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DRAFT

RG 1.58  
REV 1

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VALUE/IMPACT ASSESSMENT ON QUALIFICATION OF

3

NUCLEAR POWER PLANT INSPECTION, EXAMINATION, AND TESTING PERSONNEL

4

SD TASK NUMBER: RS 901-5

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I. The Proposed Action

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A. Description

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The Commission's Regulations require that an applicant establish

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a quality assurance program that provides for indoctrination and

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training of personnel performing activities affecting quality as

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necessary to assure that suitable proficiency is achieved and

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maintained. The proposed action will provide updated guidance

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concerning the qualification of personnel who perform inspec-

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tions, examinations, and tests during fabrication, receipt at

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the construction site, construction, preoperational and startup

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testing, and the operation phase of nuclear power plants.

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B. Need for the Proposed Action

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Regulatory Guide 1.58, "Qualification of Nuclear Power Plant

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Inspection, Examination, and Testing Personnel," dated August

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1973, provides guidance on the qualifications of inspection,

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examination, and testing personnel and endorses ANSI Standard

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N45.2.6-1973. On August 31, 1978, a revised version of the

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standard was approved by the ANSI Board of Standards Review and

1 was designated ANSI N45.2.6-1978. The revision reflects  
2 increased experience in the qualification of nuclear power plant  
3 inspection, examination, and testing personnel. Some confusion  
4 has developed concerning the applicability of Regulatory  
5 Guide 1.58 and ANSI Standard N45.2.6 to certain categories of  
6 key personnel involved in preoperational and startup test pro-  
7 grams. Current NRC guidance should be updated to reflect exper-  
8 ience in the application of qualification criteria, to remove  
9 any confusion with regard to acceptance criteria, and to establish  
10 an NRC position on the approved National Standard.

11 C. Value/Impact of the Proposed Action

12 Guidance on the qualification of nuclear power plant inspection,  
13 examination, and testing personnel is currently contained in  
14 Regulatory Guide 1.58 (August 1973) and is being used by the NRC  
15 staff in the evaluation of applications for operating licenses.  
16 Since the purpose of the proposed action is to provide updated  
17 guidance to reflect experience with use of current guidance, to  
18 remove any confusion with regard to acceptance criteria, and to  
19 establish an NRC position on the approved National Standard, the  
20 value/impact will be based on changes proposed to guidance con-  
21 tained in Regulatory Guide 1.58 (August 1973).

1 The following is a list of the proposed changes to be made to  
2 Regulatory Guide 1.58 (August 1973) as a result of the revision  
3 of the standard and the associated value/impact assessment for  
4 each change. (The first two changes result in the deletion of  
5 two Regulatory Positions contained in the August 1973 version of  
6 Regulatory Guide 1.58. The last six changes concern the addition  
7 or modification of Regulatory Positions included in the proposed  
8 revision to the regulatory guide.)

- 9 1. The revised standard expands the applicability of its require-  
10 ments and recommendations from personnel who perform inspec-  
11 tions, examinations and tests during the construction phase  
12 to personnel who perform these activities during fabrication,  
13 receipt at the construction site, construction, preoperational  
14 and startup testing, and the operation phase. Therefore,  
15 Regulatory Position 1 in the August 1973 version of Regula-  
16 tory Guide 1.58 which expresses this applicability is unnecessary  
17 and has been deleted from the proposed revision. There is  
18 no change in the current position of Regulatory Guide 1.58  
19 and present regulatory bases will not be altered.
  
- 20 2. In the August 1973 version of Regulatory Guide 1.58, Regu-  
21 latory Position 3 provides additional guidance concerning  
22 the information to be included in the certificate of quali-  
23 fication under the category of "basis used for certification."

1 The revised standard incorporates this additional guidance  
2 that was lacking in the previous version of the standard.  
3 Therefore, this Regulatory Position is unnecessary and has  
4 been deleted from the proposed revision to the regulatory  
5 guide. There is no change in staff position and present  
6 regulatory bases will not be altered.

7 3. The revised National Standard states that it is applicable  
8 to personnel who perform preoperational and startup testing.  
9 Regulatory Guide 1.8 is being revised to encompass require-  
10 ments for personnel who perform preoperational and startup  
11 testing and will provide more definitive criteria for these  
12 personnel. Modifications to the requirements for the qualifi-  
13 cation of these personnel will be addressed in the value/impact  
14 assessment for the revision to Regulatory Guide 1.8. Regula-  
15 tory Position 1 of the proposed revision to Regulatory  
16 Guide 1.58 references Regulatory Guide 1.8 for the qualifica-  
17 tion of these personnel.

18 The value of the proposed action will be the clarification  
19 of the applicability of Regulatory Guide 1.58 and Regula-  
20 tory Guide 1.8 to personnel who perform preoperational and  
21 startup testing. This action will benefit the NRC staff by  
22 providing additional guidance for license evaluations. This  
23 action is a clarification of regulatory requirements. The  
24 additional guidance will remove any confusion concerning

1 qualification requirements of these personnel. Since there  
2 will be no change in the regulatory bases for license evalua-  
3 tions, the impact will be negligible.

4 4. The August 1973 version of Regulatory Guide 1.58 endorses  
5 ASNT Recommended Practice No. SNT-TC-1A as acceptable for  
6 the qualification of nondestructive test personnel for the  
7 test methods covered by that document. The proposed revi-  
8 sion to the regulatory guide endorses SNT-TC-1A-1975 in  
9 Regulatory Position 2. For those nondestructive examina-  
10 tions required by Section III and Section XI of the ASME  
11 Boiler and Pressure Vessel Code, SNT-TC-1A-1975 is recom-  
12 mended for use in conjunction with the additional provisions  
13 of the Code.

14 The proposed action results in clarification of the appli-  
15 cability of ASNT Recommended Practice No. SNT-TC-1A-1975  
16 and the ASME Boiler and Pressure Vessel Code to personnel  
17 performing nondestructive examinations and does not represent  
18 a change in NRC staff or industry practice. There should  
19 be no impact.

20 5. Regulatory Position 3 of the proposed revision to the regula-  
21 tory guide addresses the compatibility of the ANSI Standard  
22 N45.2.6-1978 and Section III and Section XI of the ASME Boiler

1 and Pressure Vessel Code for the qualification of personnel  
2 who perform inspection, examination, and testing at nuclear  
3 power plants. It is stated that ANSI N45.2.6-1978, subject  
4 to the exceptions of the regulatory position, should be used  
5 in conjunction with the ASME Code where the ASME Code does  
6 not address the requirements covered by ANSI N45.2.6-1978.  
7 The original version of the standard ANSI N45.2.6-1973 which  
8 is endorsed by Rev. 0 of Regulatory Guide 1.58 did not contain  
9 the Code exclusion statement and was considered applicable  
10 to Code covered activities. This new regulatory position  
11 does not represent a change from present staff practice.  
12 The impact should be minimal.

13 6. The revised standard, together with Regulatory Position 5  
14 of the proposed revision to the regulatory guide, defines  
15 the capability requirements for Level I, II, and III personnel  
16 in greater detail and does not represent a significant change  
17 in qualification requirements for inspection, examination,  
18 and testing personnel. The action provides additional guidance  
19 to the NRC staff and industry but does not alter present  
20 regulatory bases for license evaluations. Therefore, any  
21 impact should be minimal.

22 7. The education and experience recommendations presented in  
23 the standard for the qualification of nuclear power plant  
24 inspection, examination, and testing personnel have been

1 revised to reflect current experience in the use of the  
2 standard. The revised standard also encompasses more  
3 alternatives for meeting the education and experience  
4 provisions of the standard.

5 Regulatory Position 6 of the proposed revision to the  
6 regulatory guide states that a commitment to comply with  
7 the regulatory guide will mean that the education and  
8 experience recommendations of the standard will be  
9 followed.

10 The value to the NRC staff should be reduced effort in deter-  
11 mining the acceptability of various alternate proposals  
12 submitted by applicants. The revision of the education and  
13 experience recommendations reflects current experience in  
14 the use of the standard and does not represent a significant  
15 change in the regulatory basis for license evaluations.  
16 Therefore, the impact should be minimal.

17 8. The revised standard includes a statement that personnel  
18 not meeting the requirements of the standard may be used in  
19 data-taking assignments or in plant or equipment operation  
20 provided these personnel are under supervision by a qualified  
21 individual participating in the test. To prevent possible  
22 abuse of this authorization, Regulatory Position 7 of the  
23 proposed revision to the regulatory guide states that

1 personnel involved in inspections, examinations, or tests  
2 who do not meet the requirements of the standard should have  
3 sufficient training to ensure an acceptable level of com-  
4 petence in the performance of their activities.

5 The value to industry will be the ability to utilize lesser  
6 trained personnel for those tasks which require little formal  
7 training. This proposed action will not represent a significant  
8 change in present NRC staff practice. There should be no  
9 impact as a result of this position.

- 10 9. An important concept that is not addressed directly in ANSI  
11 N45.2.6-1978, ANST Recommended Practice No. SNT-TC-1A-1975,  
12 or the ASME Boiler and Pressure Vessel Code is that occupational  
13 radiation exposures should be maintained as low as is reasonably  
14 achievable (ALARA). To provide guidance in this area, Regulatory  
15 Position 8 of the <sup>P</sup>Proposed revision to the regulatory guide states  
16 that inspection, examination, and testing personnel who may be  
17 exposed to radiation fields during their activities should receive  
18 instruction in radiation protection and safety.

19 The value of the proposed action will be to provide increased  
20 awareness of the ALARA concept. Present practice within the  
21 industry should not be effected. Therefore, the impact to



1 industry should be minimal. Since the proposed action does  
2 not represent a change in NRC staff position, there will be  
3 no impact on NRC staff.

4 II. Technical Approach

5 This section is not applicable to this value/impact statement since  
6 the proposed action is an update of previously issued guidance.

7 III. Procedural Approach

8 Since the proposed action is an update of information contained in an  
9 existing regulatory guide, the only appropriate procedural alternative  
10 is a revision to the existing guide.

11 IV. Statutory Considerations

12 A. NRC Authority

13 This guide would fall under the authority and safety requirements  
14 of the Atomic Energy Act. In particular, under 10 CFR Part 50,  
15 Appendix B, which requires that the quality assurance program  
16 provide for indoctrination and training of personnel performing  
17 activities affecting quality as necessary to assure that suitable  
18 proficiency is achieved and maintained.

19 B. Need for NEPA Assessment

20 The proposed action is not a major action, as defined by 10 CFR  
21 51.5(a)(10), and does not require an environmental impact statement.

1 V. Relationship to Other Existing or Proposed Regulations or Policies  
2 Regulatory Guide 1.8, "Personnel Selection and Training," contains  
3 recommendations for the selection and training of nuclear power plant  
4 personnel. Regulatory Guide 8.10, "Operating Philosophy for Main-  
5 taining Occupational Radiation Exposures As Low As Is Reasonably  
6 Achievable," contains qualification and training recommendations for  
7 plant personnel for implementing the ALARA concept. The guidance of  
8 the proposed action will be consistent with the guidance furnished in  
9 these Regulatory Guides.

10 VI. Summary and Conclusions

11 A proposed revision to Regulatory Guide 1.58, "Qualification of Nuclear  
12 Power Plant Inspection, Examination, and Testing Personnel," has been  
13 prepared. The proposed revision endorses, with certain exceptions,  
14 ANSI N45.2.6-1978.