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JUN 29 1977

MEMORANDUM FOR: Domenic B. Vassallo, Assistant Director
for Light Water Reactors, DPM

FROM: R. L. Tedesco, Assistant Director
for Plant Systems, DSS

SUBJECT: SAFETY FEATURES ACTUATION SYSTEM OUTPUT RELAYS

Plant Name: Three Mile Island Unit No. 2
Docket No: 50-320
Licensing Stage: Operating License
Milestone No: N/A
Responsible Branch: LWR-4
and Project Manager: H. Silver
Requested Completion Date: N/A
Review Status: Complete

The attached enclosure was prepared by DSS:PS, Power Systems Branch. This enclosure addressed the safety features actuation system output relays which were noted in the published Safety Evaluation Report as an unresolved issue.

The information contained in the attached enclosure may be appropriately incorporated into Section 8.3.1 of a Supplement to the published Safety Evaluation Report.

We consider this issue resolved.

Original signed by
R. L. Tedesco
Robert L. Tedesco, Assistant Director
for Plant Systems
Division of Systems Safety

Enclosure:
As stated

cc: S. Hanauer S. Varga
R. Haineman H. Silver
R. Boyd J. Glynn
W. McDonald F. Rosa
F. Ashe

106 276

Contact:
F. Ashe

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| OFFICE | 492-7276 | DSS:PSB | DSS:PSB | DSS:PS | | | |
| SURNAME | FAShe:s1 | FRosa | RTedesco | | | | |
| DATE | 6/28/77 | 6/28/77 | 6/28/77 | | | | |

Generated Missiles and Their Secondary Effect in Nuclear Power Plants," has been developed. The Working Group draft of this Safety Guide was modified by the IAEA Technical Review Committee on Design which met in March 1977, and we are soliciting public comments on this modified draft. Comments on this draft received by August 31, 1977 will be useful to the U.S. representatives to the Technical Review Committee and Senior Advisory Group in evaluating its adequacy prior to the next IAEA discussion.

Single copies of this draft may be obtained by a written request to the Director, Office of Standards Development, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555.

(5 U.S.C. 522(a).)

Dated at Rockville, Maryland this 16th day of June 1977.

For the Nuclear Regulatory Commission.

ROBERT B. MINOGUE,
Director,
Office of Standards Development.

[FR Doc.77-18478 Filed 6-29-77; 9:45 am]

REGULATORY GUIDE 1.119 SURVEILLANCE PROGRAM FOR NEW FUEL ASSEMBLY DESIGNS

Withdrawal

The Nuclear Regulatory Commission staff has withdrawn Regulatory Guide 1.119, "Surveillance Program for New Fuel Assembly Designs," which was issued for comment in June 1976. In order to broaden the scope and data base to include existing fuel assembly designs, the staff now believes that fuel surveillance programs for nuclear power plants should be plant specific and handled on a case-by-case basis rather than in a detailed generic manner. Therefore, the staff's need for data from fuel surveillance programs for both existing and new fuel designs will be included in the planned update of Regulatory Guide 1.70, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants," and reflected in the planned revision to NUREG-75/087, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants."

Regulatory guides are developed to describe and make available to the public methods acceptable to the NRC staff for implementing specific parts of the Commission's regulations and, in some cases, to delineate techniques used by the staff in evaluating specific problems. Guides may be withdrawn when they are superseded by the Commission's regulations, when equivalent recommendations have been incorporated in applicable and approved codes and standards, or when changing methods and techniques have made them obsolete.

(5 U.S.C. 552(a).)

Dated at Rockville, Maryland this 20th day of June 1977.

For the Nuclear Regulatory Commission.

ROBERT B. MINOGUE,
Director.

Office of Standards Development.

[FR Doc.77-18477 Filed 6-29-77; 9:45 am]

[Dockets Nos. 50-29, 50-271 and 50-309]

YANKEE ATOMIC ENERGY CO. ET AL.

Issuance of Amendments to Facility Operating Licenses

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 40, 36, and 30 to Facility Operating Licenses Nos. DPR-3, DPR-28 and DPR-36, issued to Yankee Atomic Electric Company, Vermont Yankee Nuclear Power Corporation and Maine Yankee Atomic Power Company, respectively, which revised Technical Specifications for operation of the Yankee Nuclear Power Station (Yankee-Rowe) located in Rowe, Franklin County, Massachusetts; Vermont Yankee Nuclear Power Station located near Vernon, Vermont; and Maine Yankee Atomic Power Station located in Lincoln County, Maine. These amendments are effective as of their date of issuance.

These amendments revise the provisions in the Technical Specifications relating to controlled entry into high radiation areas.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to these actions, see (1) the applications for amendments dated March 9, 1977 (filed by Yankee Atomic Electric Company), March 30, 1977 (filed by Vermont Yankee Nuclear Power Corporation), and March 3, 1977 (filed by Maine Yankee Atomic Power Company), (2) Amendment No. 40 to License No. DPR-3 (3) Amendment No. 36 to License No. DPR-28, and (4) Amendment No. 30 to License No. DPR-36, and (5) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C. The above items related to Yankee-Rowe are available at the Greenfield

Public Library, 422 Main Street, Greenfield, Massachusetts; those items related to Vermont Yankee are available at the Brooks Memorial Library, 244 Main Street, Brattleboro, Vermont; and those items related to Maine Yankee are available at the Wiscasset Public Library Association, High Street, Wiscasset, Maine.

A copy of items (2) through (5) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 16th day of June 1977.

For the Nuclear Regulatory Commission.

ROBERT W. REID,
Chief, Operating Reactors
Branch No. 4, Division of
Operating Reactors.

[FR Doc.77-18475 Filed 6-29-77; 8:45 am]

* Docket No. 50-320

METROPOLITAN EDISON CO., ET AL.

Hearing

In the Matter of Metropolitan Edison Co., Jersey Central Power & Light Co. and Pennsylvania Electric Co. (Three Mile Island Nuclear Station, Unit No. 2).

The evidentiary hearing in this matter (originally announced at 42 FR 2139, Jan. 10, 1977) will resume on Tuesday, July 5, 1977 at 9:00 a.m. in the Nuclear Regulatory Commission Hearing Room, Fifth Floor, 4350 East-West Highway, Bethesda, Maryland.

So Ordered.

Dated at Bethesda, Maryland this 23rd day of June 1977.

ATOMIC SAFETY AND LICENSING BOARD,
EDWARD LUTON,
Chairman.

[FR Doc.77-18802 Filed 6-29-77; 8:45 am]

ADVISORY COMMITTEE ON REACTOR SAFEGUARDS

Notice of Meetings

EDITORIAL NOTE.—The following meetings were originally announced in the FEDERAL REGISTER of Tuesday, June 28, 1977 (42 FR 32856, 32858):

| Name of committee unit | Meeting dates | Announced in FR Document No. |
|--|-----------------------|------------------------------|
| Advisory Committee on Reactor Safeguards | July 14 to 16, 1977.. | 77-18470 |
| ACRS Subcommittee on Regulatory Activities | July 13, 1977..... | 77-18467 |
| ACRS Reactor Safety Research Subcommittee, Working Group No. 1, and Emergency Core Cooling Systems Subcommittee. |do..... | 77-18472 |