



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C.
1200 K AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

Docket No. 50-333

JUL 18 1979

Power Authority of the State of New York
James A. FitzPatrick Nuclear Power Plant
ATTN: Mr. J. D. Leonard
Resident Manager
P. O. Box 41
Lycoming, New York 13093

Gentlemen:

IE Bulletin No. 79-14 that was issued on July 2, 1979, is being revised to limit the scope of work required. The changes are indicated on the enclosed replacement page No. 2 for the Bulletin. If you desire additional information regarding this matter, please contact this office.

Sincerely,


for Boyce H. Grier
Director

Enclosure:

1. IE Bulletin No. 79-14 Revision 1

cc w/encls:

George T. Berry, Executive Director
P. W. Lyon, Manager - Nuclear Operations
A. Klausmann, Director, Quality Assurance
M. C. Cosgrove, Quality Assurance Supervisor
J. F. Davis, Chairman, Safety Review Committee
V. J. Cassan, Assistant General Counsel
G. M. Wilverding, Licensing Supervisor

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ENCLOSURE 1

IE Bulletin No. 79-14

Revision 1

Date: July 18, 1979

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Action to be taken by Licensees and Permit Holders:

All power reactor facility licensees and construction permit holders are requested to verify, unless verified to an equivalent degree within the last 12 months, that the seismic analysis applies to the actual configuration of safety-related piping systems. The safety related piping includes Seismic Category I systems as defined by Regulatory Guide 1.29, "Seismic Design Classification" Revision 1, dated August 1, 1973 or as defined in the applicable FSAR. The action items that follow apply to all safety related piping 2½-inches in diameter and greater and to seismic Category I piping, regardless of size which was dynamically analyzed by computer. For older plants, where Seismic Category I requirements did not exist at the time of licensing, it must be shown that the actual configuration of ~~these~~ safety-related systems, utilizing piping 2½ inches in diameter and greater, meets design requirements.

Specifically, each licensee is requested to:

1. Identify inspection elements to be used in verifying that the seismic analysis input information conforms to the actual configuration of safety-related systems. For each safety-related system, submit a list of design documents, including title, identification number, revision, and date, which were sources of input information for the seismic analyses. Also submit a description of the seismic analysis input information which is contained in each document. Identify systems or portions of systems which are planned to be inspected during each sequential inspection identified in Items 2 and 3. Submit all of this information within 30 days of the date of this bulletin.
2. For portions of systems which are normally accessible*, inspect one system in each set of redundant systems and all nonredundant systems for conformance to the seismic analysis input information set forth in design documents. Include in the inspection: pipe run geometry; support and restraint design, locations, function and clearance (including floor and wall penetration); embedments (excluding those covered in IE Bulletin 7902); pipe attachments; and valve and valve operator locations and weights (excluding those covered in IE Bulletin 7904). Within 60 days of the date of this bulletin, submit a description of the results of this inspection. Where nonconformances are found which affect operability of any system, the licensee will expedite completion of the inspection described in Item 3.

*Normally accessible refers to those areas of the plant which can be entered during reactor operation.

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