

Westinghouse
Electric Corporation

Power Systems
Company

Box 355
Pittsburgh Pennsylvania 15230

March 8, 1979

SP-79-113

(FSS-79-140)

Dr. K. H. Sun
EPRI PMG Member, FLECHT-SEASET Program
Safety and Analysis Department
Nuclear Power Division
Electric Power Research Institute
P.O. Box 10412
Alto, California 94303

Mr. E. H. Davidson
NRC PMG Member, FLECHT-SEASET Program
Systems Engineering Branch
Division of Reactor Safety Research
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

SUBJECT: FLECHT-SEASET Program
Informal Monthly Progress Report - January 1979
Contract: NRC-04-77-127, EPRI No: RP959-1

Gentlemen:

Attached is an informal progress letter for the month of January 1979 for FLECHT-SEASET.

Sincerely,

WESTINGHOUSE ELECTRIC CORPORATION
NUCLEAR TECHNOLOGY DIVISION

H. W. Massie, Jr.
H. W. Massie, Jr.
FLECHT-SEASET Project Engineer
Strategic Projects

jb

Attachment

Dr. K. H. Sun (EPRI) 12L, 12A
Mr. E. H. Davidson (NRC) 1L, 1A

NRC Research and Technical
Assistance Report

349 349

7907120/23

FLECHT-SEASET PROGRAM
CONTRACT NRC-04-77-127
PROGRESS LETTER

cc:

Mr. S. Levine, Director
Office of Nuclear Regulatory Research
Nuclear Regulatory Commission
Washington, D.C. 20555

Dr. T. E. Murley, Director
Division of Reactor Safety Research
Nuclear Regulatory Commission
Washington, D.C. 20555

Dr. S. H. Hanauer
Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. R. L. Tedesco
Nuclear Regulatory Commission - DSS
Washington, D.C. 20555

Mr. R. F. Fraley, Executive
Secretary ACRS
Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. P. Litteneker
Idaho Operations Office
P. O. Box 2108
Idaho Falls, ID 83401

Mr. J. O. Zane, Manager
Semiscale Program
INEL
550 Second Street
Idaho Falls, ID 83401

Mr. G. Sozzi
General Electric Company
175 Curtner Avenue
San Jose, CA 95125

Dr. J. H. Holderness
Combustion Engineering, Inc.
Nuclear Power Department
P. O. Box 500
Windsor, Connecticut 06095

Dr. B. Bingham
Babcock & Wilcox Company
P. O. Box 1206
Lynchburg, VA 24505

Dr. L. S. Tong, Assistant Director
for Water Reactor Safety Research
Division of Reactor Safety Research
Nuclear Regulatory Commission
Washington, D.C. 20555

Dr. P. A. Lottes
Argonne National Laboratory
9700 South Cass Avenue
Argonne, IL 60439

Dr. J. A. Dearien, Manager
Code Verification & Applications Program
EG&G Idaho, Inc.
P.O. Box 1625
Idaho Falls, Idaho 83401

Dr. Peter Griffith
Department of Mechanical Engineering
MIT
Cambridge, Massachusetts 02139

Dr. D. C. Groeneveld
Chalk River Nuclear Laboratories
Chalk River
Ontario, Canada K0J1J0

Dr. D. A. Powers
Division of System Safety
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Professor W. Y. Chon
Dept. of Engineering Science
Aerospace Engineering & Nuclear Eng.
State University of New York
Buffalo, New York 14214

Mr. Wayne Hodges
Nuclear Regulatory Commission - DSS
Washington, D.C. 20555

Mr. E. L. Halman, Director
Division of Contracts
Nuclear Regulatory Commission
Washington, D.C. 20555

Dr. James F. Jackson
Associate Division Leader for
Reactor Safety
Energy Division (Mail Stop 555)
Los Alamos Scientific Laboratory
P.O. Box 1663
Los Alamos, New Mexico 87545

Mr. R. Jensen
Intermountain Technology
Box 1604
Idaho Falls, ID 83401

Mr. W. Korchner
Los Alamos Scientific Laboratory
of the University of California
Los Alamos, New Mexico 87545

Dr. P. North Manager
Code Development & Analysis Program
EG&G Idaho, Inc.
P.O. Box 1625
Idaho Falls, Idaho 83401

Dr. David G. Thomas, Manager
PWR BDHT Program
Oak Ridge National Laboratory
P. O. Box Y
Oak Ridge, Tennessee 37830

Mr. W. Farmer
Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. W. Kayser
Exxon Nuclear
2101 Horn Rapids Road
Richland, Washington 99352

Mr. G. E. Wilson (TSB)
EG&G Idaho Inc.
550 Second Street
Idaho Falls, Idaho 83401

Dr. H. Sullivan
EG&G Idaho, Inc.
P.O. Box 1625
Idaho Falls, Idaho 83401

Dr. Robert Fujita
Energy Division
Group Q-9 (Mail Stop 559)
Los Alamos Scientific Laboratory
P.O. Box 1663
Los Alamos, New Mexico 87545

Professor R. A. Seban
Department of Mechanical Eng.
University of California
Berkeley, CA 94720

Professor I. Catton
Dept. of Chemical, Nuclear and
Thermal Engineering
University of California
Los Angeles, CA 90024

Mr. G. F. Brockett
Intermountain Technologies, Inc.
P.O. Box 1604
Idaho Falls, Idaho 83401

Dr. G. E. Dix
Nuclear Energy Division, M/C 583
General Electric Company
175 Curtner Avenue
San Jose, CA 95125

Mr. K. V. Moore
Energy Incorporated
P.O. Box 736
Idaho Falls, Idaho 83401

Dr. Z. Rosztoczy
Nuclear Regulatory Commission - DSS
Washington, D.C. 20555

Mr. T. Catton
EG&G Idaho, Inc.
P.O. Box 1625
Idaho Falls, Idaho 83401

Dr. S. Fabric
Nuclear Regulatory Commission - RSR
Washington, D.C. 20555

Dr. D. A. Prelewicz
NUS Corporation
4 Research Place
Rockville, MD 20850

FLECHT-SEASET PROGRAM

Contract: NRC-04-77-127
EPRI Project No: RP959-1

INFORMAL MONTHLY PROGRESS REPORT

JANUARY 1979

PROGRAM MANAGEMENT - H. W. MASSIE, JR.

A PMG meeting was held on January 16-18, 1979, (at Palo Alto) to attempt to resolve funding on the 21 Rod Bundle Task and complete the contract modification language. The PMG agreed to initiate processing of a contract modification that would replace the Single Tube Task with a 21 Rod Bundle Task and delete the Alternate ECC portion of the Systems Effects Task. No authorization was given Westinghouse to continue work on the 21 Rod Bundle Task beyond the PMG set limit equal to the original budget of the Single Tube Task. Based on this, Westinghouse was forced to stop work on this task until completion of the contract modification. Preliminary language for a contract modification was agreed to by the PMG, and Westinghouse has the action to redo its June 1978 cost proposal. It is currently anticipated that work can restart in April.

Work effort is being redirected to the 161 Rod Blocked Bundle Task (from the 21 Rod Bundle Task) to optimize use of program human resources and to minimize startup costs on the 21 Rod Bundle.

TEST PLANNING AND ANALYSIS - L. E. HOCHREITER/R. P. VIJUK

Unblocked Bundle Task (Task 3.2.1)

Shakedown testing is complete. Problems with window replacement, window sealing problems, DP zero shift and turbine meter recalibration have delayed matrix testing, but the first matrix test was run January 30, 1979.

Westinghouse, EPRI, and EG&G mutually developed sequencing strategy which attempts to minimize facility changeovers, provides time between runs to analyze movies, and obtains high priority data early in the testing series.

Work on upgrading data reduction codes continued. CATALOG and FFLOWS changes are nearly complete. A comparison of the DATA code with the other DATARH version used to reduce the cosine and skew data was completed. The DATAR method was accepted for FLECHT-SEASET usage because it calculates the same averaged heat transfer coefficient with savings in required computer time.

Work continued this month to develop an error analysis method for the FFLOW computer program. This study will result in the development of a error analysis subroutine for the FFLOW mass balance computer code. Work is now beginning on the sensitivity study using the most important input and output variables that were selected from all variables.

Three prototype heater rods are now available for testing. Testing is scheduled to start in the middle of February.

21-Rod Bundle Flow Blockage Task (Task 3.2.2)

EPRI provided their detailed comments on the 21-rod bundle task plan draft report. The appropriate modifications were made and subsequently reported to EPRI at a meeting on January 26, 1979. However, per further discussion received at this review meeting, additional modification of the task plan report is being done.

The shakedown test prospectus, describing the tests to be performed prior to matrix testing in order to qualify the test facility for proper operation, was issued to Facility Engineering.

A memo describing the computer program required for the reduction of data in the hydraulic characteristics test was issued. The purpose of this program is to calculate the bundle friction factor, grid pressure loss of coefficient, and flow blockage loss coefficient utilizing bundle pressure differential data during isothermal steady-state water flow tests. This program could also be utilized in the large 161-Rod Blocked Bundle Task.

Assembly of the sleeve attachment test on a single rod was completed and is available for testing. For this test, 20 mil diameter thermocouples were successfully brazed to the short concentric sleeves. In addition, a 21-rod steam probe prototype was installed in the center grid.

As a result of the discussions with EPRI, work has been initiated on a TAP-A model to determine the effect of a "quenched" flow blockage sleeve on the response of heater rod thermocouples immediately downstream of the sleeve.

Steam Generator Separate Effects Tests (Task 3.2.7)

Test photographic requirements were issued to Facilities Engineering for the movies and still shots for the steam generator tests. The high speed movie camera was run in a mock test to check performance. The film was under exposed and further tests will be run at lower camera speeds. Requirements for pre-test and post test data collection and the test run sequence were also issued to Forest Hills Facility Engineering.

During the shakedown tests to verify the vortex meter accuracy evidence of boiler moisture carryover was noticed. Subsequent tests confirmed that at high steam flows (2250 lbs/hr) there was 7% moisture in the boiler steam. The boiler vender was consulted and he recommended chemically cleaning the boiler with a detergent solution. The boiler was cleaned and flushed out and subsequent tests indicated no measurable carryover.

Three mixer characteristics shakedown tests and five single phase tube wall T/C calibration tests were run. A preliminary review of the data from these tests indicates that the tests are valid. During these tests, the vortex meters were inoperative. The boiler flow was measured with the water meter in the feedline and a stop watch. The water meter was calibrated and it's as accurate as the vortex meters.

The bundle bypass option of the SGFLOWS program was debugged. Several additions and modifications to the original version of the program were required. The code will be used to reduce the mixer characteristics shakedown tests.

The temperature interpolation program SGTEMP was modified by transforming the elevation parameter, z , to the $\log z$ before entering the temperature interpolation subroutine. This transform helps to eliminate overshoot and oscillation in the interpolated temperatures.

Systems Effects Tests (Task 3.2.8)

Work has been continued on upper plenum design considerations. It was found that the knowledge of upper plenum velocity field was necessary for a design of hot leg nozzles and the determination of upper plenum height. Available codes which could be used to predict the hot leg and upper plenum are being investigated.

In the mean time a computer program to plot velocity fields has been developed based on previous works. This will be helpful to visualize the fluid flow in the upper plenum. Also, dimensional analysis of upper plenum flows was initiated to provide more theoretical bases.

A preliminary instrumentation plan has been issued to Facility Engineering for review. This overall instrumentation plan indicates that in order to supply all needed information for the mass and energy balance equations. The detailed draft instrumentation plan is now being prepared to be issued.

FACILITY ENGINEERING/TEST OPERATIONS - L. R. KATZ/C. E. FUCHS

Unblocked Bundle (Task 3.2.1)

The quartz windows have been installed, the window flanges heaters were installed and the housing and plenums have been insulated. The housing D/P manifold and the upper plenum D/P were bled again. The camera platforms were completed. The platforms for the timers and mirrors were fabricated. They have not been installed.

21 Rod Bundle

Work on the 21 Rod Bundle Task was stopped until the completion of a contract modification.

At the time of the work stoppage, considerable progress had been made in loop fabrication. As previously reported, the main injection pipe line, steam injection line, and several rupture disc lines were installed. All major loop and internals components had been received. Since the last reporting period, the backup housing, the upper and lower plenums, steam separator and exhaust lines were installed in the loop. The axial DP cell manifold was assembled and is ready for installation.

Loop instrumentation schematics were completed. Wiring on the thermocouple (T/C) patch panel and loop T/C wiring has started along with differential pressure and pressure transducer patch panels.

Steam Generator Separate Effects (Task 3.2.6)

Failure of the Vortex steam flow rate meter electronics slowed down shake-down progress. The meter electronics were returned to the vendor for repair; spare boards were also ordered. Both the spare and repaired boards are due in by the end of January. In the meantime, the steam generator T/C calibration tests with water injection have been completed. Also, the steam generator T/C calibration test with steam flow and the mixer characteristics tests were performed using the boiler house water meter (in lieu of the Vortex meters) to determine steam flow.

A problem with boiler carryover was resolved by using a chemical "boil-out" compound to remove oil in solution with the boiler water. Steam generator lower plenum divider plate and gasket leak rate checks were performed after the steam generator T/C calibration tests were completed. Leak rates were found to be negligible.

Twin Bundle Task

A drawing (1462EG6) and equipment specification (FEES-023) defining alterations to the small steam generator lower plenum were issued. Quotations for this modification were requested. Purchase orders for thermocouples, steam probe tubing, Grayloc fittings and sight glass windows were issued.

Detailed design drawings of the penthouse addition were received. The AE is presently obtaining construction quotations for the electrical and structural work. Design of the test vessel and internals was initiated and an analysis of the various methods to measure the water injection flow was also initiated.

The design specification for the new bundle power controller has begun.

PRELIMINARY

Accession No. _____

Contract Program or Project Title: **FLECHT SEASET**

Subject of this Document: **JANUARY Monthly status Report**

Type of Document: **Monthly status Report**

Author(s), Affiliation and Address: **H.W. Massie Jr. Project Engineer
Westinghouse Electric Corporation
P.O. Box 355, Pittsburgh Pa. 15230**

Contract No.: **NRC-04-77-127**

Date of Document: **3/8/79**

Date Transmitted to NRC: **5/30/79**

NRC Individual and NRC Office or Division to Whom Inquiries Should be Addressed:
**E. H. Davidson
Reactor Safety Research**

This document was prepared primarily for preliminary or internal use. It has not received full NRC review and approval. Since there may be substantive changes, this document should not be considered final.

This Document may be made Publicly Available:


Signature
(NRC Program or Project Sponsor or
Authorized Contractor Official)

U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

PRELIMINARY

349 356

NRC Research and Technical
Assistance Report