LICENSEE EVENT REPORT						
	CONTROL BLOCK:		(PLEAS	E PRINT OR TYP	PE ALL REQUIRED INF	ORMATION)
0 1 8	MAPPS 11 2 LO	1010101010	La La La	1010 3	4 1 1 1 1 1 1 6 LICENSE TYPE 30	(4) L 1 (5)
0 1 7 8	SOURCE L 6 0 5 0			11071 VENT DATE	9 8 U 7 1 75 REPUST	8 7 9 9 DATE 80
0 2	On July 10, 1979, wa	ter was observe	d drippin	g from lagg	ging on the CRD	eturn
0 3	line in the steam tunnel area. The lagging was removed and a pin ole leak dis-					
0 4	covered adjacent to	the inboard wel	d on the	3" SS inboa	ard manual sol	ation valve
0 5	(301-99). This leak was within the reactor coolant pressure boundary. The unit					
0 6	was shut down on Jul	11, 1979, to	examine a	nd repair t	the leak.	1
0 7						
[0]8]						
0 9 ₂ 8	SYSTEM CAUSE CAUSE SUBCODE COMPONENT CODE SUBCODE SUBC					
10	Investigation reveals		ne inboar	d weld. Th	ie valve was rei	placed with
	a straight run section of pipe eliminating the bi-metal junction. Non-destructive J					
1 2	testing was successfully performed and the system returned to service.					
13						
14						
1 5	FACILITY STATUS		METHOD OF DISCOVERY		Observation	10N (32)
	ELEASED OF RELEASE AMOUN Z 33 Z 34 1 PERSONNEL EXPOSURES	N/A 44	45		LOCATION OF RELEASE N/A	36)
7 8	PERSONNEL INJURIES NUMBER TYPE DESCRIPTION 11 12 13 PERSONNEL INJURIES NUMBER DESCRIPTION 11 12 13 PERSONNEL INJURIES NUMBER	N/A				80
1 8	0 0 0 0 40 E	N/A				
1 9	LOSS OF OR DAMAGE TO FACILITY 43	N/A	390	726069	92	80
2 0	PUBLICITY SSUED DESCRIPTION 45 N 44 10	N/A	454	284	68 69	NRC USE ONLY
	NAME OF PREPARER	John M. Fulton			PHONE: (617)	746-7900

BOSTON EDISON COMPANY PILGRIM NUCLEAR POWER STATION

DOCKET NO.: 50-293 ATTACHMENT TO LER - 79-027/01T-0

It has been determined that the pinhole leak in the CRD return line originated in the root weld between the 3" SS manual isolation valve (301-99) and the 3" CS-160 pipe. This crack migrated circumferentually through the crown weld and logitudinally into the pipe. The valve and piping were removed for radiographing and repair. During the examination it was decided to postpone replacement of the valve until the 1980 refueling outage and to install a section of 3" CS-160 pipe in its place.

A Plant Design Change Request (PDCR) was in tiated, a Safety Evaluation performed by the Engineering Department and the section of pipe installed. The purpose of the manual isolation valve was to both provide a convenient means of isolating the CRD return line for Local Leak Rate testing and operational purposes. Replacing this v. e with a straight run of pipe will not affect system operation and performance

Non-destructive testing was successfully performed and the system returned to service.