



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NRC PDR

May 31, 1979

Docket No.: 50-313

Mr. William Cavanaugh, III
Vice President, Generation
and Construction
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Dear Mr. Cavanaugh:

By Order of May 17, 1979, the Commission confirmed your undertaking a series of actions, both immediate and long term, to increase the capability and reliability of Arkansas Nuclear One, Unit No. 1 (ANO-1) to respond to various transient events. In addition, the Order confirmed that ANO-1 was shutdown and would not be restarted until the following actions had been accomplished:

- (a) Upgrade of the timeliness and reliability of the Emergency Feedwater System (EFW) by performing the items specified in Enclosure 1 of the licensee's letter of May 11, 1979. Provide changes in design for NRC review.
- (b) Develop and implement operating procedures for initiating and controlling EFW independent of Integrated Control System (ICS) control.
- (c) Implement a hard-wired control-grade reactor trip that would be actuated on loss of main feedwater and/or on turbine trip.
- (d) Complete analyses for potential small breaks and develop and implement operating instructions to define operator action.
- (e) Assign at least one Licensed Operator who has had Three Mile Island, Unit No. 2 training on the Babcock & Wilcox simulator to the control room (one each shift).

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By submittal of May 17, 1979, as supplemented by letters dated May 21, 22, 23, 24 and 29, 1979, you have documented the actions taken in response to the May 17 Order. I have reviewed this submittal, and am satisfied that, with respect to ANO-1, you have satisfactorily completed the actions prescribed in items (a) through (e) of paragraph (1) of Section IV of the Order, the specified modifications and analyses are acceptable, and the specified implementing procedures are appropriate. The bases for these conclusions are set forth in the enclosed Safety Evaluation.

As noted on page 5 of the Safety Evaluation you will be required to conduct a test during power operation to demonstrate operator capability to assume manual control of the EFW system independent of ICS. In addition, we have discussed the need for monitoring core exit temperature with your staff and they have agreed to provide a minimum of sixteen thermocouple indications of core exit temperature in the control room prior to startup. Also, your staff has agreed to provide an additional sixteen thermocouple indications of core exit temperature in the control room by October 31, 1979.

Appropriate Technical Specifications for Limiting Conditions for Operation and for surveillance requirements should be developed as soon as practicable and provided to the staff within seven days with regard to the design and procedural changes which have been completed in compliance with the provisions of the May 17, 1979 Commission Order. The revised Technical Specifications should cover:

- (1) Changes to the EFW System;
- (2) Plant alignment changes made to ensure control of the EFW independent of the ICS;
- (3) Addition of the Anticipatory Reactor Trip; and
- (4) EFW capacity.

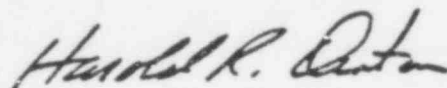
We note that by letter dated April 24, 1979, you have submitted proposed Technical Specifications for changes in setpoints for high pressure reactor trip and pilot operated relief valve actuation.

Also by letter dated May 16, 1979 you have submitted proposed changes to the Technical Specifications which define limiting conditions of operation upon loss of EFW equipment.

Within 30 days of receipt of this letter, you should provide us with your schedule for completion of the long-term modifications described in Section II of the May 17 Order.

My finding of satisfactory compliance with the requirements of items (a) through (e) of paragraph (1) of Section IV of the Order will permit resumption of operation in accordance with the terms of the Commission's Order; it in no way affects your duty to continue in effect all of the above provisions of the Order pending your submission and approval by the Commission of the Technical Specification changes necessary for each of the required modifications.

Sincerely,



Harold R. Denton, Director
Office of Nuclear Reactor
Regulation

Enclosure:
Notice of Authorization
to Resume Operation
Safety Evaluation
cc w/enclosure:
See next page

Arkansas Power & Light Company

cc w/enclosure(s):

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Honorable Ermil Grant
Acting County Judge of Pope County
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Director, Technical Assessment
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U. S. Environmental Protection Agency
Region VI Office
ATTN: EIS COORDINATOR
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First International Building
Dallas, Texas 75270

Director, Bureau of Environmental
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4815 West Markham Street
Little Rock, Arkansas 72201

UNITED STATES NUCLEAR REGULATORY COMMISSION
ARKANSAS POWER & LIGHT COMPANY
DOCKET NO. 50-313
NOTICE OF AUTHORIZATION TO RESUME OPERATION

The United States Nuclear Regulatory Commission issued an Order on May 17, 1979 (44 FR 29997, May 23, 1979), to Arkansas Power & Light Company (the licensee), holder of Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1), confirming that the licensee accomplish a series of actions, both immediate and long term, to increase the capability and reliability of ANO-1 to respond to various transient events. In addition, the Order confirmed that the licensee would maintain ANO-1 in a shutdown condition until the following actions had been satisfactorily completed:

- (a) Upgrade of the timeliness and reliability of the Emergency Feedwater (EFW) System by performing the items specified in Enclosure 1 of the licensee's letter of May 11, 1979. Provide changes in design for NRC review.
- (b) Develop and implement operating procedures for initiating and controlling EFW independent of Integrated Control System control.
- (c) Implement a hard-wired control-grade reactor trip that would be actuated on loss of main feedwater and/or on turbine trip.
- (d) Complete analyses for potential small breaks and develop and implement operating instructions to define operator action.
- (e) Assign at least one Licensed Operator who has had Three Mile Island Unit No. 2 training on the Babcock & Wilcox simulator to the control room (one each shift).

By submittal of May 17, 1979, as supplemented by letters dated May 21 and 22, 1979, the licensee has documented the actions taken in response to the May 17, Order. Notice is hereby given that the Director of Nuclear Reactor Regulation (the Director) has reviewed this submittal and has concluded that the licensee has satisfactorily completed the actions prescribed in items (a) through (e) of paragraph (1) of Section IV of the Order, that the specified modifications and analyses are

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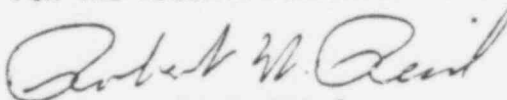
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acceptable and the specified implementing procedures are appropriate. Accordingly, by letter dated May 31, 1979, the Director has authorized the licensee to resume operation of ANO-1. The bases for the Director's conclusions are more fully set forth in a Safety Evaluation dated May 31, 1979.

Copies of (1) the licensee's letters dated May 17, 21 and 22, 1979, (2) the Director's letter dated May 31, 1979, and (3) the Safety Evaluation dated May 31, 1979 are available for inspection at the Commission's Public Document Room at 1717 H Street, N. W., Washington, D. C. 20555, and are being placed in the Commission's local public document room at the Arkansas Polytechnic College, Russellville, Arkansas. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Dated at Bethesda, Maryland
this 31st day of May 1979.

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