UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

DEC 3 1 1979

Docket Nos. 50-239
50-320

Metropolitan Edison Company
ATTN: Mr. R. C. Arnold, Sr.
Senior Vice President
100 Interpace Parkway
Parsippany, New Jersey 07054

Gentlemen:

The enclosed IE Information Notice No. 79-36 provides information on a computer code defect which can result in incorrect stress values for piping elbows.

Sincerely,

for Boyce H. Grier
Director

Enclosures:
1. IE Information Notice No. 79-36
2. List of Recently Issued IE Information Notices

CONTACT: L. E. Tripp
(215-337-5282)

cc w/encls:
J. G. Herbein, Vice President Nuclear Operations
E. G. Wallace, Licensing Manager
G. P. Miller, Manager, Unit 1
J. J. Colitz, Manager Plant Engineering (TMI-1)
J. J. Barton, Manager Site Operations
B. Elam, Manager Plant Engineering (TMI-2)
L. W. Harding, Supervisor of Licensing
R. F. Wilson, Director TMI-2
I. R. Finfrock, Jr.
R. W. Conrad
G. F. Trowbridge, Esquire
J. B. Lieberman, Esquire
Ms. Mary V. Southard, Chairperson, Citizens for a Safe Environment

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ENCLOSURE 1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

SSINS No.: 6870
Accession No:
7910250519

IE Information Notice No. 79-36
Date: December 31, 1979

COMPUTER CODE DEFECT IN STRESS ANALYSIS OF PIPING ELBOW

Description of Circumstances:

The NRC was informed on December 17, 1979 that a computer code, NUPIPE, used in the stress analysis of piping systems contained a defect which could result in incorrect stress values at one end of piping elbows. This defect was identified from discussions with a user of NUPIPE and the NUPIPE code developers. The stress calculation error can occur when a flexible joint is modeled at the end of an elbow (ELBOW-ELASTOJT connection in NUPIPE terminology). This defect in the NUPIPE code is in the transformation of the loads in the ELASTOJT coordinate axis to the piping ELBOW coordinate axis. The incorrect coordinate axis transformation may incorrectly interchange the torsion and bending moment loads on the piping elbow. This can result in the incorrect assignment of the stress intensification factor for the individual moments. The correct stress may be higher or lower than calculated by NUPIPE, depending on the loading condition.

Four conditions must exist for the possibility of the calculation of an incorrect stress in the NUPIPE code:

1. an USAS B31.1.0 - 1967 Code analysis is being done, and

2. an ELBOW-ELASTOJT connection exists, and

3. an absolute sum of two load cases is being calculated, and

4. the ELBOW torsional moment axis does not align with the ELASTOJT local x-axis.

Possible users of the NUPIPE code include the code developer, Stone and Webster, EG&G (Idaho Falls), and those using the Cybernet system. The NRC is currently reviewing the extent of usage of the NUPIPE code and the generic implications for other facilities. There is a potential for a similar defect to occur in other piping analysis computer codes.

This Information Notice is provided as early notification of a possible significant matter. It is expected that recipients will review the information for possible applicability to their facilities. No specific action is requested in response to this Information Notice. If NRC evaluations so indicate, further licensee actions may be requested or required. If you have any questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

No written response to this IE Information Notice is required.

ENCLOSURE 2

IE Information Notice No. 79-36
Date: December 31, 1979
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RECENTLY ISSUED IE INFORMATION NOTICES

Information Notice No.	Subject	Date Issued	Issued to
79-12A	Attempted Damage to New Fuel Assemblies	11/9/79	All Fuel Facilities, Research Reactors and Power Reactors with an Operating License (OL) or Construction Permit (CP)
79-27	Steam Generator Tube Ruptures at Two PWR Facilities	11/16/79	All Power Reactor Facilities with an OL or CP
79-28	Overloading of Structural Elements Due to Pipe Support Loads	11/16/79	All Power Reactor Facilities with an OL or CP
79-29	Loss of Nonsafety Related Reactor Coolant System Instrumentation During Operation	11/19/79	All Power Reactor Facilities with an OL or CP
79-30	Reporting of Defects and Noncompliances, 10 CFR Part 21	12/6/79	All Power Reactor Facilities with an OL or CP
79-31	Use of Incorrect Amplified Response Spectra (ARS)	12/13/79	All Power Reactor Facilities with an OL or CP
79-32	Separation of Electrical Cables for HPCI and ADS	12/21/79	All Power Reactor Facilities with an OL or CP
79-33	Improper Closure of Primary Containment Access Hatches	12/21/79	All Power Reactor Facilities with an OL or CP
79-34	Inadequate Design of Safety-Related Heat Exchangers	12/31/79	All Power Reactor Facilities with an OL or CP
79-35	Control of Maintenance and Essential Equipment	12/31/79	All Power Reactor Facilities with an OL or CP