

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JUNE 1979

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 and 50-265

LICENSE NOS. DPR-29 and DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Inc. and the primary construction contractor was United Engineers & Constructors. The condenser cooling method is a closed-cycle spray canal, and the Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971 and March 21, 1972 respectively, pursuant to Docket Numbers 50-254 and 50-265. The date of initial reactor criticalities for Units 1 and 2 respectively were October 18, 1971 and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit 1 and March 10, 1973 for Unit 2.

This report was compiled by Dave Hannum, telephone number 309-654-2241, ext. 179.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

- June 1 - 8: Unit One began the reporting period operating at 792 MWe. The electrical load was held constant except for minor load reductions on June 2nd due to turbine testing and high condenser backpressure on June 6th and 7th.
- June 9 - 10: Load was reduced to 214 MWe in order to replace a thermocouple on the 1A inboard MSIV. Upon completion of the repair, load was increased at 100 MWe/hr.
- June 11 - 16: Unit One held a steady load of 800 MWe.
- June 17: Electrical load was reduced to 566 MWe in order to reverse main condenser circulating water flow.
- June 18 - 22: Unit One held a steady load of 783 MWe.
- June 23 - 24: The load was reduced to 550 MWe for main condenser flow reversal.
- June 25 - 26: Load was reduced to 684 MWe to change reactor feed pump operation due to the 1A reactor feed pump outboard bearing seal line leak. After completion of repairs, load was reduced again to return the 1A Reactor Feed Pump to operation.
- June 27 - 30: Unit One held a steady load of 791 MWe.

B. Unit Two

- June 1 - 5: Unit Two began the reporting period operating at 682 MWe. Electrical load was held constant except for a reduction to 450 MWe on June 3rd for main condenser flow reversal.
- June 6 - 10: The load fluctuated between 700 MWe and 500 MWe due to high condenser backpressure.

- June 11 - 21: Unit Two held an average load of 636 MWe during this period, which included a reduction in power to 500 MWe for main condenser flow reversal.
- June 22: At 1250 Unit Two scrambled due to a spurious false condenser low vacuum signal. The reactor was made critical at 1730 and the main generator was placed on line at 2042.
- June 23: At 0106 Unit Two scrambled due to a reactor low water level condition. The 2R feedwater regulator valve went closed for no apparent reason. The reactor was made critical at 0455 and the main generator was placed on line at 1040.
- June 24: Load was steadily increased at the rate of 10 MWe/hr to 700 MWe.
- June 25 - 27: Unit Two held an average load of 625 MWe.
- June 28: Load was reduced to 440 MWe due to high condenser backpressure.
- June 29: Unit Two held an average load of 633 MWe.
- June 30: Load was reduced to 550 MWe for main condenser flow reversal.

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III. PLANT OR PROCEDURE CHANGES, TESTS,
EXPERIMENTS AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specification

A small portion to Amendment No. 50 was inadvertently omitted when originally added to DPR-29 in April of 1979. This addition consisted of changes to the Maximum Average Planer Heat Generation Rate Limits (MAPLHGR).

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no tests or experiments performed during the reporting period requiring NRC approval.

D. Other Changes, Tests, and Experiments

Speical Test 2-26: EGC - Circuitry Test

The purpose of this test was to transfer control of Unit Two to the Economic Generation Control (EGC) System. The unit was placed in both local and remote automatic recirculation flow control and several power changes were performed in each mode. The unit was then returned to master manual recirculation flow control.

Summary of Safety Evaluation:

Operation in the automatic flow control is a normal mode of operation and has been considered in the FSAR. Additional safety has been added since the EGC system has added restrictions to the range of power operation. Thus, there would be less probability of an accident or

malfunction of equipment as evaluated in the FSAR. Operation in the automatic mode of EGC was accomplished only at greater than 20% power, 65-100% core flow, and application of proper k_f factors for automatic control limitations to MCPR. Thus, there was no reduction in the margin of safety as defined in the basis for any Technical Specification.

E. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the safety-related maintenance performed on Unit One and Unit Two during the reporting period. The headings indicated in this summary include Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

M.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
2967-79		VIV. (A0 1-1601-60)	The air supply fitting was loose.	The valve was operable although leaking air.	The fitting was tightened and the valve was cycled 3 times.
2477-79		RHR Hex. outlet vlvs. (1-1001-5A/B)	The position indicator was out of adjustment.	The position indicator did not agree with the light valve position indication. RHR was operable.	The position indicator was readjusted. The valves were stroked 3 times.
2866-79		RCIC Steam Line High Flow Switch (1-1360-1B)	The micro switch was defective.	The high flow switch operation was erratic. RCIC was operable.	The positive dp micro switch was replaced and calibrated.
2868-79	79-19/03L	Drywell-Torus Vacuum Bkr, VIV (1-1601-320)	The shaft bushing and packing were defective.	The valve had dual indication. Valve would have operated if required.	The packing and shaft bushing were replaced.
				415 270	

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
2878-79		CRD 54-31 (2-300).	The rod select relay was dirty.	The rod would not settle. Scram operability was unaffected.	The relay was cleaned.
3033-79		18" Main Steam Rad. Monitor (2-1734-B)	The detector connector was loose.	The monitor spiked HI-HI. Redundant monitors were operable.	The connector was tightened. The monitor was bench tested.
3008-79	79-10/03L	Fuel Pool Rad Monitor (2-1743-A).	The GM tube was defective.	The 902-10 indicator differed from the local monitor reading. B Monitor was operable.	The GM tube was replaced and calibrated. The vent system trip was functionally tested.
2986-79		RHR valve (2-1001-4A)	The main ckt breaker was out of adjustment.	The breaker tripped while testing valve operability. RHR was operable.	The breaker was adjusted. The valve was cycled 3 times.
2778-79		HPCI Steam Drain valve (A0 2-2301-31)	The operator diaphragm was cracked.	The operator was leaking air. HPCI was operable at all times.	The diaphragm was replaced.

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UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
2944-79		Off Gas Monitor (2-1705-13)	The monitor connectors were loose.	The "A" monitor was spiking downscale and the "B" monitor was erratic. Monitors were operable.	The connectors were tightened and the monitors functionally tested.
3208-79		"B" Main Steam Rad Monitor (2-1705-02B).	The monitor needed recalibrating.	The monitor was reading low. Monitors were operable.	The monitor was recalibrated.
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IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all license event reports for Quad-Cities Units one and two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

<u>UNIT ONE</u>		
<u>Licensee Event Report Number</u>	<u>Date of Occurrence</u>	<u>Title of Occurrence</u>
79-22/03L	6-22-79	Unit One Diesel Generator Fuel line leak after returning to service.
<u>UNIT TWO</u>		
<u>Licensee Event Report Number</u>	<u>Date of Occurrence</u>	<u>Title of Occurrence</u>
79-12/03L	6-22-79	Unit Two Diesel Generator Out of Service Longer than 1½ hours due to obstruction in winding.

V. DATA TABULATIONS

The following data tabulations are presented in this report.

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

POCKET NO. 050-254
 UNIT One
 DATE 6-30-79
 COMPLETED BY D. Hannum
 TELEPHONE (309) 654-2241,
Ext. 179

OPERATING STATUS

0000 060179
 2400 063079

1. Reporting period: 0000 060179 Gross hours in reporting period: 720
 2. Currently authorized power level (MWe): 2511 Max. depend. capacity (MWe-Net): 769 Design electrical rating (MWe-Net): 769
 3. Power level to which restricted (if any) (MWe-Net): NA
 4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours	<u>720</u>	<u>3278.8</u>	<u>50611.7</u>
6. Reactor reserve hours	<u>0</u>	<u>0</u>	<u>3329.6</u>
7. Hours generator on line	<u>720</u>	<u>3149.9</u>	<u>18092.8</u>
8. Unit reserve shutdown hours.	<u>0</u>	<u>19.8</u>	<u>909.2</u>
9. Gross thermal energy generated (MMBtu)	<u>1707530</u>	<u>6589668</u>	<u>95397544</u>
10. Gross electrical energy generated (MWh)	<u>549271</u>	<u>2083829</u>	<u>30656324</u>
11. Net electrical Energy Generated	<u>524045</u>	<u>1957138</u>	<u>28589711</u>
12. Reactor service factor	<u>100.0</u>	<u>75.5</u>	<u>80.9</u>
13. Reactor availability factor	<u>100.0</u>	<u>75.5</u>	<u>86.2</u>
14. Unit service factor	<u>100.0</u>	<u>72.5</u>	<u>76.9</u>
15. Unit availability factor	<u>100.0</u>	<u>73.0</u>	<u>78.3</u>
16. Unit capacity factor (Using MDC)	<u>94.6</u>	<u>58.6</u>	<u>59.4</u>
17. Unit capacity factor (Using Des. MWe)	<u>92.2</u>	<u>57.1</u>	<u>57.9</u>
18. Unit forced outage rate	<u>0.0</u>	<u>4.5</u>	<u>8.0</u>
19. Shutdowns scheduled over next 6 months (Type, date, and duration of each):			
20. If shutdown at end of report period, estimated date of startup:			<u>NA</u>

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* The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

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OPERATING DATA REPORT

REPORT NO. 050-265
 UNIT Two
 DATE 6-30-79
 COMPLETED BY D. Hannum
 TELEPHONE (309) 654-2241,
Ext. 179

OPERATING STATUS 0000 060179
2400 063079

1. Reporting period: _____ Gross hours in reporting period: 720
 2. Currently authorized power level (MWe): 2511 Max. depend. capacity (MWe-Net): 769 Design electrical rating (MWe-Net): 769
 3. Power level to which restricted (if any) (MWe-Net): NA
 4. Reasons for restriction (if any): _____

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>711.5</u>	<u>4240.9</u>	<u>49617.3</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>702.6</u>	<u>4194.7</u>	<u>47297.6</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MMBtu)	<u>1422758</u>	<u>8791768</u>	<u>97533892</u>
10. Gross electrical energy generated (MMBtu)	<u>437673</u>	<u>2744563</u>	<u>31232940</u>
11. Net electrical Energy Generated	<u>399636</u>	<u>2546811</u>	<u>29308271</u>
12. Reactor service factor	<u>98.8</u>	<u>97.6</u>	<u>80.5</u>
13. Reactor availability factor	<u>98.8</u>	<u>97.6</u>	<u>85.3</u>
14. Unit service factor	<u>97.6</u>	<u>96.6</u>	<u>76.7</u>
15. Unit availability factor	<u>97.6</u>	<u>96.6</u>	<u>77.9</u>
16. Unit capacity factor (Using MDC)	<u>72.2</u>	<u>76.3</u>	<u>61.8</u>
17. Unit capacity factor (Using Des. MWe)	<u>70.4</u>	<u>74.3</u>	<u>60.3</u>
18. Unit forced outage rate	<u>2.4</u>	<u>0.8</u>	<u>9.8</u>
19. Shutdowns scheduled over next 6 months (Type, date, and duration of each):			
20. If shutdown at end of report period, estimated date of starting:	<u>NA</u>		

* The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray coal.

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Docket No. 050-254

Unit One

Date 6-30-79

Completed by D. Hannum

Telephone (309) 654-2241
 Ext. 179

MONTH June 1979

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1. 753
 2. 730
 3. 757
 4. 747
 5. 752
 6. 747
 7. 734
 8. 745
 9. 400
 10. 641
 11. 762
 12. 760
 13. 762
 14. 762
 15. 757
 16. 742

17. 653
 18. 753
 19. 748
 20. 733
 21. 739
 22. 745
 23. 741
 24. 690
 25. 730
 26. 742
 27. 752
 28. 751
 29. 766
 30. 738
 31. ---

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INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day of the reporting month. Complete to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX 3
AVERAGE DAILY UNIT-POWER LEVEL

Revision 4
June 1976

Docket No. 050-265

Unit Two

Date 6-30-79

Completed by D. Hannum

Telephone (309) 654-2241,
Ext. 179

MONTH June 1979

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>636</u>
2.	<u>627</u>
3.	<u>532</u>
4.	<u>621</u>
5.	<u>615</u>
6.	<u>609</u>
7.	<u>530</u>
8.	<u>524</u>
9.	<u>533</u>
10.	<u>525</u>
11.	<u>618</u>
12.	<u>656</u>
13.	<u>638</u>
14.	<u>636</u>
15.	<u>532</u>
16.	<u>548</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>546</u>
18.	<u>588</u>
19.	<u>551</u>
20.	<u>584</u>
21.	<u>581</u>
22.	<u>318</u>
23.	<u>187</u>
24.	<u>556</u>
25.	<u>590</u>
26.	<u>571</u>
27.	<u>567</u>
28.	<u>538</u>
29.	<u>576</u>
30.	<u>534</u>
31.	<u>---</u>

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INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 5
March 1978

DOCKET NO. 050-254

UNIT NAME Quad Cities One

DATE June 30, 1979

REPORT MONTH June 1979

COMPLETED BY D. Hannum

TELEPHONE (309) 654-2241,
Ext. 179

NO.	DATE	TYPE FOR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
4	790609	F		H	4		CC	INSTRU	Load was reduced to 214 MWe to replace a thermocouple on the inboard 1A MSIV.

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-513
Revision 5
March 1973

DOCKET NO. 050-265

UNIT NAME Quad Cities Two

COMPLETED BY D. Hannam

DATE June 30, 1979

REPORT MONTH June 1979

TELEPHONE (309) 654-2241

Ext. 179

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
10	790622	F	7.9	A	3		IA	RELAYX	Unit Two scrambled due to a spurious false condenser low vacuum signal.
11	790623	F	9.6	A	3		CH	VALVEX	Unit Two scrambled due to reactor low water level. The 2B feedwater regulator valve went closed.

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VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission.

A. Main Steam Relief Valve Operations

There were no main steam valve actuations during the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

There were no control rod drive scram timing exercises performed during the reporting period.

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VII. REFUELING INFORMATION

The following information about future reloads at Quad Cities Station was requested in a January 26, 1978 licensing memorandum (78-24) from D.E. O'Brien to C. Reed et. al. titled "Dresden, Quad-Cities, and Zion Station - NRC request for refueling information dated January 18, 1978.

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QUAD-CITIES REFUELING
INFORMATION REQUEST

Revision 1
March 1978

1. Unit: 1 Reload: 5 Cycle: 6
2. Scheduled date for next refueling shutdown: September 1, 1980 (Shutdown)
3. Scheduled date for restart following refueling: Dec 15, 1980 (Startup)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
5. Scheduled date(s) for submitting proposed licensing action and supporting information: The QCI B5 licensing submittal is scheduled for Sept 1, 1980
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE

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7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
- b. Number of assemblies in spent fuel pool: 343
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 1460
- b. Planned increase in licensed storage: None
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

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Q. C. C. S. R.

RELOAD LICENSING PACKAGE
PREPARATION SCHEDULE

QTP 300-535
Revision 1
March 1973

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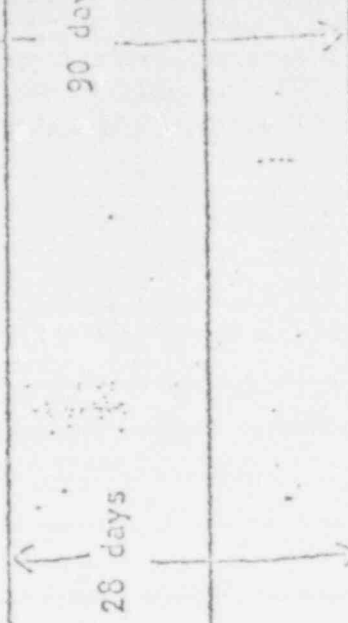
ACTIVITY

UNIT Q1
RELOAD 4
CYCLE 5

RESPONSIBILITY CENTER

DATE	RESPONSIBILITY CENTER	ACTIVITY
1/15/78	GE NFS	NFS receives draft licensing Submittal from GE Transmit copy of draft licensing Submittal for Comments
1/29/78	NFS NFS	Transmit NFS and Site comments/questions to GE Begin Tech. Spec. con. Safety Evaluation and Cover Letter
2/30/78	GE NFS	NFS receives final licensing Submittal and answers to CECo questions from GE Complete final NFS review licensing Submittal and answers to CECo questions
1/1/78	NFS	Transmit complete package on/off site review
1/3/78	Station	On-site review completed
1/6/78	PSA	Off-site review completed
1/11/78	NLA	Completed licensing package received by NRC
1/12/79	-	Anticipated unit shutdown
1/9/79	-	Receipt of operating License
3/9/79	-	Anticipated Unit Startup - Assumes 56 day outage 8 weeks

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Prepared by _____ NFS/DVA

Date: 12/23/77

PRELIMINARY CHECKLIST FOR RELOAD LICENSE AGREEMENTS

UNIT: Quad-Cities 1 ;
 RELOAD: 4 ;
 CYCLE: 5 ;

Item	Page	Require Changes
Screw Reactivity	4	Generalize wording and reference the submitt. NEDO-XXXX
Safety Valve Setpoints LSSS	1.2/2.2-1	None. Adequate pressure margin.
Bases	1.2/2.2-2,3	None, if the peak vessel pressure is 1325 psig. during S.V. sizing trans.
REM Setpoints LCO	3.2/4.2-14 3.2/4.2-7	Change to (.65w+XX) as req'd. Change operability to XX's Change Reference 1 to NEDO-XXXX.
Bases	3.2/4.2-8	
Auto Flow Control LCO	3.3/4.3-5	None. Stability analysis not limiting.
Bases	3.3/4.3-11	None.
MAPLHCR LCO	Fig. 3.5.1 (shfts. 1 to 3)	*Revise curves to reflect new analyses.
Bases	3.5/4.5-14	*Change references to reflect new analyses of NEDO-24046.
MCPB LCO	3.5/4.5-12	New values: **1.XX (7 x 7) 1.XX (8 x 8)
Bases	3.5/4.5-14	Generalize description of limiting transient(s).

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*LHCR changes are being handled under separate cover.
 includes additional 0.XX CPR penalty for Fuel Loading Error Accident

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-832
Revision 1
March 1978

- * 1. Unit: 2 Reload: 4 Cycle: 5 (next outage)
2. Scheduled date for next refueling shutdown: September 30, 1979 (shutdown EOC4)
3. Scheduled date for restart following refueling: January 20, 1980 (Startup EOC5)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: Similar Tech. Spec. changes to Reload 3 Cycle 4.
5. Scheduled date(s) for submitting proposed licensing action and supporting information: Reload Submittal to be provided approximately 90 days prior to shutdown.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
New fuel designs: Retrofit 8 x 8 fuel (approximately 196).
7. The number of fuel assemblies:
a. Number of assemblies in core: 724
b. Number of assemblies in spent fuel pool: 745
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
a. Licensed storage capacity for spent fuel: 1460
b. Planned increase in licensed storage: None.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: Last refueling date at present capacity: September, 85.

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RELOAD LICENSING PACKAGE
PREPARATION SCHEDULE

GTP 300-533
Revision 1
March 1978

287 415

UNIT OC 2
RELOAD
CYCLE

DATE RESPONSIBILITY CENTER

ACTIVITY

10/6/77	GE NPS	NPS receives draft Licensing Submittal from GE Transmit copy of draft to Station for comments.
10/20/77	NPS NPS	Transmit NPS and Site comments/questions to GE Design Tech. Spec. changes, Safety Evaluation and Cover Letter.
11/5/77	GE NPS	NPS receives final Licensing Submittal and answers to CECo questions from GE. Complete final NPS review of Licensing Submittal and answers to CECo questions.
11/16/77	NPS	Transmit complete package for on/off site review
11/16/77	Station	On-site review completed
11/18/77	PSA	Off/site review completed
12/1/77	PLA	Completed Licensing package received by NPS
1/16/78	-	Anticipated unit shutdown 28 days 90 days
3/5/78	-	Receipt of operating License
3/15/78	-	Anticipated Unit Startup - Assumes 8 Weeks Day outage

Prepared by JAS NFS/GWR

Date 2/21/78

VIII GLOSSARY

The following abbreviation which may have been used in the Monthly Report, are defined below:

CRD	-	Control Rod Drive System
SBLC	-	Standby Liquid Control System
MSIV	-	Main Steam Isolation Valve
RHRS	-	Residual Heat Removal System
RCIC	-	Reactor Core Isolation Cooling System
HPCI	-	High Pressure Coolant Injection System
SRM	-	Source Range Monitor
IRM	-	Intermediate Range Monitor
LPRM	-	Local Power Range Monitor
APRM	-	Average Power Range Monitor
TIP	-	Traveling Incore Probe
RBCCW	-	Reactor Building Closed Cooling Water System
TBCCW	-	Turbine Building Closed Cooling Water System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
HEPA	-	High-Efficiency Particulate Filter
RPS	-	Reactor Protection System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
RBM	-	Rod Block Monitor
BWR	-	Boiling Water Reactor
ISI	-	In-Service Inspection
MPC	-	Maximum Permissible Concentration

PCI	-	Primary Containment Isolation
SDC	-	Shutdown Cooling Mode of RHRS
LLRT	-	Local Leak Rate Testing
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
R.O.	-	Reportable Occurrence
DW	-	Drywell
RX	-	Reactor
EHC	-	Electro-Hydraulic Control System
MCPB	-	Minimum Critical Power Ratio
PCIONR	-	Preconditioning Interim Operating Management Recommendations
LER	-	Licensor Event Report
ANSI	-	American National Standards Institute
NIOSH	-	National Institute for Occupational Safety and Health
ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring