



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

50-320 22055

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June 27, 1979

Docket No. 50-312

Mr. J. J. Mattimoe
Assistant General Manager and
Chief Engineer
Sacramento Municipal Utility District.
6201 S Street
P. O. Box 15830
Sacramento, California 95813

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Dear Mr. Mattimoe:

By Order of May 7, 1979, the Commission confirmed your undertaking a series of actions, both immediate and long term, to increase the capability and reliability of the Rancho Seco Nuclear Generating Station to respond to various transient events. In addition, the Order confirmed that you would shut down Rancho Seco on April 28, 1979, and maintain the plant in a shut-down condition until the following actions had been satisfactorily completed:

- (a) Upgrade the timeliness and reliability of delivery from the Auxiliary Feedwater System by carrying out actions as identified in Enclosure 1 of your letter of April 27, 1979.
- (b) Develop and implement operating procedures for initiating and controlling auxiliary feedwater independent of Integrated Control System control.
- (c) Implement a hard-wired control-grade reactor trip that would be actuated on loss of main feedwater and/or turbine trip.
- (d) Complete analyses for potential small breaks and develop and implement operating instructions to define operator action.
- (e) Provide for one Senior Licensed Operator assigned to the control room who has had Three Mile Island Unit No. 2 (TMI-2) training on the B&W simulator.

By submittal of May 14, 1979, as supplemented by seven letters dated May 22, 24, 29, 30(3) and June 6, 1979, you have documented the actions taken in response to the May 7 Order. We have reviewed this submittal, and are satisfied that, with respect to Rancho Seco, you have satisfactorily completed the actions

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prescribed in items (a) through (e) of paragraph (1) of Section IV of the Order, the specified analyses are acceptable, and the specified implementing procedures are appropriate. The bases for these conclusions are set forth in the enclosed Safety Evaluation.

As noted on page 13 of the Safety Evaluation, you will be required to conduct a test during power operation to demonstrate operator capability to assume manual control of the Auxiliary Feedwater System independent of the Integrated Control System.

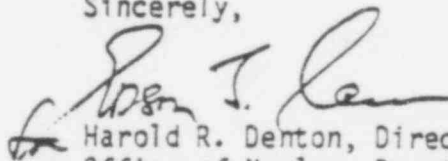
Appropriate Technical Specifications for Limiting Conditions for Operation and for surveillance requirements should be developed as soon as practicable and provided to the staff within seven days with regard to the design and procedural changes which have been completed in compliance with the provisions of the May 7, 1979 Commission Order. The revised Technical Specifications should cover:

- (1) Addition of flow indication to the Auxiliary Feedwater System;
- (2) Addition of the Anticipatory Reactor Trips; and
- (3) Changes in set points for high pressure reactor trip and PORV actuation.

Within 30 days of receipt of this letter, you should provide us with your schedule for completion of the long term modifications described in Section II of the May 7 Order, and you should submit for staff review the model used in the analysis for potential small breaks referenced in your letter of May 14, 1979.

My finding of satisfactory compliance with the requirements of items (a) through (e) of paragraph (1) of Section IV of the Order will permit resumption of operation in accordance with the terms of the Commission's Order; it in no way affects your duty to continue in effect all of the above provisions of the Order pending your submission and approval by the Commission of the Technical Specification changes necessary for each of the required modifications.

Sincerely,



Harold R. Denton, Director
Office of Nuclear Reactor
Regulation

Enclosures:

1. Safety Evaluation
2. Notice

cc w/enclosures: See next page

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Sacramento Municipal Utility
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cc w/enclosure(s) and incoming
dtd.: 5/14, 5/22, 24, 29, 30(3) & 6/6/79
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