tubes," HW-77594, Hanford Works, (1963).

F. R. Zaloudek, "Steam-Water Critical Flow From High Pressure Systems," Interim Report, HW-80535, Hanford Works, (1964).

- d. H. K. Fauske, "Contribution to the Theory of Two-Phase, One-Component Critical Flow," AEC Report, ANL-6633, (October 1962).
- e. R. E. Henry, "A Study of One- and Two-Component, Two-Phase Critical Flows at Low Qualities," ANL-7430, Argonne National Laboratory, (1968). R. E. Henry, H. K. Fauske, and S. T. McComas, "Two-Phase Critical Flow at Low Qualities," <u>Nuclear Science and Engineering, Vol. 41,</u> pp. 79-98, (1970).

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f. E. S. Starkman, V. E. Schrock, K. F. Neusen, and D. J. Maneely, "Expansion of a Very Low Quality Two-Phase Fluid through a Convergent-Divergent Nozzle." <u>Journal of Basic Engineering</u>, Trans, ASME, pp. 247-256, (June 1954).

## RESPONSE TO REQUEST 1

Responses to this request, given below, refer to each individual reference by the alphabetical designation above.

- (a) An orifice does not represent a flow passage long enough to assure phase equilibrium (Summary Points 1 & 2 of Section I). The models of NEDO-21052 are restricted to equilibrium states, and therefore permit no meaningful comparison with orifice flows. In the limit, an orifice would represent a flow length of zero, which clearly results in too short a transit time for nucleation and bubble growth to achieve phase equilibrium.
- (b) Faletti's data was obtained in annular flow passages which were long enough to assure phase equilibrium. Figure 8 of NEDO-21052 shows that this data is well predicted by the SEM in terms of <u>exit choked properties</u>. However, upstream stagnation properties were not presented and a meaningful comparison with the HEM cannot be made (Summary Points 1 & 2 of Section I).
- (c) A meaningful comparison cannot be made with data of Zaloudek for short flow passages of 3.0 inches and less (Points 1 & 2 of Section I). However, Zaloudek's data for longer flow passages is well predicted by the SEM in Figure 8 of NEDO-21052.
- (d) Fauske's data was obtained in test sections long enough to assure phase equilibrium, and is included in Figure 8 of NEDO-21052. The data is seen to be predicted by the SEM as

expected since measurements yielded exit properties. This data in terms of exit properties represents a slip-choked condition, and should not correlate with the HEM. Additional work of Fauske reported in Ref. 19 of NEDO-21052, which gives stagnation pressures, is predicted well by the HEM.

(e) Henry's reference ANL 7430 presents data similar to that given in ANL 7740, which is shown in Figures 6 and 7 of NEDO-21052, and strongly supports the existence of entrance homogeneous and exit slip choked conditions.

The references of Henry, Fauske, and McComas deals with two-dimensional rapid expansion effects at the exit. Their study incorporates data reported elsewhere, and leads to the conclusion that although discharge slip ratios are less than required for annular flow, some degree of slip occurred, which is consistent with the description of NEDO-21052. No further comparisons were made with material in these references.

(f) The data of Starkman, et.al. was based on short converging-diverging flow nozzles, and therefore is not appropriate for comparison with equilibrium models (Points 1 & 2 of Section I).

## REQUEST 2

A comparison of various critical flow data in the following reference including those data of G. Sozzi\* demonstrates that the HEM model underpredicts experimental data. Justify the differences between the conclusions in NEDO-21052 and those of the authors.

> K. H. Ardron and R. A. Furness, "A Study of the Critical Flow Models Used in Reactor Blowdown Analysis," Nuclear Engineering and Design 39, (1976), p. 257-266.

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<sup>\*</sup> Sozzi, G. L., and Sutherland, W. A., "Critical Flow of Saturated and Subcooled Water at High Pressure," NEDO-13418, May 1975.

#### **RESPONSE TO REQUEST 2**

The following responses pertain to figures contained in the reference by Ardron and Furness.

The data of Maneely and Friedrich in Figure 2 is for flow passages too short to assure equilibrium (Points 1& 2 of Section I). Therefore, no valid comparison can be made with equilibrium models. The data of Fauske and Faletti in Figure 2 was not presented in the original references in terms of stagnation conditions. The procedure employed by Ardron and Furness to obtain stagnation properties is not described, and leaves the results of Figure 2 open to question (Point 10 of Section I). The same data of Faletti and Fauske compares favorably with the SEM in Figure 8 of NEDO-21052 in terms of the published discharge properties.

Data in Figure 3 is for flow passages too short to achieve phase equlibrium, and should not be compared with either the HEM in terms of stagnation properties or the SEM in terms of discharge properties (Points 1 & 2 of Section I).

Figure 4 at 62 bars includes some data from flow passages too short for equilibrium, i.e. less than about 10 cm, for which no valid comparison can be made with equilibrium models (Points 1& 2 of Section I). The remaining data of Sozzi and Sutherland falls close to the HEM curve. It appears that the Sozzi-Sutherland experimental points selected for this comparison are from the upper error band of their collected data since additional data from the same series of tests, presented in Figure 4 of NEDO-21052, shows a distribution above and below the HEM.

Figure 5 for saturated water blowdown contains only a few data points of Fauske at L/D = 40 with D = 6 mm and of the apparently upper error band data of Sozzi and Sutherland at L/D = 18 and 29 with D = 12.7 mm, most of which lie close to the HEM. Figure 5 of NEDO-21052 shows most of the same applicable data plus some additional points of Allemann for a 6.8 inch pipe, which lie below the HEM.

Although it is a minor observation, it appears that the theoretical HEM curves shown by Ardron and Furness are somewhat lower than those calculated using the 1967 ASME Steam Tables, used in Ref. 1. Differences are perhaps 10 percent, but it does cause more of the data to lie below the HEM curve than is indicated by their graphs.

### REQUEST 3

Semi-scale tests discussed in TREE-NUREG-1006 have produced transient critical flow data that are in general agreement with the HEM model for two-phase flow but are in excess of HEM for saturated and subcooled flows. Provide a comparison of the HEM model discussed in NEDO-21052 with the Semi-scale test data and justify the differences in your conclusion with those of the author.

> D. G. Hall, "A Study of Critical Flow Prediction for Semi-scale Mod-1 Loss-of-Coolant Accident Experiments," TREE-NUREG-1006, Idaho National Engineering Laboratory, December 1976.

## RESPONSE TO REQUEST 3

The HEM of NEDO-21052 should not be compared with the semi-scale tests discussed in TREE-NUREG-1006, which were based on flow nozzles too short to permit phase equilbrium (Points 1 & 2 of Section I). The semi-scale tests use a "Henry" nozzle for limiting the blowdown flow rate. This nozzle has a relatively short throat length, less than 1.0 inch and is known to produce non-equilibrium phase change. Henry "calibrated" the non-equilbrium effects by an "N-Factor" to account for differences between equilibrium and non-equilibrium flashing. It is interesting to note that semi-scale has recently replaced the Henry nozzle with one of slightly longer throat length, about 3.0 inch. \*\* This new nozzle is

Henry, R. R., and Fauske, H. K., "The Two-Phase Critical Flow of One Component Mixtures in Nozzles, Orifices, and Short Tubes," ASME Paper No. 70-WA/H7-5.

TREE-NUREG-1118, Aug. 1977

scaled from the LOFT counterpart. Although these nozzles (Henry and LOFT counterpart) had the same flow area, the longer length nozzle produced significantly lower blowdown flow rates.

## REQUEST 4

Page 2 of NEDO-21052 states that for pipe lengths greater than 10 cm, the discharge rate is unaffected by further increases in pipe length. This conclusion is inconsistent with the data of Sozzi\* in Figure 5 which indicate the flow is greater for a 9-inch pipe length than for a 4.5-inch length or a 12.5-inch length. Discuss the reasons for this inconsistency.

## **RESPONSE TO REQUEST 4**

The word "unaffected" in NEDO-21052 is too strong, and would be accurate if it were revised to read "not strongly affected". Further increases of pipe length beyond approximately 10 cm show an extremely reduced effect on flow rate. The data of Sozzi and Sutherland shown in Figure 5 of NEDO-21052 is subject to a degree of uncertainty in measurement, and some scatter is expected. The collection of data presented in Figure 5 of NEDO-21052 from several investigators shows a strong trend of agreement with the HEM.

#### REQUEST 5

Page 3 states that steady-state flow is required upstream before HEM equation 8 can be derived. Justify the applicability of this equation for transient critical flow.

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Sozzi, G. L., and Sutherland, W. A., "Critical Flow of Saturated and Subcooled Water at High Pressure," NEDO-13418, May 1975.

#### RESPONSE TO REQUEST 5

The term "steady flow" is here regarded as "quasi-steady flow". Unsteadiness in the flow is attributed to either the initial rupture and decompression of a pipe, or the time-dependent vessel pressure reduction. The material of NEDO-21052 applies after initial pipe decompression when all propagation effects have disappeared. Instantaneous values of vessel pressure should be employed in predicting discharge rates as long as vessel decompression rate is slow relative to the residence time of a fluid particle as it travels through the pipe.

### REQUEST 6

Provide the details of the mathematical solution of equation 8 used to produce the critical flows in Figure 1 and the critical pressure ratios in Figure 2.

#### RESPONSE TO REQUEST 6

One could obtain the partial derivative of v with respect to P at constant entropy s directly from a programmed steam table library at given values of P and quality x, then obtain corresponding stagnation enthalpy  $h_0$  and pressure P<sub>0</sub> from additional energy and state equations. However, the procedure employed to obtain Figure 1 of Ref. 1 was based on the energy equation,

$$h_{o} = h + \frac{G^2 v^2}{2g_{c}}$$

for homogeneous flow. The above equation was written as

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$$G = \sqrt{\frac{2g_c(h_o - h_f - xh_{fg})}{v_f + xv_{fg}}}$$

where

$$= \frac{s_0 - s_f}{s_{fo}}$$

X

Beginning with stagnation properties P and h, the entropy s was determined from the 1967 ASME Steam Properties. Then G was calculated for successively lower pressures P until an absolute maximum of G was found numerically, which was termed GM at pressure PM. This procedure is valid for homogeneous flows, but not for slip flows. The computation is straightforward and accurate when the stagnation state is saturated. However, subcooled stagnation states sometimes lead to very sharp peaks of G which are difficult to determine accurately. However, static pressure PM at the peak G is quite distinct. It was noted that the sharp peaks in G for some subcooled stagnation states occurred at very low qualities associated with the HEM, and therefore, a frictionless liquid flow rate GBM was obtained from the Bernoulli equation for an incompressible liquid flowing from stagnation pressure to PM. The greater of GBM or GM was employed to obtain Figure 1 in NEDO-21052. Results of this procedure were verified by a number of computations giving pressures and flow rates at 1.0 psi increments to observe the actual GM and PM where the peak occurred. Discrepancies were negligible.

## **REQUEST 7**

Provide the critical flows in Figure 1 in tabular form for subcooled and two-phase critical flows at 10% increments of stagnation quality at the following stagnation pressure in psia: 100., 200., 400., 600., 800., 1000., 1200.

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## RESPONSE TO REQUEST 7

The following tables provide the information requested. Stagnation qualities shown are not all at precise 10% increments, but these values were readily available from the original computations. The quality increments shown should be satisfactory. The symbols are interpreted as:

PO	=	Stagnation Pressure, Psia
HO	=	Stagnation Enthalpy, BTU/LBM
QLI	=	Stagnation Quality
GM	=	Maximum Homogeneous Mass Flux, LBM/S-Ft <sup>2</sup>
PM	=	Pressure at Condition of Maximum Mass Flux, Psia
QLE	=	Quality at Condition of Maximum Mass Flux
GBM	=	Bernoulli mass flux based on liquid flow from
		stagnation pressure to PM, LBM/S-ft2, calculated
		only for subcooled stagnation states.

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NEDO-21052-A PO = 100

STEAM DOME

HO	QLI	GM	PM	QLE
1187.1644	1.0000	206.0080	58.0000	0.96403
1151.6194	0.96000	209.9721	58.0000	0.92711
1116.0744	0.92000	214.1740	58.0000	0.89019
1080.5295	0.88000	218.6385	58.0000	0.85326
1044.9845	0.84000	223.3942	58.0000	0.81634
1009.4395	0.80000	228.4740	58.0000	0.77941
973.8946	0.76000	233.9165	58.0000	0.74249
938.3496	0.72000	239.7670	58.0000	0.70556
902.8046	0.68000	246.0791	58.0000	0.66864
867.2597	0.64000	252.9326	59.0000	0.63197
831.7147	0.60000	260.3960	59.0000	0.59496
796.1697	0.56000	268.5604	59.0000	0.55795
760.6248	0.52000	277.5427	59.0000	0.52093
725.0798	0.48000	287.4887	59.0000	0.48392
689.5348	0.44000	298.6187	60.0000	0.44673
653.9899	0.40000	311.1571	60.0000	0.40963
618.4449	0.36000	325.4134	60.0000	0.37253
582.8999	0.32000	341.8920	61.0000	0.33499
547.3550	0.28000	361.1556	61.0000	0.29781
511.8100	0.24000	384.2080	62.0000	0.26002
476.2650	0.20000	412.4302	63.0000	0.22207
440.7201	0.16000	448.1699	64.0000	0.18396
405.1751	0.12000	495.7559	66.0000	0.14486
369.6301	0.08000	564.0678	69.0000	0.10458
334.0852	0.04000	677.4754	74.0000	0.06202
298.5402	0.00000	989.0438	90.0000	0.00869

## SUBCOOLED

HO	GM	PM	QLE	GBM
298.4355 277.7292 257.1734 236.7462 216.4284 196.2024 176.0516 155.9610 135.9174 115.9095 95.9285 75.9655 56.0130	984.8863	89.5000 75.0000 55.5000 40.0000 28.5000 19.5000 13.0000 8.5000 3.5000 2.0000 1.0000	0.00902	3636.8185 4879.1381 5695.8158 6248.0554 6661.2808 6955.5750 7161.5603 7303.3229 7402.7255 7483.0891 7543.0429 7577.3160

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di.

## STEAM DOME

HO	QLI	GM	PM	QLE
1198.3323	1.0000	404.9911	115,0000	0.95840
1164.6192	0.96000	412.6401	116,0000	0.92263
1130.9062	0.92000	420.7460	116,0000	0.88628
1097.1932	0.88000	429.3486	116,0000	0.84993
1063.4801	0.84000	438,5008	116,0000	0.81357
1029.7671	0.80000	448.2634	116.0000	0.77722
996.0541	0.76000	458.7188	117,0000	0.74314
962.3411	0.72000	469.9450	117.0000	0.70474
928.5280	0.68000	482.0366	117.0000	0.66833
894.9150	0.64000	495.1184	118.0000	0.63205
861.2020	0.60000	509.3398	118,0000	0.59559
827.4889	0.56000	524.8582	118.0000	0.55914
793.7759	0.52000	541.9175	119.0000	0.52265
760.0629	0.48000	560.7505	119.0000	0.48615
726.3499	0.44000	581.7339	120.0000	0.44951
692.6368	0.40000	605.2927	121.0000	0.41278
658.9238	0.36000	632.0075	122.0000	0.37594
625.2108	0.32000	662.6570	123.0000	0.33901
591.4977	0.28000	698.3628	124.0000	0.30198
557.7847	0.24000	740.7081	126.0000	0.26448
524.0717	0.20000	792.0562	128.0000	0.22680
490.3586	0.16000	856.3963	130.0000	0.18893
456.6456	0.12000	940.5868	134.0000	0.14986
422.9326	0.08000	1058.6405	139.0000	0.10992
389,2196	0.04000	1246.1863	149.0000	0.06672
355 5065	0 00000	1677 9457	174 0000	0 01418

## SUBCOOLED

HO	<u>GM</u> .	PM	QLE	GBM
355.4012 334.1763 313.1728 292.3600 271.7132 251.2077 230.8241 210.5445 190.3504 170.2264 150.1582 130.1334 110.1415 90.1748 70.2241 50.2806	1689.6094	176.5000 156.0000 120.0000 91.5000 68.5000 50.0000 36.0000 25.5000 17.5000 11.5000 4.5000 3.0000 1.5000 0.5000 0.5000	0.01265	4744.2828 6439.8882 7545.7720 8355.4823 8973.8619 9431.6510 9775.2076 10042.2253 10250.7159 10398.5106 10519.4758 10586.7201 10664.0382 10729.3969 10729.3969

NEDO-21052-A PO = 400

STEAM DOME

HO	QLI	GM	PM	QLE
1204.5891 1173.3723 1142.1555 1110.9387 1079.7219 1048.5051 1017.2882 986.0714 954.8546 923.6378 892.4210 861.2042 829.9874 798.7706 767.5538 736.3370 705.1202 673.9034 642.6866 611.4698 580.2529 549.0361 517.8193 486.6025 455.3857 424.1689	1.00000 0.96000 0.92000 0.88000 0.84000 0.84000 0.76000 0.72000 0.68000 0.64000 0.64000 0.56000 0.56000 0.48000 0.44000 0.44000 0.36000 0.28000 0.28000 0.24000 0.12000 0.08000 0.04000	800.6125 815.3308 830.8904 847.3849 864.9036 883.5644 903.4964 924.8504 947.8042 972.5743 999.4087 1028.6131 1060.5820 1095.7591 1134.7449 1178.2655 1227.3554 1283.2825 1347.8150 1423.5836 1514.3767 1626.0762 1769.0988 1962.8817 2252.8882 2810.5223	232.0000 232.0000 232.0000 233.0000 233.0000 234.0000 235.0000 235.0000 236.0000 237.0000 239.0000 240.0000 242.0000 242.0000 244.0000 244.0000 245.0000 251.0000 255.0000 255.0000 259.00000 259.00000 259.0000 259.0000 25	0.95057 0.91509 0.87962 0.84439 0.80888 0.77356 0.73818 0.70261 0.66714 0.63161 0.59601 0.56035 0.52464 0.48880 0.45303 0.41701 0.38088 0.34445 0.30806 0.27108 0.23387 0.19643 0.15720 0.11683 0.07446 0.02281
		SUBCOOLED		
HO	GM	PM	QLE	GBM
424.0582	2818.8928	338.5000	0.02176	6130 38

424.0582 401.9112 380.1244 358.6411 337.4160 316.4134 295.6010 274.9535 254.4496 234.0686 213.7919 193.6026 173.4848 153.4242 133.4083 113.4271 93.4720	2818.8928	338.5000 323.0000 259.0000 205.5000 161.0000 124.5000 94.5000 71.0000 52.0000 37.5000 26.5000 18.5000 18.5000 12.5000 8.0000 5.0000 3.0000 1.5000	0.02176	6130.3812 8362.8217 9896.1754 11047.7030 11940.1259 12653.0251 13207.1949 13659.5830 14013.8501 14294.0878 14510.1471 14685.1980 14830.8125 14941.7567 15028.7747 15109.6915
93.4720 73.5348 53.6078		1.5000 1.0000 0.5000	507	15109.6915 15143.1222 15183.1630
			577 1	93

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STEAM DOME

HO	QLI	GM	PM	QLE
1203.6560 1174.3777 1145.0095 1115.8212 1086.5430 1057.2647 1027.9865 998.7082 969.4300 940.1517 910.8734 881.5952 852.3169 823.0387 793.7604 764.4822 735.2039 705.9256 674.6474 647.3691 618.0909 588.8126 559.5344 530.2561 500 9778 471.6996	1.00000 0.96000 0.92000 0.88000 0.84000 0.84000 0.76000 0.76000 0.68000 0.68000 0.64000 0.52000 0.52000 0.48000 0.44000 0.44000 0.36000 0.32000 0.24000 0.24000 0.24000 0.24000 0.20000 0.16000 0.00000	198. 5280 1220. 1341 1242. 8471 1266. 8888 1292. 3870 1319. 5053 1348. 4202 1379. 3380 1412. 5106 1448. 2129 1486. 8056 1528. 7052 1574. 4021 1624. 5324 1679. 8829 1741. 4174 1810. 4236 1888. 5701 1978. 2159 2082. 4625 2206. 0445 2356. 2594 2544. 7552 2793. 7370 3150. 2029 3759. 9063	$\begin{array}{c} 349.0000\\ 349.0000\\ 350.0000\\ 351.0000\\ 351.0000\\ 352.0000\\ 353.0000\\ 354.0000\\ 355.0000\\ 356.0000\\ 357.0000\\ 359.0000\\ 361.0000\\ 363.0000\\ 365.0000\\ 367.0000\\ 367.0000\\ 370.0000\\ 370.0000\\ 373.0000\\ 376.0000\\ 381.0000\\ 385.0000\\ 385.0000\\ 385.0000\\ 385.0000\\ 391.0000\\ 400.0000\\ 412.0000\\ 427.0000\\ 450.0000\\ 492.0000\\ \end{array}$	0.94281 0.90809 0.87357 0.83901 0.80440 0.76975 0.73505 0.70031 0.66552 0.63068 0.59582 0.56086 0.52582 0.49068 0.45545 0.42004 0.38450 0.34882 0.31269 0.27652 0.23977 0.20209 0.16330 0.12323 0.08032 0.03092
		SUBCOOL	ED	
HO	GM	PM	QLE	GBM
471.5836 448.5145 425.9657 403.8434 382.0750 360.6086 339.3981 318.4064 297.6048 276.9675 256.4726 236.1013 215.8333 195.6529 175.5443 155.4945 135.4894 115.5197 95.5771 75.6545 55.7435	3766.8044	495.0000 494.0000 327.0000 262.5000 208.5000 163.5000 126.5000 96.5000 72.5000 38.5000 38.5000 27.5000 19.0000 12.5000 8.5000 3.0000 1.5000 1.0000 0.5000	0.02989 597 194	7060.0314 9690.8485 11537.6085 12932.3896 14033.9479 14923.8848 15647.2990 16236.4663 16716.6492 17109.3225 17434.4019 17688.3306 17901.0759 18082.0593 18208.5132 18338.4065 18429.5818 18517.1230 18554.2065 18599.3794

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P0 = 800

		STEAM DO	IME	
HO	QLI	GM	PM	QLE
1139.3852 1171.8026 1144.2199 1116.6372 1089.0545 1061.4719 1033.8892 1006.3065 978.7238 951.1412 923.5585 895.9758 868.3931 840.8105 813.2278 785.6451 758.0624 730.4798 702.8971 675.3144 647.7318 620.1491 592.5664 564.9837 537.4011 509.8184	1.00000 0.96000 0.92000 0.88000 0.80000 0.76000 0.72000 0.68000 0.64000 0.64000 0.56000 0.52000 0.48000 0.44000 0.44000 0.36000 0.32000 0.28000 0.28000 0.24000 0.20000 0.16000 0.12000 0.00000	1601.8977 1629.9740 1659.5905 1690.8980 1724.0607 1759.2771 1796.7579 1836.7660 1879.6159 1925.6386 1975.2756 2029.0095 2087.4536 2151.3914 2221.6874 $_99.5358$ 2386.4370 2484.3241 2595.8665 2724.6109 2875.8504 3057.2753 3281.5638 3570.6659 3969.5847 4598.1583	467.0000 468.0000 469.0000 470.0000 471.0000 473.0000 474.0000 474.0000 475.0000 478.0000 481.0000 481.0000 487.0000 487.0000 487.0000 500.0000 500.0000 506.0000 517.0000 517.0000 517.0000 551.0000 571.0000 571.0000 596.0000	0.93483 0.90103 0.86719 0.83331 0.79940 0.76555 0.73154 0.69749 0.66351 0.62935 0.59516 0.56090 0.52652 0.49207 0.45748 0.38777 0.35247 0.31720 0.28134 0.24470 0.20770 0.16952 0.12951 0.08760 0.03926
		SUBCOOLE	<u>D</u>	
HO	GM	PM	QLE	GBM
509.6954 485.6426 462.3010 439.5291 417.2208 395.2975 373.6997 352.3750 331.2859 310.4010 289.6915 269.1344 248.7106 228.3993 208.1841 188.0502 167.9807 147.9632 127.9873 108.0446 88.1255 68.2244	4599.8453	646.0000 666.0000 552.0000 453.5000 298.5000 238.5000 188.5000 147.0000 113.0000 85.5000 64.0000 47.0000 33.5000 16.0000 10.5000 7.0000 4.0000 2.5000 1.5000 0.5000	0.03833	7808.9177 10737.3901 12817.4548 14417.1921 15691.9110 16734.9841 17592.6584 18305.6648 18898.5032 19391.3306 19792.3354 20126.7449 20411.4873 20641.9126 20834.6799 20996.6174 21117.1968 21243.0747 21322.3916 21388.4294 21478.9524

		PO = 1000		
		STEAM DOME		
H0 1192.9348 1166.9201 1140.9054 1114.8907 1088.8760 1062.8613 1036.8467 1010.8320 984.8173 958.8026 932.7879 906.7733 880.7586 854.7439 828.7292 802.7145 776.6998 750.6852 724.6705 698.6558 672.6411 646.6264 620.6117 594.5971 568.5824 542.5677	QLI 1.00000 0.96000 0.92000 0.88000 0.84000 0.80000 0.76000 0.72000 0.64000 0.64000 0.64000 0.56000 0.52000 0.52000 0.44000 0.44000 0.36000 0.32000 0.28000 0.28000 0.28000 0.24000 0.24000 0.20000 0.12000 0.08000 0.04000 0.04000 0.04000 0.04000 0.04000 0.04000 0.04000 0.04000 0.04000 0.04000 0.04000 0.04000 0.04000 0.0000 0.00000 0.000000 0.00000000	GM 2012.2927 2046.7310 2083.0151 2121.3189 2161.8434 2204.7964 2250.4569 2299.1033 2351.1008 2406.8358 2466.8086 2531.5530 2601.8217 2678.4101 2762.3641 2854.9411 2957.8119 3073.1270 3203.6497 3353.2249 3527.2164 3733.7677 3985.0953 4302.4026 4725.8312 5349.1533	PM 586.0000 587.0000 589.0000 591.0000 592.0000 593.0000 596.0000 600.0000 603.0000 605.0000 605.0000 612.0000 616.0000 625.0000 637.0000 637.0000 643.0000 643.0000 653.0000 643.0000 675.0000 675.0000 691.0000 712.0000 742.0000 789.0000	QLE 0.92624 0.89316 0.86019 0.82716 0.79395 0.76070 0.72758 0.69436 0.66095 0.62757 0.59407 0.56050 0.52682 0.49300 0.52682 0.49300 0.45905 0.42490 0.39059 0.35591 0.32111 0.28559 0.24989 0.21324 0.17562 0.13650 0.09480 0.04826
		SUBCOOLED		
H0 542.4394 517.2948 493.1074 469.6575 446.7965 424.4142 402.4286 380.7760 359.4042 338.2763 317.3574 296.6183 276.0373 255.5917 235.2636 215.0368 194.8930 174.8193 154.8007 134.8277 114.8884 94.9765 75.0855 55.2095	<u>GM</u> 5354.2654	PM 792.5000 839.5000 701.5000 583.0000 480.5000 393.0000 202.5000 202.5000 122.5000 122.5000 122.5000 122.5000 122.5000 122.5000 37.5000 37.5000 26.5000 18.0000 12.0000 8.0000 5.0000 3.0000 1.5000	0.04728 597	<u>GBM</u> 8416.5233 11617.6445 13882.0126 15650.8838 17074.6096 18255.5742 19233.2727 20047.8584 20738.8613 21317.7761 21800.7817 22211.1865 22554.7144 22835.1482 23076.9878 23287.1355 23458.3445 23592.7053 23714.5371 23816.3879 23917.4829 23961.3743 1 924015.7004
NEDO-21052-A P0 = 1200

		STEAM D	OME	
H0 1184.8117 1160.2950 1135.7783 1111.2616 1086.7449 1062.2282 1037.7115 1013.1948 988.6781 964.1614 939.6447 915.1279 890.6112 866.0945 841.5778 817.0611 792.5444 768.0277 743.5110 718.9943 694.4776 669.9609 645.4442 620.9275	QLI 1.00000 0.96000 0.92000 0.88000 0.84000 0.76000 0.76000 0.72000 0.68000 0.64000 0.64000 0.56000 0.56000 0.48000 0.48000 0.48000 0.36000 0.32000 0.28000 0.24000 0.22000 0.16000 0.12000 0.08000	GM 2431.4695 2472.0484 2514.7522 2559.7791 2607.3369 2657.6737 2711.0918 2767.9049 2828.5217 2893.3443 2962.9444 3037.9084 3119.0055 3207.1339 3303.3925 3409.0895 3526.0767 3656.4971 3803.1829 3970.1049 4162.6888 4388.5272 4659.6316 4995.1085	<u>РМ</u> 707.0000 709.0000 711.0000 712.0000 714.0000 720.0000 720.0000 720.0000 724.0000 727.0000 734.0000 738.0000 738.0000 742.0000 754.0000 754.0000 755.0000 767.0000 767.0000 755.0000 814.0000 814.0000 853.0000	QLE 0.91694 0.88477 0.85253 0.82012 0.78778 0.75538 0.72307 0.69040 0.65791 0.62527 0.59254 0.55972 0.52678 0.49371 0.46039 0.32523 0.32523 0.32523 0.29042 0.25527 0.21889 0.18218 0.14372
596.4107 571.8940	0.04000	5429.0553 6035.7411	\$88.0000 932.0000	0.10223 0.05778

## SUBCOOLED

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H0	GM 6041,4884	932,0000	0.05758	GBM
545.3754		1013.0000		8952.7666
520.2658		852.5000		12370.5387
496.0989		714.0000		14807.3864
472.6645		593.5000		16725.0950
449.8132		490.0000		18279.2239
427.4359		4r1.5000		19565.4343
405.4542		325.5000		20653.4263
383.8055		261.5000		21008.0209
302.4309		163,0000		23004 6526
341.3035		126 5000		23560, 2781
299 6571		96.5000		24036.8718
279.0771		72,5000		24439.7224
258.6352		53.5000		24781.3384
238.3102		39.0000		25064.7664
218.0871		27.5000		25313.6941
197.9493		19.0000		25522.0771
177.8806		13.0000		25692.0505
157.8702		8.5000		25843.0715
137.9053		5.5000		25965.5591
117.9753		3.0000		26163 5260
98.0/54		2.0000		26250 5508
50 2255		0.5000		26308 9785
20.3330		0.0000		20000.0700

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#### **REQUEST 8**

Calculation of break flow using the HEM model is dependent on the stagnation pressure and stagnation enthalpy that occurs at the break. Describe in detail the method by which stagnation pressures and enthalpies will be determined to predict break flows for containment analysis. Provide and justify all equations and assumptions. This discussion should include the blowdown code used to predict flow rates into the containment.

#### RESPONSE TO REQUEST 8

Vessel pressures and enthalpies in the region of a broken pipe are determined by a nodalization of vessel internal compartments for which mass, energy, and state analyses are performed in such programs as CIPT, LAMB, and SAFE, for which the equations already have been justified. The computation of blowdown flow rate is done by tabular interpolation of stagnation pressure and enthalpy occurring in that node directly adjacent to the blowdown flow path.

#### REQUEST 9

Discuss the effect of break size on the critical flow rate and justify extrapolation of small scale test results to reactor conditions.

#### **RESPONSE TO REQUEST 9**

The data of Sozzi and Sutherland in NEDO 13418, Figure 10, and Allemann, Ref. 14. of NEDO-21052, show that increased pipe diameters tend to give lower critical mass fluxes. Although reasons for this behavior are not fully understood, it is expected to be largely caused by two-dimensional effects both in the vessel and the pipe, which are amplified in larger scale. The one-dimensional HEM agrees best with small pipe data and

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tends to overpredict larger pipe data. Therefore, one should predict conservative blowdown rates from large pipes when using the one-dimensional HEM.

#### REQUEST 10

NEDG-21052 discusses two critical flow models; the homogeneous equilibrium model and a modified slip-flow model. Discuss the manner by which you intend to apply these two models for containment analysis. In addition, since the HEM model is limited to pipe lengths greater than four inches and less than an equivalent fl/d of three, discuss any limitations on the application of the HEM model to the Mark I, Mark II, or Mark III containment designs with regard to break location assumptions.

#### RESPONSE TO REQUEST 10

The method proposed to employ the HEM and slip models of NEDO-21052 is this:

The maximum discharge mass flow rate should be determined with the HEM based on vessel stagnation properties near the discharging flow passage for pipes of fL/D less than approximately 3.0. There is no further need to determine slip flow properties at discharge since slip properties will not alter the upstream homogeneous choked conditions. For fL/D greater than about 3.0, the slip model described in APED 4827 should be employed for discharge rates. (The value of fL/D = 3.0 is approximately where a pipe friction model based on slip flow would unchoke the entrance homogeneous condition.) Postulated large pipe breaks in a BWR involves flow paths of sufficient length to establish phase equilibrium, and thus lend themselves to the HEM for a determination of the maximum discharge rate.

LDS: csc/20L2

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### SUPPLEMENTARY INFORMATION

NRC request for this information was contained in a letter from D. G. Eisenhut (NRC) to L. J. Sobon (GE) dated January 30, 1978

GE response with the following information was provided in a letter from L. J. Sobon (GE) to D. G. Eisenhut (NRC) dated June 30, 1978

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#### NRC QUESTIONS

## REPLY TO NRC REQUEST

## FOR ADDITIONAL INFORMATION ON NEDO-21052

Reference:

NRC Letter Dated January 30, 1978, D. G. Eisenhut to L. J. Sobon

#### REQUEST 1(a)

Identify the computer code that will be used to calculate the break flow. Provide documentation or available references which give all of the equations and assumptions used in the code. Where appropriate, provide specific page numbers for the reports referenced.

#### RESPONSE 1(a)

The computer code used to calculate breakflow for containment analysis is the M3CPT03 code which uses the technology described in NEDO-20533. The pressure vessel model used in these calculations is described in Section 2 (page 2-1 to 2-8) and the details of the pipe inventory model are contained in Appendix B (page B-1 to B-14).

The following discussion provides details of the methods and assumptions used to calculate the short term mass and energy release to the Mark I primary containment for the Design Basis Accident (DBA) recirculation line break.

#### I. Background and Definition

Following a postulated instantaneous double-ended guillotine break at the safe-end to pipe weld on the suction side of the recirculation system, fluid discharges from both broken ends at a rate which is choked by the break areas. The initial mass flux is determined from the initial reactor pressure, the subcooled liquid enthalpy in the recirculation system and the Moody Homogeneous Equilibrium Model (HEM). After the initial blowdown periods, the two break flows make step changes to flows which are choked by the minimum flow area in each broken section. During the period of time necessary to deplete the initial subcooled liquid within the jet pump downcomer region and within the broken recirculation loop, the break fluid enthalpy is assumed to remain at the initial value. After the initial subcooled inventory is depleted, the break flow becomes saturated and discharges at the critical flow rate specified by the HEM model when evaluated at the reactor vessel pressure. During the entire blowdown period, the mass and energy release is calculated assuming HEM critical flow from two sections of straight, frictionless pipe. The following procedure summarizes the steps for determining the initial blowdown periods for the broken sections and for determining the subcooled liquid inventory depletion time.

#### II. Methods

#### A. Initial Mass Flow Rates

The initial critical mass flux is a specified fraction of the final (steady state) critical mass flux at a given enthalpy and pressure. This ratio of initial-to-final critical mass flux, at any given pressure, is a function of the degree of fluid subcooling at that pressure. The calculational procedure for determining the initial critical flow from each section takes the conservative approach of using a steady state critical mass flux multipler of 0.72, the HEM initial-to-final critical

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mass flux ratio for saturated liquid at 1000 psia. The initial break mass flux is based upon the reactor vessel steam dome pressure and the core inlet fluid enthalpy at 102% of the maximum licensed core thermal power.

Pipe Side Initial Mass Flow Rate 1)

M<sub>1.1</sub> = F x G<sub>M</sub> x A<sub>BR</sub> Where F = steady state flow multiplier (ratio of initial to quasi - steady flow) = 0.72 for subcooled liquid flow (HEM) G<sub>M</sub> = HEM critical mass flux, lbm/sec-ft<sup>2</sup>  $A_{RD}$  = Pipe flow area at break location, ft<sup>2</sup> Safe-End Side Initial Mass Flow Rate 2) M<sub>1.2</sub> = F x G<sub>M</sub> x A<sub>BR</sub> Total Initial Mass Flow Rate

3)

 $\dot{M}_{I, Total} = \dot{M}_{I,1} + \dot{M}_{I,2}$ 

Initial Mass Flux Duration Β.

> Pipe Side Initial Mass Flux Duration 1)

> > For jet pump plants (BWR/3&4), the flow rate from the downstream section (pipe side) of the broken recirculation loop is eventually limited by the total jet pump nozzle flow area. Since the total jet pump nozzle flow area is significantly less than the break area, the final choked flow through the jet pump nozzles is much smaller than the initial choked flow through the pipe break. To account for initial/final blowdown effects and the limiting restriction of the jet pump nozzles, it is conservatively assumed that the break flow is limited by the pipe area at the break location until the initial fluid inventory

in the broken recirculation loop is totally depleted. At that time the break flow rate drops to the lower value determined by the total jet pump nozzle flow area.

For plants (BWR/2) not utilizing jet pumps, the duration of the initial critical flow is the time it would take for an unobstructed acoustic wave to travel twice the length of the downstream section of broken pipe (a round trip from the break location to the vessel and back). This acoustic reflection takes approximately 50 milliseconds.

After this period of time, the break flow rate increases (step change) to a critical flow rate determined by either the break area or the discharge side safe-end nozzle flow area, whichever is limiting.

Pipe Side Initial Mass Flux Duration

a) If  $A_{L1}/A_{BR} < F$ 

Then  $t_1 = \frac{M_1}{F \times G_M \times A_{BR}}$ 

(BWR 2)

(BWR 3/4)

- b) If  $A_{L1}/A_{BR} \ge F$ 
  - Then  $t_1 = \frac{2L_1}{C}$

where M<sub>1</sub> = Initial fluid mass occupying the discharge side broken section, 1bm.

A<sub>L1</sub> = Limiting flow area on discharge side broken section, ft<sup>2</sup>

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L, = Length of discharge side section, ft.

C = Sonic speed, ft/sec.

2) Safe-End Side Initial Mass Flux duration

The period of initial blowdown from the upstream, or safe-end, section of the broken line is also determined by the simplified acoustic model described previously. The initial blowdown period is assumed to be the length of time it would take for an acoustic wave to reflect off the vessel penetration.

Safe-End Side Initial Mass Flux Duration

If  $A_{L2}/A_{BR} \ge F$ then  $t_2 = \frac{2L_2}{C}$ 

During this initial period, the break flow remains subcooled and the mass flux,  $G_{H}$ , is evaluated based on the reactor vessel pressure and the subcooled enthalpy.

C. Subsequent Mass Flow Rates

Following the initial blowdown periods, the break flows are choked by the limiting flow area in each section. In jet pump plants where the Reactor Water Cleanup (RWCU) system has a pipe which is common to both the broken recirculation loop and the intact loop, there exists a second critical flow path in the pipe section downstream of the break location. The RWCU system penetrates the recirculation loops between the safe-end break and the jet pump nozzles. Since the total jet pump nozzle flow area is small compared to the recirculation line area, the postulated break area will accommodate the additional RWCU pipe flow from the pressurized unbroken recirculation loop. For the plants with the above mentioned RWCU piping, the smallest flow area in the connecting RWCU line is added to

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the total jet pump nozzle area to obtain the total final critical flow area for the discharge section (pipe side) of the broken loop.

1) Pipe Side Final Mass Flow Rate

 $M_{F,1} = G_M \times (A_{L1} + A_{RWCU})$ 

where A<sub>RWCU</sub> = Limiting area in the RWCU system pipe common to both recirculation loops, ft<sup>2</sup>.

2) Safe-End Side Final Mass Flow Rate

 $M_{F,2} = G_M \times A_{L2}$ 

3) Total Final Mass Flow Rate

 $\overset{\bullet}{M}_{F,Total} = \overset{\bullet}{M}_{F,1} + \overset{\bullet}{M}_{F,2}$ 

D. Subcooled Liquid Depletion Time

To account for the subcooled liquid initially present within the jet pump downcomer region surrounding the core shroud and within the broken recirculation loop, a subcooled inventory depletion time is calculated considering critical mass flow rates based upon initial reactor conditions. During this subcooled depletion period, M3CPT03 assumes that the break fluid remains (MGC) at the initial break enthalpy.

$$H_{sc} = t_{1} + \frac{H_{sc} - [M_{I,1} t_{1} + M_{I,2} t_{2} + M_{F,2} (t_{1} - t_{2})]}{M_{F,1} + M_{F,2}}$$

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M<sub>sc</sub> = Initial mass of liquid in the jet pump downcomer region and in the broken loop.

## E. Saturated Blowdown From Reactor Vessel

Following the depletion of the subcooled liquid inventory, the break flow changes to critical saturated flow governed by transient reactor conditions. This saturated blowdown continues until break uncovery, at which time the break discharge becomes two phase flow.

NOTE: The above response indicates that plants without jet pumps (BWR/2) have slightly different mass flow rate characteristics for the design basis recirculation line break than plants with jet pumps (BWR/3,4). However, during the preparation of plant unique containment response analyses for the Mark I Containment Program, the BWR/2 plants were found to include flow restricting devices (venturis) in the discharge side of the recirculation piping. With these restrictors, the general mass flow rate characteristics for a design basis recirculation line break at the safe end to pipe weld are the same for plants with or without jet pumps, i.e., the flow rate drops from the initial value based on the pipe break area to a lower value determined by the jet pump nozzle flow area (BWR/3,4) or by the restrictor flow area (BWR/2).

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#### REQUEST 1(b)

The discussion in NEDO-21052 indicates that the use of the HEM model is appropriate to calculate choking at the entrance of pipe sections upstream of full area breaks and would not be appropriate for orifice type breaks. Describe the manner by which split breaks, limited off-set breaks, and similar break configurations, which approximate an orifice geometry, will be analyzed.

#### RESPONSE 1(b)

The drywell pressurization rate and containment response due to an instantaneous double ended guillotine break at the safe end of the recirculation piping is bounding for Mark I plants. Therefore, no analyses are performed for drywell pressurization rate or containment response which involve breaks having areas smaller than a double-ended guillotine break.

#### REQUEST 1(c)

Describe the noding arrangement that will be used in the vessel and reactor piping. Discuss the manner by which the conditions determined from this noding arrangement will be used to establish the break flow rate. Discuss whether the HEM flows will be calculated by internal programming or by external calculation.

#### RESPONSE 1(c)

The M3CPT03 vessel model uses a single node which is assumed to be at saturated conditions. The code establishes break flow using a table lookup at the vessel pressure and enthalpy. To handle subcooled conditions, an enthalpy is inserted which overrides the computed vessel enthalpy. The magnitude of this subcooled enthalpy input is determined from plant steady state energy balances. The duration of subcooled liquid blowdown is based on the time it takes the mass of subcooled liquid in the recirculation system piping and the downcomer region around the jet pumps to be discharged at the inital break flowrate. Once the subcooled liquid is depleted the break flow is calculated based on the saturated properties in the vessel.

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#### REQUEST 1(d)

Discuss the manner by which the break flow rate will be determined during pipe decompression for short-term mass and energy releases (0 to 2 seconds). Justify the application of the model during this time period giving due consideration to the subcooled nature of the fluid and the probable location of the choking plane at the break rather than the pipe entrance. These conditions may cause the fluid to be in a nonequilibrium state. This justification should be supported by comparison with staff-approved analytical models or applicable experimental data.

#### RESPONSE 1(d)

For the time that it takes the initial inventory to clear from the pipe side of the recirculation system break, the flow is taken as a fraction of the HEM flow corresponding to vessel pressure, the subcooled enthalpy of the fluid in the line and the pipe area at the break location. For liquid breaks (applicable to Mark I), this fraction is 0.72.

For the short side of breaks at the vessel safe end, the fraction 0.72 is also used, but only for the time that it takes a sonic wave (with velocity  $\sqrt{5000}$  f/s) to travel from the break to the vessel and back to the break.

The conservatism of the 0.72 multiplier combined with the HEM flow model has been verified by comparison of M3CPT03 predicted mass and energy release using these assumptions to RELAP calculations using the Henry-Fauske-Moody flow model. These results are described below.

# I. Mass Release Rates and Break Flow Specific Enthalpies Calculated Using RELAP

Utilizing the data listed in Table 1, a RELAP run was made to calculate the break flow rates and the corresponding fluid enthalpies for 50 seconds following the postulated break. The reactor pressure vessel was modeled after a 218 BWR/6 standard vessel and the recirculation line geometry was that of a typical 218 BWR/5. The RPV and recirculation lines were modeled to account for frictional and geometric losses. Figure 1 shows the nodalization scheme used in the RELAP analysis. Figures 2 through 5 show the break mass flow rates and the corresponding break fluid enthalpies. The results shown in Figures 2 and 3 indicate that the jet pumps uncover at approximately 8 seconds and the safe-end side of the break uncovers at approximately 11 seconds. When uncovery occurs, the break flow quality rapidly increases, as shown in Figures 4 and 5. When the breaks uncover, there are significant reductions in mass and energy release rates, due to the high quality two-phase break flow, accompanied by a rapid depressurization of the vessel.

# II. Mass Release Rates and Break Flow Specific Enthalpies Calculated Using M3CPT03/HEM

Mass release rates and break fluid enthalpies were calculated out to 50 seconds by the containment code M3CPT03, using the HEM critical flow option. The RPV (218 BWR/6) and the recirculation loop (218 BWR/5) input data used in the analysis were the same data (Table 1) used in the comparative RELAP analysis. The method and assumptions used in the M3CPT03 analysis are described in Response 1(a). The results of the analysis are shown in Figures 6 and 7. Figure 6 shows that the recirculation line initial fluid inventory is depleted approximately 1.2 seconds following break occurrence. The break flow rate from the pipe side of the break is then limited by the total jet pump nozzle flow area. The RWCU line flow area was not included in these comparative analyses. MSIV closure, starting at 0.5 seconds,

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causes a vessel pressurization which produces an increasing break flow rate until the subcooled liquid inventory is depleted at approximately 10 seconds. The break discharge then changes to critical saturated liquid flow governed by transient reactor conditions. The break uncovers at about 20 seconds and the break flow becomes two-phase for the remainder of the transient. After break uncovery, the vessel rapidly depressurizes. Figure 7 shows the changes in break-specific energy corresponding to the changes in break flow described previously.

## III. Integrated Mass and Energy Release Rates From RELAP and M3CPT03/HEM Analysis

The mass release rates and the corresponding break fluid specific enthalpies given in Figures 2 through 5 for RELAP and given in Figures 6 and 7 for M3CPT03/HEM were integrated and the results are shown in Figures 8 and 9. The results show that the total energy release calculated by M3CPT03 is approximately 17 percent higher than the total energy release calculated by the RELAP code. The integrated mass release rates are  $4.71 \times 10^5$  lbm for M3CPT03 and  $4.08 \times 10^5$  lbm for RELAP. The energies that would be released to the primary containment over 50 seconds are  $2.84 \times 10^8$  BTUs for M3CPT03 and  $2.43 \times 10^8$  BTUs for RELAP.

## IV. Applicability of BWR/5 Comparison to BWR/2,3 and 4

The BWR/5 recirculation loop model and the BWR/6 vessel model used in the RELAP analysis are essentially the same as would be used in a similar study of the BWR/3 or BWR/4. There are minor hardware differences such as the number of nozzles in each jet pump, the flow control mechanism and the type of recirculation flow rate measuring device. However, these hardware differences would not impact a RELAP comparison to M3CPT03 because they would not affect the location of the limiting critical flow areas. The flow control

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valve was a minimum of 90 percent open during the reported RELAP analysis and the venturi flow nozzle in BWR/3s and 4s would not represent limiting flow areas in a similar study. In other words, the break flow rates from both broken sections would choke at the same locations (jet pump nozzles and suction side safe-end nozzle) in a BWR/3 or 4 RELAP analysis as in the reported analysis.

There are recirculation loop dimensional variations in BWR/3, 4 and 5. The BWR/5 recirculation loop in the reported study was approximately 100 feet in length from the suction side safe-end nozzle to the vessel penetration at the jet pump end of the loop. The corresponding lengths of piping in BWRs/3 and 4 vary from 106 feet to 119 feet. The BWR/5 recirculation line flow area at the break location was 1.755 ft<sup>2</sup>. In BWRs/3 and 4, the corresponding areas vary from 2.24 ft<sup>2</sup> to 3.67 ft<sup>2</sup>. In the reported analysis, the total jet pump nozzle flow area in each recirculation loop was 0.35 ft<sup>2</sup>. In BWR/3 or 4, the same area ranges from 0.48 ft<sup>2</sup> to 0.78 ft<sup>2</sup>

These dimensional variations would have no impact on a comparative study of RELAP and M3CPT03/HEM because the increases in critical flow areas would have the same relative impact on each code. The critical blowdown rate would increase in direct proportion to the increase in choking area and a comparison of RELAP and M3CPT03 should look relatively the same as the reported comparison.

The vessel initial subcoooled liquid enthalpy used in the reported analyses is approximately the same as the typical core inlet enthalpy for a BWR/3 or 4. Once again, any variation in initial vessel liquid enthalpy would have the same relative effect on the blowdown calculations for both codes.

The noted differences between a BWR/5 and BWRs/3 and 4 would not affect a comparison of RELAP to M3CPT03 since the variations would have the effect of increasing or decreasing the mass and energy

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release in approximately the same proportion for each code. Therefore, on a comparative basis, the reported results should be applicable to BWRs/3 and 4.

Although the BWR/2 plants do not have jet pumps\*, the conclusions made above for BWRs/3 and 4 plants are also valid for BWR/2 plants. If a comparative analysis were made between RELAP and M3CPT03/HEM for a BWR/2 plant, one final limiting critical flow area would result in the broken pipe at some location between the break and the pipe connection to the vessel. M3CPT03 considers only two locations, the pipe area at the break location and the safe-end nozzle at the inlet to the vessel. M3CPT03 uses the smaller of these areas as the final limiting choked flow area. The pipe side initial blowdown period is on the order of 50 milliseconds and, therefore, has no significant impact on M3CPT03 results.

If the pipe area at the break location is the limiting area, there is no reduction in break pressure because M3CPT03 assumes the pipe to have no pressure drop between the vessel and the break location. If the inlet safe-end is limiting, the blowdown pressure will be the same as in the previous case, i.e., the vessel pressure.

In RELAP analysis, it is possible that there would be a choking point somewhere between the inlet safe-end and the break location because RELAP models the variations in geometry (pump, etc.) and friction. If this were the case, the calculated blowdown rate would be less than it would be if calculated at the inlet safe-end because of pressure drop considerations and reduced critical flow area. Also, if RELAP break flow choked at the break location, the critical mass flux would be reduced due to pressure drop considerations. Therefore, compared to M3CPT03, the highest break flow rates that RELAP could calculate would result from choking the final flow rate at the same locations as does M3CPT03. Since, in the BWR/5 analysis using M3CPT03, the calculated blowdown rates were higher than those

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\*See Figures 10 and 11 for piping schematics of BWRs/2,3,4 and 5.

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calculated using RELAP when choking the flow at the same locations for both models, it can be concluded that M3CPT03 would also produce higher mass and energy release rates from the pipe side of a BWR/2. From a comparative standpoint, an M3CPT03/HEM analysis should produce even more conservative results in the case of a BWR/2 than it did in the reported BWR/5 analysis.

#### V. Conclusions

The reported analysis and the applicability of the comparison to BWRs/2,3 and 4 shows that the method utilizing M3CPT03/HEM for calculating the short-term mass and energy release to the Mark I containment for the JBA would be bounding and conservative relative to the reported RELAP analysis of any Mark I plant. The primary cause of this conservatism is the behavior of the pressure vessel. The standard M3CPT03 analysis forces the breakflow to remain all liquid until 80% of the inventory is depleted. This causes the vessel pressure to remain high. In RELAP, two phase mixture starts to flow when flashing starts to occur in the liquid adjacent to the break. This results in a more rapid depressurization rate. The conclusion is that the whole analysis using HEM and M3CPT03 is conservative relative to the RELAP analysis using the Henry-faushe-Moody flow model. The same relative behavior is expected for all BWRs/2,3 and 4.

NOTE: The above response indicates that plants without jet pumps (BWR/2) have slightly different mass flow rate characteristics for the design basis recirculation line break than plants with jet pumps (BWR/3,4). However, during the preparation of plant unique containment response analyses for the Mark I Containment Program, the BWR/2 plants were found to include flow restricting devices (venturis) in the discharge side of the recirculation piping. With these restrictors, the general mass flow rate characteristics for a design basis recirculation line break at the safe end to pipe weld are the same for plants with or without jet pumps, i.e., the flow rate drops from the initial value based on the pipe break area to a lower value determined by the jet pump nozzle flow area (BWR/3,4) or by the restrictor flow area (BWR/2).

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## TABLE 1

## Reactor Pressure Vessel and Recirculation Loop Data and Initial Conditions (RELAP and M3CPT03)

Initial Reactor Power	2894. MWt
Initial Vessel Dome Pressure	1040. psia
Initial Core Inlet Fluid Enthalpy	527.5 BTU/1bm
Initial Feedwater Enthalpy	399. BTU/1bm
Initial Mass of Liquid in RPV	399,180. 1bm
Initial Mass of Vapor in RPV	17,180. 1bm
Initial Mass of Liquid in Each Recirculation Loop	129,130. 1bm
Initial Steamline Flow Rate	3459. 1bm/sec
Mass of Passive Heat Slabs	1,895,564. lbm
Total Fuel Bundle Heat Transfer Area	61,151. ft <sup>2</sup>
RPV Inside Diameter	18.17 ft.
RPV Height	69.31 ft.
Recirculation Line Flow Area (At Break)	1.755 ft <sup>2</sup>
Total Jet Pump Nozzle Flow Area in Each Recirculation Loop	0.348 ft <sup>2</sup>
Decay Heat	ANS-5

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218 BWR/S RECIRC LINE BREAK

# FIG.I RELAP MODEL



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FIG.3 PIPE SIDE BREAK FLOW

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FIG.5 PIPE SIDE BREAK ENERGY







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#### REQUEST 1(e)

The comparisons of the HEM model with data presented in NEDO-21052 indicate that, for many conditions, the HEM model provides a best estimate rather than a conservative estimate of the break flow rate. Therefore, justify the use of the HEM model as a design tool, for the stagnation pressure and quality range of interest.

#### RESPONSE 1(e)

There are at least two design philosophies which start at the common basis that some margin of conservatism is essential in nuclear containments.

One philosophy introduces conservatism at all levels of computation or data interpretation so that final design requirements are based on "conservatisms built on conservatisms". Designs based on this philosophy have excessive conservatisms and can be criticized only with regard to economics.

The second philosophy employs the most accurate prediction possible for all interconnected phenomena, finally arriving at an overall "best estimate" for each design parameter. Known degrees of conservatism then are introduced which insures a safe, regulated, design within the framework of competitive economics.

Justification of the HEM as a best estimate design tool becomes one important link in the second philosophy, and represents a desirable step toward improved containment design procedures.

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#### **REQUEST 2**

Provide a comparison of the break flow as a function of time for a postulated double-ended recirculation line break using both the HEM model and the 1965 Moody frictionless slip flow model in typical BWR/4 plant. The break flow rate should be provided for the first 50 seconds following rupture. Identify the computer code and noding arrangement used in the analysis.

#### RESPONSE 2

Table 2 presents a comparison of the break flow rates calculated as a function of time using both the slip and HEM models. These calculations were performed for a plant with the typical BWR/5 recirculation system evaluated for Response 1(d). As the discussion in Response 1(d) indicates, the mass and energy release from a break in a BWR/5 is representative of the response for BWRs/2,3 and 4.

The M3CPT03 computer code was used to perform these calculations. The noding arrangement used by this model is described in Response 1(c).

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	HO	M	OG	EN	EO	US		EQ	UI	L	IE	3R	I	UN	4	M	OD	E	L	B	L	OW	D	OW	N	
		F	10	W	RA	TE	S	V	S	M	00	DD	Y	5	SL	1	P	F	LC	W		RA	T	ES	£.,	
F	OR	1	A	21	8	BW	R	- 5	R	E	CI	R	C	UL	A	T	10	N	L	.1	N	Ε	B	RE	AK	

		HEM		FLOW MODEL		
Time	Blowdown Flow	Rates (1b/sec)	Time	Blowdown Flow	Rates (1b/sec)	
(sec)	Liquid	Steam	(sec)	Liquid	Steam	
0. T 0.00175	21700	0.0	0. T 0.00175	15800	0.0	
0.00175 T 1.192	26000	0.0	0.00175 T 1.62	9 23700	0.0	
1.192	18080	0.0	1.629	18900	0.0	
5.000	20400	0.0	5.000	20600	0.0	
9.808	24600	0.0	9.390	24300	0.0	
9.808	12600	0.0	9.390	17800	0.0	
20.48	12800	0.0	17.515	17900	0.0	
20.49	5020	2610.	17.523	6820	3410.	
30.0	2940	2080.	20.0	5780	3210.	
40.0	1840	1130.	30.0	2440	2030.	
50.0	1280	578.	40.0	1480	855.	
			50 0	1880	170	

Bases:

- Bases:
- Inventory Flow Multiplier = 0.72
- Degrad of subcooling corresponds to 2004 Mwt.

1) Inventory Flow Multiplier = 0.50

 Degree of subcooling corresponds to 2894 Mwt.

#### REQUEST 3

It appears that the mass flow rates experimentally determined by Zaloudek ("Steam-Water Critical Flow from High Pressure Systems", HW-80535, January, 1964) and Sozzi and Sutherland ("Critical Flow of Saturated and Subcooled Water at High Pressure", NEDO-13418, July, 1975) are in excess of those calculated by the HEM model when the stagnation conditions approach saturation. Justify the application of the HEM model in this regime.

#### **RESPONSE 3**

Comparisons with data of Sozzi and Sutherland shown in NEDO-21052 confirm the best estimate nature of the HEM for both saturated and subcooled stagnation conditions.

The particular Zaloudek data, Figures 2 through 5 of HW-80535 is attached. The solid curve labeled HEM was obtained from Figure 1 in NEDO-21052, employing the values of stagnation enthalpy and upstream (stagnation) pressure given.

It is seen that the HEM gives accurate predictions of all the data presented for both saturated and subcooled stagnation states.



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## NRC STAFF TOPICAL REPORT EVALUATION

(Enclosure to December 27, 1978 letter from D. Eisenhut (NRC) to L. J. Sobon (GE))

ENCLOSURE

## TOPICAL REPORT EVALUATION

REPORT NO.: NEDO - 21052

### REPORT TITLE: Maximum Discharge Rate of Liquid-Vapor Mixtures from Vessels

REPORT DATE: September, 1975 ORIGINATING ORGANIZATION: General Electric Company

REVIEWED BY: Analysis Branch, DSS

#### SUMMARY OF TOPICAL REPORT

In topical report NEDO-21052 and the additional information provided November 8, 1977, and June 30, 1978, GE has provided a proposed method for predicting break flow for use in containment response analyses. The containment response analysis is used directly in the load combinations for the containment structural assessment and it establishes the boundary conditions for the suppression pool hydrodynamic testing program for plants with Mark 1 containments.

Application of the break flow methods will be limited to double-ended break sizes in the recirculation piping of plants with Mark 1 containments. These are BWR-3's and 4's with jet pumps and BWR-2's without jet pumps. For containments of the Mark 1 design, the first second of blowdown is significant since this is the time period when the vents from the drywell to the suppression pool are clearing. At this time, the maximum structural pool loads are experienced. The firstten seconds of flow are also important since the peak drywell pressure is reached at the er.' of this period. The pressure response of a typical Mark; 1 containment to a postulated recirculation line break is given in Figure 1.

Topical Report NEDO-21052 provides a comparison of the homogeneous equilibrium critical flow model (HLM) with experimental test data. The model was developed using the assumption that the flow process is isentropic and the report provides curves of mass flux as a function of the stagnation enthalpy and pressure. The flow rates are essentially identical to the HEM flow tables contained in the RELAP-4 computer program. For pipes longer than four inches, with low friction, GE concludes that HEM provides a best estimate for prediction of critical flow rates.

A slip flow model is also developed in NEDO-21052 for use with long pipes, but this model will not be used for the prediction of flow rates in the Mark 1 test program. Therefore, this model is not considered in this topical report evaluation.

The additional information provided November 8, 1977 includes tables of HEM flow rates calculated by GE and additional justification for use of the HEM based on experimental test data comparisons.

The information provided June 30, 1978 discusses application of HEM in the M3CPT03 single node blowdown code and provides a comparison of the break flow predicted by M3CPT03 to that predicted by RELAP-4 for a typical BWR with jet pumps. Since the one node M3CPT03 code does not consider local pressure variations when computing the flow from the broken recirculation piping, GE multiples the initial break flow calculated using HEM by a factor of 0.72 for the initial pipe decompression period. The basis for this factor is provided in topical report MEDO-20533 (Ref. 1) and is derived from solution of the mass, energy and momentum conservation equations assuming isentropic flow. The pipe decompression period is about 50 milliseconds for pipes without a restriction between the break and the vessel, and is determined by the time required for a sonic pressure wave to traverse the distance between the break and the vessel and back. The sonic velocity at these conditions is approximately 5000 feet/second.

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For BWR-3's and 4's, the jet pump nozzles provide a large flow restriction at the vessel inlet nozzle. For these plants, the 0.72 factor is utilized for the time required to exhaust the pipe fluid inventory between the break and the jet pump nozzles (about one second). The flow rates during the pipe decompression periods for each side of the break are assumed constant and determined from the HEM tables (flow vs stagnation enthalpy and initial pressure).

Following the initial pipe decompression period, flow rates are determined by HEM, which is programmed into the M3CPT03 code. The code assumes a constant input subcooled enthalpy until the initial subcooled mass of water in the vessel is depleted. After this period, flow rates are determined using the stagnation liquid enthalpy and pressure calculated by M3CPT03. The break flow is assumed to be all liquid until the reactor system inventory is 80% exhausted. Since critical flow rates for liquids are larger than those for two-phase mixtures, these assumptions act to maximize the release to the containment. The switch to two-phase flow is made at about 20 seconds into the transient which is well beyond the times of peak drywell pressure and pressurization rate.

#### STAFF EVALUATION

General Electric has presented the homogeneous equilibrium model as a best estimate calculation to be used as part of a method for predicting break flows. They propose to introduce conservatism by use of the nonmechanistic one node blowdown model. In our evaluation, we consider both the comparisons of HEM to available experimental test data, and the application of HEM with GE's methodology to assess the overall conservatism.

### A. Verification of HEM Flow Rates by Comparison with Experimental Data

The GE justification for use of HEM in predicting break flow is based primarily on the data of Sozzi and Sutherland presented in Ref. 2. These experiments involved the blowdown of a vessel through various nozzles of varying length and diameter. The effect of increased nozzle length was found to decrease the flow rate. A large sensitivity was observed for nozzles less than four inches in length and a smaller sensitivity was observed for nozzles greater than four inches in length. GE attributes the large sensitivity of short pipes to the non-equilibrium condition of the fluid at the point of discharge. For pipe lengths longer than four inches, they conclude that the fluid will have the opportunit, to reach equilibrium before leaving the test section so that the flow rates could be predicted by HEM. Pipe lengths longer than four inches would reduce the flow rate only by the reduced stagnation pressure resulting from the increased frictional pressure drop.. Flow rates predicted by HEM were found to agree with the Sozzi and Sutherland data in Ref. 2 for pipes longer than four inches. The HEM model was also compared in NEDO-21052 to data taken by Uchida, Fauske, Henry, Allemann and Zaloudek. These comparisons also showed that general agreement was obtained for pipe lengths longer than four inches. Most of the data were for small diameter pipes of less than one inch ID. The Allemann data, however, included pipes up to 6.8 inches ID and also showed agreement with the HEM predictions.

The effect of nozzle diameter on break flow was evaluated by Sozzi and Sutherland for pipes less than one inch in diameter and 1.75 inches in length. These results indicated that mass flux decreases as diameter increases. Simon (Ref. 4) evaluated the effect of both length and diameter for nozzles of four inches and smaller. The results are presented here as Figure 2. In these studies, a complex relationship was observed on the effect of both nozzle length and diameter on the break flow. The flow rate was observed to either increase or decrease with increased nozzle diameter as a function of the nozzle length.

These studies indicate that small pipe data may not necessarily be applicable for predicting flows from large diameter pipes. The recirculation line area for plants with MARK 1 containments range from 2 to 4 ft.<sup>2</sup> while most of the test data is for pipe diameters in the order of a few inches.

Critical flow data for large area pipe sections from 1 to 2 ft.<sup>2</sup> are currently being obtained at the Marviken facility (Ref. 5). Preliminary comparisons of the HEM with data from the first two tests have been made by our consultants at the Brookhaven National Laboratory. Comparison curves are attached as Figure 3 and 4. These figures indicate that HEM underpredicts the data by as much as 40%. The results indicate equilibrium conditions may not be reached for large diameter pipes as was observed by Sozzi and Sutherland for small diameter pipes.

In one location the flow length that is available for choking in the BWR-3 and 4 type plants does not appear to be sufficient to produce equilibrium conditions even for pipes of small diameter. The jet pump nozzles provide a reduction in flow area resembling the geometry of an orifice. For orifices, the data of Sozzi and Sutherland indicate flows in excess of HEM. This is because the short transit time through the test section does not permit steam bubbles to form sufficiently for the equilibrium state to be reached. The

fluid is consequently discharged at a lower quality and higher density than would be predicted by equilibrium theory, and mass flow rates in excess of HEM are measured. For sharp-edged orifices, flows about 150% larger than HEM were measured for saturated water at 1000 psi.

Orifice flow data obtained by Silver, Bailey, and Schrock were compared by Collins in Ref. 3 to the predictions of HEM. For flow of saturated water, the data was observed to be about 150% larger

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#### than HEM values.

Another experimental data comparison was made by Simon in Ref. 4 utilizing data taken by Uchida, Fauske, Friedrich, Burnell, Forster and Esthemer. For flow of saturated water through an orifice at 1000 psi, flows 150% larger than HEM were also observed. Flow rates were found to decrease as the nozzle lengths increased and converge on HEM for lengths of about eight inches.

The available experimental data indicates that HEM may significantly underpredict flow rates through the jet pump nozzles since they resemble an orifice. However, the jet pump nozzles represent only 20% of the total flow area, and would not produce a major portion of the total break flow.

### B. Application of HEM of Prediction of Break Flows

Following a double-ended pipe break, the sudden discharge of fluid will produce a decompression wave which travels down the pipe to the vessel. If the pipe is open to the vessel, a compression wave will be produced at the vessel which then travels to the break. During the period of wave travel, the stagnation condition at the break will be reduced from the original state.

Using the isentropic flow assumption discussed in NEDO-20533, Ref. 1, GE calculated the flow rate during the initial wave propagation period to be 72% of the value obtained using HEM at the original stagnation condition. For the assumed condition of isentropic flow, we obtain similar results using the methods presented by our consultant at BNL in Ref. 6.

For open pipes connected to a vessel, the period of reduced flow is of short duration since the wave propagation speed is approximately the speed of sound for liquids (5000 ft/sec). At this velocity, the time required for the pressure wave to traverse a BWR recirculation pipe would be about 50 milliseconds.

For a pipe which has a blockage at the vessel such as the jet pump nozzles, a wave of reduced magnitude would be reflected from the vessel so that the flow rate will decrease from the initial value. This situation would occur for the recirculation piping of BWR-3's and 4's which enter the reactor vessel through the jet pump nozzles.

Instead of decreasing the flow rate during the initial blowdown period as the pressure in the pipe is reduced, GE proposes to assume that the flow remains constant at the initial value of 0.72 times HEM until the initial pipe inventory is exhausted. This requires about 1.2 seconds. Following this time, the flow is based on 1.0 times HEM using the flow area of jet pump nozzles. The 0.72 factor is larger than the value actually predicted using the methodology of NEDO-20533 since it is based on the assumption that

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the discharged fluid is saturated. If the actual subcooled state of the fluid in the recirculation piping were utilized, a slightly lower value would be obtained.

For the piping section connected to the vessel at the vessel outlet location, GE will use the 0.72 multiplier only for the brief amount of time required for the acoustic wave to traverse through the piping to the vessel and return. Following this period, a flow rate of 1.0 times HEM and the pipe cross sectional area will be used to compute flow for the duration of the blowdown.

As justification for the reduced flow rate during the pipe decompression period, GE has provided a comparison of break flows using the RELAP-4 code for a typical BWR with jet pumps. The RELAP-4 analysis utilized the Henry-Fauske model to predict break flows when the flow was subcooled and the Moody slip flow model was used to predict flow for saturated fluid conditions.

The flow rates calculated by these models are about 60% higher than HEM for saturated and slightly subcooled conditions typical of a BWR. Comparisons of the RELAP-4 flow models to test data from the Marviken experiments were made by the staff in Ref. 7 and by INEL to Semiscale test data in Ref. 8. These comparisons indicate that the models are conservative.

The BWR RELAP-4 model included a multinode description of the reactor vessel piping. The multinode piping description permits RELAP to calculate the acoustic wave propagation following the break. Since the GE model does not take credit for the depressurization of the line between the break and the jet pumps until the line has been evacuated, the model produced 20% higher flows for this period than RELAP. The comparison of the integrated break flow between RELAP-4 and the GE model is attached as Figure 5.

Following the end of the pipe blowdown period, the GE results continued to be more conservative than the RELAP-4 predictions. This results primarily because GE assumes the fluid leaving the vessel is at the liquid stagnation enthalpy. This enthalpy is lower than the two-phase stagnation enthalpy calculated by the RELAP-4 code. The assumption of an all liquid blowdown increases the break flow calculated using HEM so that by the end of 10 seconds, which is about the time of the peak drywell pressure, the GE prediction still exceed RELAP by 15%. The GE results continued to be higher than RELAP for the remainder of the blowdown. The total mass release in the GE model is higher than the RELAP prediction for the total blowdown because of the conservative treatment of feedwater in the GE model. The feedwater is assumed to be within the reactor vessel at an elevated temperature rather than in the system piping.

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#### STAFF CONCLUSIONS

Based on our comparisons of the HEM to experimental data as discussed in the preceeding evaluation, we cannot conclude that HEM is either a conservative or best estimate method for predicting break flow. The Marviden tests provide a break geometry similar to the vessel outlet side of the postulated break. The evaluations of our consultant at BNL indicate that for these tests flow rates are underpredicted by as much as 40% using HEM. For the vessel inlet side of the break that contains the jet pump nozzles, the flow geometry resembles an orifice. The data in References 2, 3 and 4 indicate that for orifice geometry the flow rates could be in excess of HEM by as much as 150%

GE has utilized HEM in a non-mechanistic reactor system model which does not take credit for pressure reduction in the piping during the early portion of blowdown and conservatively assumes all liquid flow during most of the remainder of the blowdown. By comparison of the mass and energy predictions of the GE model to those of a conservative RELAP-4 analysis, we have concluded that the GE model is conservative for prediction of critical flow rates for a postulated double-ended recirculation line break for BWRs with MARK 1 containments.

The GE methodology on the application of HEM to reactor blowdown is presented in the form of answers to the NRC questions. We require that this and the other supporting material in the letters of November 8, 1977 and June 30, 1978 be incorporated into the approved version of topical report NEDO-21052.

#### References

- W. Bilanim, "The Genral Electric Mark III Pressure Suppression Containment System Analytical Model", General Electric Report NEDO-20533, June 1974.
- G. L. Sozzi and W. A. Sutherland, "Critical Flow of Saturated and Subcooled Water at High Pressure", General El ctric Report NEDO-13418, July 1975.
- R. L. Collins, "Choked Expansion of Subcooled Water and the I.H.E. Flow Model, ASME Journal of Heat Transfer, Vol. 100, May 1978.
- U. Simon, "Blowdown Flow Rates of Initially Subcooled Water" ANS Topical Meeting on Water Reactor Safety, CONF-730304, March 1973.
- L. Ericson et al, "The Marviken Full Scale Critical Flow Tests Interim Reports", Results from tests 1, 2, 3, 4, and 5, 1978, Unpublished.
- 6. P. G. Kroeger, "The Propagation of Phase-Change Fronts in Moving Fluids," BNL-NUREG-50687, August 1977.
- W. L. Jensen, NRC Memo, "Preliminary Investigation of Marviken Critical Flow Data", May 1978.
- Douglas G. Hall, "A Study of Critical Flow Prediction for Semiscale MOD-1 Loss-of-Coolant Accident Experiments, TREE-NUREG-1006, December 1976.

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Note: The flow rate k is the measured rate normalized to the HEM value.

Figure 2 FLOW RATE FACTOR RELATED TO NOZZLE GEOMETRY (SHARP-EDGED INLET)



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Figure 3 COMPARISON OF COMPUTED AND MEASURED MASS FLUX HISTORIES FOR MARVIKEN TEST NO. 1 (15 C SUBCOOLING), USING A QUASI-STATIONARY ISENTROPIC (IHE) DISCHARGE FLOW MODEL NEDO-21052-A



Figure 4 COMPARISON OF COMPUTED AND MEASURED MASS FLUX HISTORIES FOR MARVIKEN TEST NO. 2 (30 C SUBCOOLING), USING A QUASI-STATIONARY ISENTROPIC (IHE) DISCHARGE FLOW MODEL NED0-21052-A

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## TECHNICAL INFORMATION EXCHANGE

AUTHOR SUBJECT	TIE NUMBER 75NED53 OATE May 1979	
F. J. Moody 730		
TITLE Maximum Discharge Rate of	GE CLASS	
Liquid-Vapor Mixtures from Vessels	GOVERNMENT CLASS	
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SUMMARY	And a second sec	

A discrepancy exists in theoretical predictions of the two-phase equilibrium discharge rate from pipes attached to vessels. Theory which predicts critical flow data in terms of pipe exit pressure and quality severely overpredicts flow rates in terms of vessel fluid properties. This study shows that the discrepancy is explained by the flow pattern. Due to decompression and flashing as fluid accelerates into the pipe entrance, the maximum discharge rate from a vessel is limited by choking of a homogeneous bubbly mixture. The mixture tends toward a slip flow pattern as it travels through the pipe, finally reaching a different choked condition at the pipe exit.

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