RC FORM 366 7.77) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) $(\mathbf{1})$ CONTROL BLOCK: 334 0 - 0 0 0 0 - 0 0 S 0 1 LICENSE NUMBER 0 0 0 2 4 5 0 0 5 3 0 7 9 8 0 6 2 7 7 9 9 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 CON'T REPORT Ke) 0 0 1 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) On May 30, 1979, during routine surveillances, (Reactor Low Pressure Pump Start 0 2 Functional/Calibration and Reactor Low Pressure Valve Permissive Functional/ 0 3 Calibration) it was discovered that pressure switches 263-52A, 263-52C, 263-54A and 0 4 263-548 tripped slightly outside their Tech. Spec. allowable band. No probable 0 5 consequences, other switches in inst. system would initiate E.C.C.S. 0 6 0 7 VALVE COMP. CAUSE CAUSE SYSTEM SUBCODE COMPONENT CODE SUBCODE T | R | U | (14 S 1(15 Z (16) SI 1E (13) 1 NI LE. SIF 0 9 REVISION OCCURRENCE REPORT SEQUENTIAL CODE TYPE NO. REPORT NO. EVENT YEAR LER/RO 0 0 3 11 6 9 REPORT NUMBER COMPONENT PRIME COMP. NPRD-4 FORM SUB. ATTACHMENT SUBMITTED METHOD SUPPLIER ACTION FUTURE (22) HOURS (25 (24) N 0 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The failure of these switches to trip at the desired setpoint was attributed to 10 isetpoint drift. 4 80 9 METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32) (30)FACILITY OTHER STATUS % POWER Routine Surveillance B (31) 0 0 0 NA H (28) 5 80 45 44 CONTENT LOCATION OF RELEASE (36) ACTIVITY AMOUNT OF ACTIVITY (35) RELEASED OF RELEASE NA NA 6 80 45 44 10 11 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE 0 10 10 NA 7 80 309 087 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 0 (40) NA 1 8 7907090306 80 11 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION TYPE NA Z 9 (42) 80 NRC USE ONLY PUBLICITY DESCRIPTION (45) JED. 44 NA 80.5 6.8 69 X475 1702 203-44 BUSINES

ATTACHMENT TO LER 79-16/3L NORTHEAST NUCLEAR ENERGY COMPANY PROVISIONAL LICENSE NUMBER DPR-21 DOCKET NUMBER 50-245

IDENTIFICATION OF OCCURRENCE

Engineered safety feature instrument settings which were found to be less conservative than those established by technical specifications but which did not prevent the fulfillment of the functional requirement of the Emergency Core Cooling System.

CONDITIONS PRIOR TO OCCURRENCE

The reactor was shutdown and cooled down in the refuel mode. Core inspection and vacuuming was in progress.

DESCRIPTION OF OCCURRENCE

On May 30, 1979, during routine surveillances (Reactor Low Pressure Pump Start Functional/Calibration and Reactor Low Pressure Valve Permissive Functional/ Calibration), it was discovered that pressure switches 263-52A, 263-52C, 263-54A, and 264-54B tripped slightly outside their Technical Specification allowable band.

SWITCH	FUNCTION	ACTUAL TRIP_POINT (PSIG)	TECH. SPEC. TRIP POINT (PSIG)	AMOUNT OUT OF SPEC. (PSIG)
263-52A	Valve Permissive	365	338 + 25	2
263-52C	Valve Permissive	378	351 + 25	
263-54A	Pump Start	378	351 + 25	2
263-54B	Pump Start	384	351 + 25	8

APPARENT CAUSE OF OCCURRENCE

The failure of these switches to trip at the desired setpoint was attribued to setpoint drift.

ANALYSIS OF OCCURRENCE

Two of these pressure switches provide a low pressure permissive signal for opening of the Emergency Core Cooling System admission valves. The other two pressure switches provide a low pressure permissive signal for starting the Emergency Core Cooling System pumps. Surveillance testing of the remaining switches, in each one-out-of-two-twice logic system indicated that their settings were within the required range. Hence, the setpoint drift did not impair the system's ability to perform its intended function.

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CORRECTIVE ACTION

The pressure switches in question were reset to within the required range and tested. All other switches in the logic systems were tested and found to be within the acceptance criteria.

The pressure switches in question were manufactured by the Meltron Corporation, Model No. 372-6SS49A-292, range of 28-1400 psig. This report is similar in nature to RO-76-27/3L and RO-76-47/3L.

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