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> RG 1.33 REV 3

	VALUE/IMPACT ASSESSMENT ON
QUALITY	ASSURANCE REQUIREMENTS FOR THE OPERATION OF NUCLEAR POWER PLANTS
I. <u>The</u>	Proposed Action
Α.	Description
	Appendix B, "Quality Assurance Criteria for Nuclear Power Plants
	and Fuel Reprocessing Plants," to 10 CFR Part 50, "Licensing of
	Production and Utilization Facilities," establishes quality assur-
	ance requirements for the operation of nuclear power plant safety-
	related structures, systems, and components. The proposed action
	will provide updated guidance for compliance by the applicant
	with quality assurance program requirements for the operation
	phase of nuclear power plants.
Β.	Need for the Proposed Action
	Regulatory Guide 1.33, Revision 2, 'Quality Assurance Program
	Requirements (Operation)," dated February 1978, provides guid-
	ance on quality assurance program requirements for the operation
1 A S	phase of nuclear power plants and endorses National Standard ANSI
	N18.7-1976/ANS-3.2. With the implementation of the SD/IE Feed-
	back Program between the Offices of Standards Development and
	I. <u>The I</u> A.

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Inspection and Enforcement, several problem areas (discussed in Section C of this Preliminary Value/Impact Statement) with regard to guidance on quality assurance program requirements for the operations phase of nuclear power plants have been identified. Current NRC guidance should be updated to provide more concise guidance and reflect experience in the use of this Regulatory Guide.

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9 1. NRC 10 Guidance for complying with the Commission's Regulations 11 with regard to overall quality assurance program requirements 12 for the operation phase of nuclear power plants is currently 13 contained in Regulatory Guide 1.33, dated February 1978 and 14 is being used by the NRC staff in the evaluation of applica-15 tions for operating licenses. Since the purpose of the pro-16 posed action is to provide updated guidance to reflect 17 experience with the use of current guidance, the value/impact 18 will be based on changes proposed to guidance contained in 19 Regulatory Guide 1.33 (Revision 2,

Value/Impact of the Proposed Action

20 The following is a list of the significant changes to be made to 21 the Regulatory Guide and the associated value/impact assessment 22 for each change.

(a) Regulatory Position 1 will be reworded to specifically state that the identified procedures should be omitted only if they are not applicable to an applicant due to the configuration of the nuclear power plant.

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5 The present wording of the Regulatory Position allows for 6 the misinterpretation that the applicant can omit procedures 7 without careful justification.

8 The value of the proposed change will be less time spent by 9 the NRC staff informing applicants of the requirements for 10 having detailed written procedures. Since there is no change 11 in staff position, the impact will be minimal.

12 (1	b) To provide a more complete list of typical procedures in
13	Appendix A of the Guide, the following items will be added:
14	(1) Quality Assurance Program implementation procedures
15	(2) Inservice Inspection
16	(3) Hydrogen Explosions

17The addition of these procedures to Appendix A will provide18improved guidance concerning which activities should be19covered by written procedures. The value will be less time20spent by the NRC staff to inform applicants of those activities

which require detailed written procedures. Since the additional guidance is a present staff position, the impact will be negligible.

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(c) Identification of the types of instruments that require periodic calibration and procedural coverage will be clarified to specifically include both portable and installed equipment.

8 Subdivision 8.a of Appendix A is intended to apply to 9 installed instruments as well as portable equipment. ever, there is some confusion within the industry concerning 10 11 the application of the subdivision. The value of the change 12 will to the clarification of a present staff position. This 13 will reduce the amount of time spent by the NRC staff to 14 inform applicants of the types of instruments that require 15 periodic calibration and procedural coverage. The impact 16 will be negligible since there will be no change in staff 17 position.

(d) Clarification of the position that procedures should be
 prepared for all Technical Specification required surveil lance, whether included in the surveillance section or not,
 will be accomplished.

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The value of this action will be the removal of confusion that results from the use of older technical specifications which do not apply to Subdivision 8.b of Appendix A. The NRC staff will spend less time to inform applicants of the true staff position. The impact will be negligible since the result of this action is the clarification of a staff position.

(e) A change will be made to more clearly specify the need to establish procedures which cover all aspects of reactor engineering surveillance.

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- 11 The expansion of these procedures will provide additional 12 guidance to the industry. The value will be less time spent 13 by the NRC staff to inform applicants of the need for written 14 procedures in this area. Since there will be no change in the 15 staff position, the impact will be negligible.
- (f) Subdivision 9.a of Appendix A will be modified to state that
 even though minor, routine maintenance activities may not
 require detailed procedural coverage, they are still subject
 to general administrative procedural controls.

This change removes the misleading implication that major maintenance activities could be accomplished without procedures. The value of this proposed action will be less time spent by the NRC staff to inform applicants of activities which require detailed procedural coverage. Since this change does not effect the present staff position, there will be negligible impact.

8 (g) To remove the implication that only the activities listed 9 in Subdivision 9.d of Appendix A could be considered either 10 maintenance or operating procedures, it will be stated that 11 the items listed are only examples of such activities.

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12Due to the wording of this subdivision, confusion has arisen13within the industry concerning procedures that could be14categorized either as maintenance or operating procedures.

15The value will be less time spent by the NRC staff to inform16the applicant of the true intention of the subdivision.17The impact will be minimal since the change corresponds to18the present NRC staff position.

(h) Additional areas which should be addressed by administrative
 procedures will be included in Subdivision 9.e of Appendix A
 to assist in solving the problems of inadequate retest

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14		inform applicants individually of this
15		. this guidance is presently a staff position,
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18	tion	ic guidance for compliance with quality
19	assu	interests for the operation phase of nuclear
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2. Other Government Agencies

Not applicable, unless the government agency is an applicant, as TVA.

3. Industry

5 The changes to be made to Regulatory Guide 1.33 will not 6 alter present regulatory bases for license evaluations. 7 The value of the proposed action will be to provide more 8 specific guidance concerning quality assurance program 9 requirements. In addition to removing confusion within 10 the industry, the proposed action will facilitate the 11 development of a method acceptable to the NRC staff for 12 complying with the Commission's Regulations with regard 13 to quality assurance program requirements for the opera-14 tion phase of nuclear power plants. The expansion of 15 guidance presented in the Regulatory Guide may initially 16 appear to increase the requirements for compliance with 17 the Commission's regulations. However, since the additional guidance is presently used by the NRC staff in 18 19 license evaluation, the impact on industry will be minimal.

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Workers (ALARA Considerations)

Not Applicable.

5. Public

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2			The value to the public will be more efficient license appli-
3			cation preparation by industry which could reduce the cost
4			of nuclear power plants. Additional value would be more
5			efficient use of tax dollars as a result of the increased
6			guidance for license evaluations by the NRC staff.
7		٥.	Decision on Proposed Action
8			Updated guidance should be furnished on quality assurance prog- o
9			requirements for the operation phase of nuclear power plants.
10	II.	Tech	nical Approach
11		This	section is not applicable to this value/impact statement since

12 the proposed action is an update of previously issued guidance.

13 III. Procedural Approach

14	Since the proposed action is an update of information contained in an
15	existing regulatory guide, the only appropriate procedural alterna-
16	tive is a revision to the existing guide.

- 17 IV. Statutory Considerations
- 18 A. NRC Authority

19This guide would fall under the authority of the Atomic Energy Act.20In particular, under 10 CFR Part 50, Appendix B, which establishes

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- quality assurance requirements for the operation of nuclear power plant safety-related structures, systems, and components.
- B. <u>Need for NEPA Assessment</u>
 The proposed action is not a major action, as defined by 10 CFR
 51.5(a)(10), and does not require an environmental impact
 6 statement.
- 7 V. <u>Relationship to Other Existing or Proposed Regulations or Policies</u>
 8 The relationship of the guidance furnished by the proposed action to
 9 guidance provided by other Regulatory Guides is discussed in Regula 10 tory Position C.2 of Regulatory Guide 1.33.
- 11 VI. Summary and Conclusions

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A revision to Regulatory Guide 1.33, "Quality Assurance Program"
 Requirements (Operation)," should be prepared.