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OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

Licensee Event Report
Reportable Occurrence No. 50-219/79-22/3L-0

Report Date

July 27, 1979

Occurrence Date

June 27, 1979

Identification of Occurrence

During emergency service water pump operability test, Containment Spray System I was found to be in a degraded mode. This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.2.

Conditions Prior to Occurrence

The plant was operating at steady state power.

Power: Generator, 653 MWe
Reactor, 1907.3 MWt
Flow: Recirculating, 15.1×10^4 gpm
Feedwater, 7.128×10^6 lb/hr
Stack Gas: 3.22×10^4 μ Ci/sec

Description of Occurrence

On Wednesday, June 27, 1979, at approximately 1530 hours, while performing an emergency service water pump operability test after electrical maintenance, a leak was detected on the emergency service water side of containment spray heat exchanger (1-2). A 3/4" threaded nipple connection between the waterbox and the piping leading to safety valve V-3-83 was found leaking.

Apparent Cause of Occurrence

It appears that the 3/4" nipple failed due to corrosion.

Analysis of Occurrence

Two separate primary containment spray system loops are provided for reliability. Each loop under the postulated accident conditions has the

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heat removal capacity of removing fission product decay heat to maintain the primary containment pressure below the design pressure of 35 psig.

The safety significance of this event is considered to be minimal since Containment Spray System I was not considered to be inoperable but only degraded. Containment Spray System II was available for operation.

Corrective Action

The 3/4" nipple was replaced on the containment spray heat exchanger (1-2) waterbox. On March 1, 1979, a similar failure occurred on containment spray heat exchanger (1-4). It was determined at that time that apparent cause of the leak was attributed to improper installation of the 3/4" nipple at the waterbox connection. All four 3/4" nipples were replaced at that time (March 1979). The nipple that failed will be sent to GPU Labs for further investigation to determine what type of corrosion caused the nipple to deteriorate. The 3/4" nipples on the waterbox side of the remaining containment spray heat exchangers will be replaced. Also, similar type nipples (carbon steel) on the waterbox side of the heat exchangers will be inspected.

Failure Data

Similar event -- Reportable Occurrence No. 50-219/79-04/3L-0.