

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of )  
 )  
WISCONSIN ELECTRIC POWER COMPANY ) Docket No. 50-266  
(Point Beach Nuclear Plant, )  
Unit 1) )  
 )

ORDER MODIFYING CONFIRMATORY ORDER OF NOVEMBER 30, 1979

I

Wisconsin Electric Power Company (the Licensee) is the holder of Facility Operating License No. DPR-24 which authorizes the Licensee to operate the Point Beach Nuclear Plant, Unit 1, located in Two Creeks, Wisconsin, under certain specified conditions. License No. DPR-24 was issued by the Atomic Energy Commission on October 5, 1970, and is due to expire on July 25, 2008.

II

Inservice inspections of the Point Beach Unit 1 steam generators performed during August 1979 and October 1979 outages have indicated extensive general intergranular attack and caustic stress corrosion cracking on certain of the external surfaces of the steam generator tubes. The NRC Staff determined in November 1979 that additional operating conditions would be required to assure safe operation prior to resumption of operation of Point Beach Unit 1 from a refueling outage. Such conditions were imposed by Confirmatory Order for Modification of License dated November 30, 1979. In addition to those conditions, the Staff has now determined that additional conditions are required to provide continued assurance that Point Beach Unit 1 can be operated safely.

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These additional conditions are analyzed in a Staff Safety Evaluation Report, dated this date, which is attached to this Order. The Licensee has agreed to this condition by letter dated December 31, 1979.

III

Accordingly, pursuant to the Atomic Energy Act of 1954, as amended, and the Commission's Rules and Regulations in 10 CFR Part 2 and Part 50, IT IS HEREBY ORDERED THAT License No. DPR-24 be amended, in the manner hereafter provided, to include the following conditions in addition to those conditions listed in the Confirmatory Order of November 30, 1979:

1. Unit 1 will be operated at a reactor coolant pressure of 2000 psia with the associated parameters (i.e., overtemperature  $\Delta T$  and low pressurizer pressure trip point) with the limits indicated in the Safety Evaluation Report appended to this Order.
2. The licensee shall develop and follow the necessary procedures for operating Unit 1 at the conditions described in condition 1 above.

IV

In view of the above, this amendment of License No. DPR-24 is made immediately effective. Accordingly, within 48 hours of receipt of this Order, the Point Beach Unit 1 facility shall be operated at a reactor coolant system pressure of 2000 psia within the parameters described above.

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Any person whose interest may be affected by this Order may within twenty days of the date of this Order request a hearing with respect to this Order. Any such request shall not stay the effectiveness of this Order. Any request for a hearing shall be addressed to the Director of Nuclear Reactor Regulation, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555.

In the event a hearing is requested, the issues to be considered at such hearing shall be:

- 1) Whether the facts stated in Section II of this Order are correct;  
and
- 2) Whether this Order should be sustained.

FOR THE NUCLEAR REGULATORY COMMISSION



Harold Denton, Director  
Office of Nuclear Reactor Regulation

Attachment:  
Staff Safety Evaluation Report,  
dated January 3, 1980

Effective date: January 3, 1980  
Bethesda, Maryland

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO THE POINT BEACH UNIT 1 STEAM GENERATOR

TUBE DEGRADATION DUE TO DEEP CREVICE CORROSION

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-266

INTRODUCTION

Wisconsin Electric Power Company (the licensee) has requested changes to the Technical Specifications of Point Beach Units 1 and 2 to allow operation at either 2000 or 2250 psia (Reference 1). These changes include (1) defining over-temperature -  $\Delta T$  trip equation for each operating pressure, and (2) redefining the low pressure trip to allow adequate operating margin when operating at the lower pressure (2000 psia).

Although 2250 psia is the design operating pressure, both units have been previously operated at the lower pressure. A brief history of the previous operation of Point Beach Units 1 and 2 is given by the licensee in References 1 and 7 outlining the reasons for changing the pressure, the dates at which these changes were made and providing the references to the various Amendment requests for NRC and the subsequent Staff Safety Evaluation Reports. Presently both units are operating at 2250 psia. The licensee requested the change to permit operation at 2000 psia to reduce stress on the steam generator tubes.

This change to a lower pressure adversely affects the departure from nucleate boiling ratio (DNBR) and requires justification that the reactor is still adequately protected. The proposed change in the over temperature -  $\Delta T$  (OT $\Delta T$ ) trip provides this protection for some cases. For situations where the OT $\Delta T$  trip does not operate, adequate protection must be shown by other analysis. The loss of flow and rod drop events are two events in which DNBR protection is provided by means other than the OT $\Delta T$  trip.

Modification of the reactor low pressurizer pressure trip to provide more margin between the lower operating pressure and this trip also requires justification that the applicable criteria for transient and accident analyses are still satisfied.

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DUPLICATE DOCUMENT

Entire document previously  
entered into system under:

ANO

8001160270

No. of pages: 7