



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Mr. George T. Berry  
General Manager and Chief  
Engineer  
Power Authority of the State  
of New York  
10 Columbus Circle  
New York, N. Y. 10019

Mr. Vito J. Cassan  
Assistant General Counsel  
Power Authority of the State  
of New York  
10 Columbus Circle  
New York, New York 10019

Mr. Peter W. Lyon  
Manager-Nuclear Operations  
Power Authority of the State  
of New York  
10 Columbus Circle  
New York, New York 10019

Mr. J. D. Leonard, Jr.  
Resident Manager  
James A. Fitzpatrick Nuclear Power  
Plant  
P. O. Box 41  
Lycoming, New York 13093

Director, Technical Development  
Programs  
State of New York Energy Office  
Agency Building 2  
Empire State Plaza  
Albany, New York 12223

Oswego County Office Building  
46 E. Bridge Street  
Oswego, New York 13126

George M. Wilverding, Licensing Supervisor  
Power Authority of the State of New York  
10 Columbus Circle  
New York, New York 10019

Docket File  
NRC PDR  
Local PDR  
NRR Rdg  
ORB#3 Rdg  
H. Denton  
E. Case  
V. Stello  
D. Eisenhut  
B. Grimes  
R. Vollmer  
T. Ippolito  
R. Reid  
V. Noonan

G. Knighton  
A. Schwencer  
D. Ziemann  
P. Check  
G. Lainas  
D. Davis  
OELD  
OI&E (3)  
S. Sheppard  
Project Manager  
ACRS (16)  
NRC Participants  
→ TERA  
J. Buchanan

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JUNE 29 1979

Docket No. 50-333

MEMORANDUM FOR: D. Eisenhut, Acting Director, Division of Operating Reactors

FROM: P. J. Polk, Project Manager, ORB #3

THRU: T. A. Ippolito, Chief, ORB #3, DOR

SUBJECT: OPEN MEETING ON JUNE 19, 1979 WITH THE POWER AUTHORITY OF THE STATE OF NEW YORK REGARDING SAFETY RELATED PIPING REANALYSIS FOR THE JAMES A. FITZPATRICK NUCLEAR POWER PLANT

As a follow-up to a letter dated June 8, 1979, the NRC staff requested a meeting with the Power Authority of the State of New York (PASNY). The referenced letter forwarded the licensee's request to startup the James A. FitzPatrick (JAF) Nuclear Plant contingent upon partial completion of the piping seismic reanalysis necessitated by the Show Cause Order of March 13, 1979. The purpose of the follow-up meeting was to ensure that the licensee had a clear understanding of staff requirements for interim, as well as unlimited, plant operation.

The subject meeting was held in Bethesda, Maryland on June 19, 1979. Meeting participants (enclosure 1) included PASNY personnel and representatives of Stone & Webster (S&W) Engineering Corporation. Although the meeting was open to public observation, no members of the public were in attendance other than a representative from the State of New York Energy Department.

The licensee presented a current status of the reanalysis effort. In addition, PASNY indicated the portion of the total effort which would be completed and submitted for staff approval prior to operating the plant on an interim basis. In sum, the reanalysis has been divided into four areas of reanalysis: (1) Piping seismic computer calculations, (2) Pipe support calculations, (3) Equipment nozzle calculations, and (4) Containment penetration calculations. With respect to items (1), (3) and (4) the reanalysis is near completion. In anticipation of correction of as-built problems and relief from vendor interface requirements, the licensee expects to provide a submittal which indicates stresses and loads are within allowable limits. Regarding pipe supports, item (2), PASNY indicated that a partial submittal would be provided. This submittal will document support design in those areas of the plant that are inaccessible during normal plant operation.

Of the revised total 984 pipe supports, 335 are located in inaccessible areas and 649 are accessible during normal plant operation. Of the inaccessible 335, to date 234 have been reviewed. Twenty three supports (8 due to as-built problems and 15 due to increased loadings) have been found deficient and are being modified. In addition to finalizing the review of the total 335 supports the licensee was requested to address the significance with respect to piping integrity of the 23 supports requiring modification in order to extrapolate these results to cover the remaining 649 supports.

With respect to the review of the 649 accessible supports the staff indicated that a schedule would be required and that 60 days was considered a reasonable period to complete the reanalysis. The schedule should be carefully considered to prioritize the more significant safety systems for early completion.

During interim plant operation, if allowed, the licensee is expected to provide a procedure for problem identification. Upon identifying a problem, a Licensing Event Report (LER) would be required within 24 hours. Upon issuance of an LER, the licensee will be allowed 7 days or Technical Specification time limits, whichever is less, to accomplish modifications or to show cause why JAF should not be shutdown.

In conclusion, the licensee was informed that the staff will require approximately two weeks to review the PASNY submittal requesting interim operation. The licensee was also informed of the fact that the staff is continuing to review the S&W modeling techniques, and that an I&E field as-built verification should be expected in the near future. The licensee was advised that their submittal should address: (1) The reanalysis effect on the location of pipe restraints, (2) Base plate design, and (3) Vellan valve use at JAF. In order to assist in the preparation of the submittal, the Surry Plant Safety Evaluation Report (SER) will be forwarded when approved.



Philip J. Polk, Project Manager  
Operating Reactors Branch #3  
Division of Operating Reactors

Enclosure:  
Attendance list

Enclosure 1

NRC - JAF ATTENDEES

PASNY

P. Early  
J. Helland  
O. Mallon  
J. Dainora  
J. Leonard  
P. Reichert  
R. Rajaram  
R. Paccione

SWEC

C. Grochmal  
P. Pam

S&W

W. Kennedy  
K. Chu  
T. Siskin  
J. Hall  
W. Matson

New York State Energy Dept.

J. Baranski

NRC

P. Polk  
J. Fair  
M. Mulkey  
V. Noonan (afternoon only)

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