



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PDR

JUL 18 1978

MEMORANDUM FOR: Lessons Learned Task Force

FROM: John A. Olshinski, Lessons Learned Task Force

SUBJECT: AIF SYSTEMS AND EQUIPMENT DESIGN CRITERIA  
SUBCOMMITTEE MEETING

On July 13, 1979, I met with the AIF Systems and Equipment Design Criteria Subcommittee. Enclosure 1 lists the subcommittee membership. The subcommittee met on 12 and 13 July 1979 to discuss the various topics listed in Enclosure 2. Subcommittee member responsibilities for each topic are also listed in Enclosure 2.

I discussed various Lessons Learned work or recommendations that impacted these topics. In addition, I indicated the proposed schedule of publishing NUREG-0578 (TMI-2 Lessons Learned Task Force Status Report and Short Term Recommendations).

Particular suggestions or recommendations that were made by the subcommittee included:

1. Considerations of possible negative impacts of containment inerting because of restricted access to equipment and possible negative health effects on plant personnel.
2. Evaluation of the use of stainless steel cladding as a preventative measure in limiting combustible gas generation vice the use of inerting of containments as a mitigation measure.
3. Consideration of evaluation of balance-of-plant items because of their impact on the initiation and mitigation of events as well as their impact on operator actions necessitated because of balance-of-plant initiated events.
4. Consideration of discussions with Naval Reactors concerning their quality control and system classification requirements when evaluating adequacy of current safety grade vs. non-safety grade classification of systems and components.

Additional discussion was held concerning analysis of core uncover symptoms and the necessity for reactor vessel level detection instrumentation.

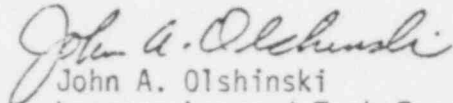
⑤  
7908030 493

440 002

JUL 18 1978

The subcommittee presently intends to make recommendations on their assigned areas by mid-October.

I encouraged the subcommittee to maintain contact with the Lessons Learned Task Force. I also encourage members of Lessons Learned to maintain contact with the subcommittee through Fred Stetson(AIF). The subcommittee work in review of systems and design criteria can provide an expanded informational base to the Lessons Learned Task Force in the evaluation of these areas.

  
John A. Olshinski  
Lessons Learned Task Force

cc: F. Stetson (AIF)  
PDR

440 003

SYSTEMS & EQUIPMENT DESIGN CRITERIA SUBCOMMITTEE

CHAIRMAN: Dr. D. Clark Gibbs  
 Director of Nuclear Activities  
 Middle South Services, Inc.  
 Box 61000  
 New Orleans, Louisiana 70161  
 504/529-5262 x. 271, 217  
 Telecopy: x. 262

SECRETARY: Fred. T. Stetson, Jr.  
 Mgr. Reactor Licensing  
 and Safety  
 Atomic Industrial Forum  
 7101 Wisconsin Avenue  
 Washington, D. C. 20001  
 301/654-9260 x. 268

Vice Chairman: Roy H. Dunham  
 Director of Engineering Design  
 Tennessee Valley Authority  
 W 11 A 9, 400 Commerce Avenue  
 Knoxville, Tennessee 37902  
 615/632-2651

Robert Spilker  
 Project Engineer  
 Ebasco Services, Inc.  
 19 Rector Street  
 New York, New York 10006  
 212/785-8087

Mr. John MacAdoo  
 Assistant Manager Nuclear Safety  
 Westinghouse Electric Company  
 P. O. Box 355  
 Pittsburgh, Pa. 15230  
 412/373-4374

Dr. Donald H. Roy  
 Manager Engineering  
 Nuclear Power Generation Div.  
 Babcock & Wilcox  
 P. O. Box 1260  
 Lynchburg, Virginia 24505  
 804/384-5111 x. 2375

*ROBERT W. HILL*  
 Gibbs & Hill, INC.  
 393 Seventh Avenue  
 New York, New York 10001  
 212/760-4000

James D. Crawford  
 Director, Plant Engineering  
 Power Systems Division  
 Combustion Engineering, Inc.  
 1000 Prospect Hill Road  
 Windsor, Connecticut 06095  
 203/688-1911 x. 3313

Mr. Carroll Bitting  
 Manager of Projects  
 Gilbert Commonwealth Associates  
 525 Lancaster Avenue  
 Reading, Pa. 19603  
 215/775-2600 x. 2645

Mr. Ken Swarts  
 Senior Engineering Manager  
 Brown & Root, Inc.  
 P. O. Box 3  
 Houston, Texas 77001  
 713/676-8185

*STEPHEN J. MILIOTI*  
 American Electric Power Company  
 2 Broadway  
 New York, New York 10004  
 212/440-9275

EPRI Liaison: Mike Kolar  
 415/855-2660

NRC Contact: John A. Olshinski, DSS; *for Tennessee, BACKUP*

Owners Groups Liaisons

.440 004

B&W:  
 GE:

CE:  
 W:

*Enclosure 1*

## ENCLOSURE 2

### Containment Isolation (Frank Hutchinson)

Backfit diverse actuation required by SRP 6.2.4.  
Rethink essential and non-essential systems.  
Loss of component cooling water to RCP motor bearings.  
Loss of component water to RCP seals.  
Criteria for actuation.  
Criteria for systems isolated.  
Ability to pump back gases and liquids to containment.

### Emergency Power Supplies (John MacAdoo)

Pressurizer Heaters (i.e. need to maintain hotleg subcooling with loss of offsite power or use of an alternate pressure source e.g. inert gas, air compressor, or controllable head pump).  
PORV and block valves.  
Pressurizer level indicator.  
Reactor coolant pumps (e.g. pony motors or reduced voltage/frequency - capability of pump to accommodate low speed operation).  
Reliability of AC/DC power supplies - onsite and offsite.

### Adequate Core Cooling (Don Roy)

Instrumentation and detection of inadequate core cooling.  
RHR design basis and adequacy; access capability and equipment locations.  
Increase letdown cooler capacity.  
Prevention through design of degraded cooling.  
core cooling systems  
containment systems  
radwaste and effluent systems  
Analysis of core uncover symptoms.  
Reactor vessel level detection.

### Post Accident Hydrogen Control (Roy Dunham)

Provide reliable and dedicated penetrations for purge and recombiner systems.  
Inert all mark I and II containments. (Vermont Yankee and Hatch).  
Provide capability to use recombiners at all plants.  
Filtered, vented containments on PWR's.

### Instruments to Follow Accident (Ken Swarts)

Improve post-accident sampling.  
Increase range of effluent monitors.  
Provide hi-range containment monitor.  
Improve in-plant and effluent iodine measurement.  
Reg. Guide 1.97 Rev. 1 August, 1977 - implementation.

Safety and Process System Design Classifications (Bob Spilker)

Reliability.  
Environmental qualification.  
Power supply.  
Establishment of a class 2E and 3E designation.

Miscellaneous (NSSS) (Ed Scherer)

Low temperature overpressure protection.  
Reactor coolant system venting.

Miscellaneous (BOP) (Steve Milioti)

Auxiliary building ventilation and filtration.  
Radiation control for systems outside containment.  
Perform leak test and provide tech specs for safety and process systems.  
Perform shielding review for safety and process systems.