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APPLICATION FOR LICENSE TO EXPORT  
BYPRODUCT, SOURCE, OR SPECIAL NUCLEAR MATERIAL

Submit in Triplicate

Carefully Read Instructions on Back

1. DATE OF APPLICATION SEPT. 4, '79	2. APPLICANT'S REFERENCE NO. (if any) VARIOUS	3. COUNTRY OF ULTIMATE DESTINATION JAPAN
4. NAME OF APPLICANT MITSUILINE TRAVEL SERVICE OF AMERICA, INC. ON BEHALF STREET ADDRESS OF MITSUI & CO. (USA), INC. CITY, STATE, AND ZIP CODE (415) 873-3066 SO. SAN FRANCISCO, CA. 94080		5. ULTIMATE CONSIGNEE IN FOREIGN COUNTRY (Name and address) JAPAN ATOMIC ENERGY RESEARCH INSTITUTE 1-13, 1-CHOME, SHINBASHI, MINATO-KU TOKYO, JAPAN
6. INTERMEDIATE CONSIGNEE IN FOREIGN COUNTRY (Give name and address. If same as ultimate consignee, state "Same.") TOKYO SHIBAURA ELECTRIC COMPANY, LTD. 13-12, 3-CHOME, MITA, MINATO-KU TOKYO, JAPAN		7. IF PURCHASER IN FOREIGN COUNTRY IS OTHER THAN ULTIMATE CONSIGNEE, GIVE NAME AND ADDRESS. (If same, state "Same.") MITSUI & CO., LTD. 2-1, 1-CHOME, OTEMACHI CHIYODA-KU, TOKYO, JAPAN
8. (a) QUANTITY TO BE SHIPPED (See instructions on back) 1 GRAM URANIUM-235	(b) COMMODITY DESCRIPTION (Include chemical and physical form; for special nuclear material and byproduct material also specify isotopic content; if in a device, identify the device, manufacturer, and model number.) IN A MAXIMUM OF 300 NUCLEAR DETECTION INSTRUMENTS CONTAINING URANIUM ENRICHED TO 95% U-235.	

(c) SHIPPING AND PACKING PROCEDURES (Required for special nuclear material. See instructions on back.)

NONE

9. END USE OF COMMODITIES COVERED BY THIS APPLICATION: (Describe fully, stating what will be produced or manufactured, what service  
will be rendered, or the nature of the research that will be performed.) (See instructions on back for special nuclear material.)

TO BE INSTALLED AT EXPERIMENTAL REACTORS AT 1) 2-4, SHIRANE, SHIRAKATA, TOKAI-MURA,  
NAKAGUN, IBARAKI-KEN, 2) 1233 WATANUKI-CHO, TAKASAKI-SHI, GUNMA-KEN AND 3) AT 3607  
NIIBORI, NARITA-CHO, OHARAI MACHI, HIGACHI-IBARAKI-GUN, IBARAKI-KEN

10. The applicant, and any official executing this certificate on behalf of the applicant named in Item 4, certify that this application  
is prepared in conformity with Title 10, Code of Federal Regulations, Parts 30 and 36 (if for byproduct material) or Part 40 (if  
for source material), or Part 70 (if for special nuclear material), and Part 71 (for transport of radioactive material, if applicable)  
and that all information contained herein, including any supplements attached hereto, is true and correct to the best of their  
knowledge and belief.

INTERNATIONAL SFGRDS  
AND  
EXPORT/IMPORT

1979 SEP 10 AM 11 00

RECEIVED  
U.S. NRC

MITSUILINE TRAVEL SERVICE OF AMERICA, INC.  
(Applicant named in Item 4)

By: M. Yamazaki  
DISTRICT MANAGER

(Title of certifying official authorized to act on behalf of the  
applicant)

Warning: 18 U.S.C. Section 1001; Act of June 25, 1948: 62 Stat. 749; makes it a criminal offense to make a willfully false state-  
ment or representation to any department or agency of the United States as to any matter within its jurisdiction

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NAME AND ADDRESS OF SUPPLIER: GENERAL ELECTRIC: 175 CURTNER AVENUE  
SAN JOSE, CA. 95125

DATE OF PROPOSED COMPLETION OF FINAL SHIPMENT:WHENEVER REQUIRED

DATE OF PROPOSED FIRST SHIPMENT: WHENEVER REQUIRED

PROPOSED EXPIRATION DATE OF EXPORT LICENSE: THE LONGEST PERIOD OF TIME  
RECOGNIZED BY NRC.

FOR NUCLEAR REACTORS, THE DESIGN POWER LEVEL IN THERMAL OR ELECTRICAL WATTS:

EXPERIMENTAL REACTOR AT TOKAI-LAB: THERMAL 45 MW

EXPERIMENTAL REACTOR AT TAKASAKI-LAB:

EXPERIMENTAL REACTOR AT OHARAI-LAB: THERMAL 50-MW

DATE WHEN EQUIPMENT IS NEEDED ABROAD: WHENEVER REQUESTED

LIST OF ITEMS: URANIMUM-235

IN A MAXIMUM OF 300 NUCLEAR DETECTION INSTRUMENTS  
CONTAINING URANIUM ENRICHED TO 95% U-235.

POOR  
ORIGINAL

1056 007



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

September 11, 1979

Docket No. 50-409

LICENSEE: Dairyland Power Cooperative (DPC)  
FACILITY: LaCrosse Boiling Water Reactor (LACBWR)  
SUBJECT: SUMMARY OF MEETING HELD ON AUGUST 22, 1979

NRC and DPC representatives met in Bethesda, Maryland, on August 22, 1979, to review containment ventilation system dampers (ventilation system isolation valves) reliability. The meeting attendees are listed in the attachment.

By letter dated February 1, 1979, DPC responded to NRC letter dated November 29, 1978, titled "Containment Purging During Normal Plant Operation." The NRC cited two specific events that had occurred recently that raised several questions relative to potential failures of automatic isolation of the large diameter (up to 48 inches in diameter) purge penetrations which are used during power operation. In both of these events, the isolation signals required to automatically close the purge valves for containment integrity were manually overridden to allow containment purging with radiation levels above the signal normally required automatic valve closure. Our November 29, 1978 letter requested that utilities of operating plants provide a commitment to cease containment purging during reactor operation or justify continued purging. DPC noted in its letter of February 1, 1979, that the LACBWR containment building was originally designed for continuous ventilation. This feature was reviewed and found acceptable by NRC when the LACBWR was initially authorized for power operation.

The LACBWR containment building ventilation system has five 20-inch flanged butterfly valve-type dampers. The dampers are located inside the containment vessel. Four of the dampers would be used in the event of an accident involving release of radioactivity to the containment atmosphere to seal the containment ventilation system to prevent a potentially excessive release of radioactive fission products to the atmosphere. Two of the four dampers are in series in the inlet duct (redundant valves) and two are in the exhaust duct (redundant). The fifth damper is located in the recirculation duct and is normally closed but is designed to open when the four isolation valves close to permit internal recirculation if the structure is not pressurized or heated excessively.

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MEETING SUMMARY DISTRIBUTION:

Docket 50-409  
NRC PDR  
Local PDR  
ORB#2 Reading  
NRR Reading  
H. R. Denton  
E. G. Case  
D. Eisenhut  
R. Vollmer  
B. Grimes  
W. Gamill  
J. Miller  
L. Shao  
T. Carter  
D. Crutchfield  
D. Ziemann  
V. Noonan  
Seismic Review Group  
A. Schwencer  
T. Ippolito  
R. Reid  
G. Lainas  
P. Check  
R. Clark  
F. Pagano  
G. Knighton  
J. Shea  
H. Smith  
OELD  
OI&E(3)  
R. Fraley, ACRS(16)  
L. Nichols  
E. Reeves  
J. Shapaker  
T. Quay  
J. Kerrigan  
S. Brown  
J. Wetmore  
P. Tam(ACRS)  
J. R. Buchanan  
TERA

DPC presented additional information (and later agreed to document the details of the oral presentation) that supplemented their February 1979 submittal. The DPC presentation covered the items listed below:

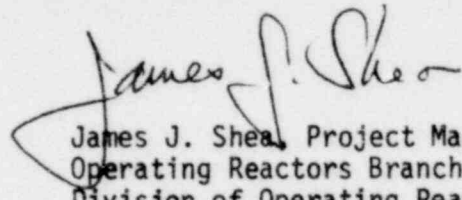
1. Basic Valve Design (Exhibits).
2. Application and Operating Parameters and Response.
3. Test History - Maintenance Experience.
4. Radiological Impact of Non-Ventilation.
5. Operational Impact of Non-Ventilation-Surveillance, Exposure.
6. Sensible Heat Rise with Non-Ventilation.
7. Reliability Conversations with Manufacturer.
8. Optional Test Program.
9. LOCA Qualifications Sensing and Operation Circuitry.
10. LOCA Qualifications for Valve.

The containment ventilation damper valves were designed originally to close on:

1. high radioactivity in the containment exhaust duct.
2. high reactor pressure.
3. high containment pressure.
4. loss of electrical power supply.
5. manual operation from control room.

September 11, 1979

DPC is currently negotiating to perform vent valve closure tests to demonstrate valve closure reliability. It is tentatively planned that the tests would be performed at Langley Field, Virginia, after reaching accord with the valve manufacturer, Allis Chalmers, and NRC. The NRC re-evaluation of containment ventilation dampers closure reliability to further enhance the health and safety of the public will continue when the additional information to be provided by DPC and the valve qualification test results are available for review.



James J. Shea, Project Manager  
Operating Reactors Branch #2  
Division of Operating Reactors

Attachment:  
List of Attendees

cc:  
See next page

1056 011

Dairyland Power Cooperative

- 4 -

September 11, 1979

cc

Fritz Schubert, Esquire  
Staff Attorney  
Dairyland Power Cooperative  
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La Crosse, Wisconsin 54601

O. S. Heistand, Jr., Esquire  
Morgan, Lewis & Bockius  
1800 M Street, N. W.  
Washington, D. C. 20036

Mr. R. E. Shimshak  
La Crosse Boiling Water Reactor  
Dairyland Power Cooperative  
P. O. Box 135  
Genoa, Wisconsin 54632

Coulee Region Energy Coalition  
ATTN: George R. Nygaard  
P. O. Box 1583  
La Crosse, Wisconsin 54601

Charles Bechhoefer, Esq., Chairman  
Atomic Safety and Licensing Board  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dr. George C. Anderson  
Department of Oceanography  
University of Washington  
Seattle, Washington 98195

Mr. Ralph S. Decker  
Route 4, Box 190D  
Cambridge, Maryland 21613

La Crosse Public Library  
800 Main Street  
La Crosse, Wisconsin 54601

Mr. Frank Linder  
General Manager  
Dairyland Power Cooperative  
2615 East Avenue South  
La Crosse, Wisconsin 54601

1056 012

ATTACHMENT  
LIST OF ATTENDEES  
AUGUST 22, 1979

DPC

H. A. Towsley  
L. J. Krajewski  
B. Angle  
R. Shimshak

NES Inc.

R. Milos

BNL

R. O. Smith

NRC

J. Shea  
L. Nichols  
E. A. Reeves  
J. W. Shapaker  
T. Quay  
J. Kerrigan  
S. Brown  
J. Wetmore  
P. Tam (ACRS)

1056 013