ARKANSAS POWER & LIGHT COMPANY POST OFFICE BOX 551 LITTLE ROCK, ARKANISAS 72203 (501) 371-4422

July 24, 1979

WILLIAM CAVANAUGH III Vice President Generation & Construction

2-079-7

Director of Nuclear Reactor Regulation ATTN: John F. Stolz, Chief Light Water Reactors Branch #1 U.S. Nuclear Regulatory Commission Washington, D.C. 20555

> Subject: Arkansas Nuclear One-Unit 2 Docket No. 50-368 License No. NPF-6 Proposed Technical Specifications Change-EFW Pump Response Time (File: 2-1511.1)

Gentlemen:

This letter proposes a change to the Arkansas Nuclear One-Unit 2 (ANO-2) Technical Specifications. We wish to change Table 3.3-5, Items 7a and 8a from \leq 21.4 seconds to \leq 97.4 seconds for the following reasons.

Items 7a and 8a of Table 3.3-5 give the maximum allowable response time for the steam-driven emergency feedwater (EFW) pump as less than or equal to 21.4 seconds while the maximum allowable response time for the motor-driven EFW pump is \leq 112.4 seconds assuming loss of off-site power and 97.4 seconds assuming off-site power is available.

We have verify 1 through our NSSS vendor that no credit was taken in the ANO-2 safety analysis for delivery of EFW from either pump prior to 97.4 seconds following an actuation signal. Therefore, we wish to incorporate the 97.4 second response time for the steam-driven EFW pump. A revised page 3/4 3-20 of the ANO-2 Technical Specification with vertical lines showing the items changed is included.

MEMBER MIDDLE SOUTH UTILITIES SYSTEM

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Pursuant to the requirements of 10 CFR 170.12, we have determined our amendment request to require a Class III fee of \$4,000 as it involves a single safety issue. Accordingly, a check in the amount of \$4,000 is remitted.

Very truly yours, William Cavanaugh, III

WC:DCT:nak

Enclosure

STATE OF ARKANSAS COUNTY OF PULASKI

SS

William Cavanaugh*III, being duly sworn, states that he is Vice President, Generation & Construction, for Arkansas Power & Light Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this Supplementary Information; that he has reviewed or caused to have reviewed all of the statements contained in such information, and that all such statements made and matters set forth therein are true and correct to the best of his kncwledge, information and belief.

William Cavanaugh III

SUBSCRIBED AND SWORN TO before me, a Notary Public in and for the County and State above named, this 24th day of July 1979.

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My Commission Expires:

My Commission Expires 9/1/81

TABLE 3.3-5 (Continued)

ENGINEERED SAFETY FEATURES RESPONSE TIMES

INI	TIATING SIGNAL AND FUNCTION	RESPONSE TIME IN SECONDS
4.	Containment PressureHigh-High	
	a. Containment Spray	< 42.1*/27.1**
5.	Steam Generator Pressure-Low	
	a. Main Steam Isolation.	<u><</u> 3.9
	b. Feedwater Isolation	≤ 36.4*/21.4**
6.	Refueling Water Tank-Low	
	a. Containment Sump Valve Open	<u><</u> 145.0
7.	Steam Generator Level-Low	
	a. Emergency Feedwater - Train A	< 97.4
	b. Emergency Feedwater - Train B	<pre> 112.4*/97.4** </pre>
8.	Steam Generator <u>AP-High</u> Coincident With Stea	m Generator Level-Low
	a. Emergency Feedwater - Train A	≤ 97.4
	b. Emergency Feedwater - Train B	< 112.4*/97.4**

TABLE NOTATION

*Diesel generator starting and sequence loading delays included.

** Diesel generator starting delays not included, sequence loading delays included. Offsite power available.

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