

CONSUMERS POWER COMPANY  
DOCKET 50-155  
REQUEST FOR CHANGE TO THE TECHNICAL SPECIFICATIONS  
LICENSE DPR-6

For the reasons hereinafter set forth, it is requested that the Technical Specifications contained in Operating License DPR-6, Docket 50-155, issued to Consumers Power Company on May 2, 1964, for the Big Rock Point Plant, be changed as described in Section I below:

I. CHANGES

Section 3.7(b)

Replace phrase, "At least once every 12 months," with:

"Each reactor shutdown for refueling, but in no case at intervals greater than two years."

Section 3.7(c)

Replace phrase, "At least once every 12 months," with:

"Each reactor shutdown for refueling, but in no case at intervals greater than two years."

NOTE: Revised Technical Specifications pages are attached. Proposed changes are shown by a vertical line in the right-hand margin.

II. DISCUSSION

The above change in the surveillance interval is to reflect the interval required by Section XI, IWV and IWP of the Boiler and Pressure Vessel Code and 10 CFR 50, Appendix J.

III. CONCLUSIONS

Based on the foregoing, the Big Rock Point Plant Review Committee and Safety and Audit Review Board have reviewed these changes and find them acceptable.

CONSUMERS POWER COMPANY

By R B DeWitt  
R B DeWitt, Vice President-  
Nuclear Operations

Sworn and subscribed to before me this 7th day of January 1980.

Dorothy H. Bartkus  
Dorothy H Bartkus, Notary Public  
Jackson County

(Seal)

My commission expires March 26, 1983.

1735 192

8001110

436

PROPOSED  
TECHNICAL  
SPECIFICATIONS  
CHANGES FOR  
BIG ROCK POINT

1735 193

## 3.5.2 (Contd)

(b) Water addition to the containment sphere must be manually stopped before the accumulated water level reaches an elevation of 596 feet.

(c)

(Deleted)

3.6 CONTAINMENT REQUIREMENTS

Containment sphere integrity shall be maintained during power operation, refueling operation, shutdown and cold shutdown conditions except as specified by a system of procedures and controls to be established for occasions when containment must be breached during cold shutdown.

3.7 CONTAINMENT SPHERE LEAKAGE TESTING

For the purpose of this specification, leakage rate is defined as the percent of the contained atmosphere (weight basis) which escapes per day (24 hours) under the defined pressure conditions through any leaks in the containment boundary and all isolation valves and their associated piping.

The maximum allowable integrated leakage rate shall not exceed 0.5%/day of the containment atmosphere (weight basis) at the design pressure of 27 psig. The procedure for containment sphere leakage testing shall be:

- (a) At least once every 6 months, the personnel lock, the equipment lock and the sphere supply-and-exhaust ventilation valves shall be pressurized, with air to 20 psig, to test their leak tightness. The sum of leakage rates from these valves and locks shall be less than 0.25%/day of the containment atmosphere (weight basis) at 20 psig.
- (b) Each reactor shutdown for refueling, but in no case at intervals greater than two years, the following valves shall be tested for operability from both the manual and automatic modes of operation and, at the same time, shall be tested, for leak tightness by means of a pressure test utilizing air or the normal working fluid at a pressure not less than 20 psig:

Main Steam Isolation (MO-7050)

\*Main Steam Drain (MO-7065)

Clean-Up System Resin Sluice (CV-4091, CV-4092, CV-4093)

Reactor and Fuel Pit Drain Isolation (CV-4027, CV-4117)

Reactor Enclosure Clean Sump Isolation (CV-4031, CV-4102)

Reactor Enclosure Dirty Sump Isolation (CV-4025, CV-4103)

\*Operability, automatic controls, and instrumentation tests required only if valve is opened for use during operation.

PROPOSED

1735 194

## 3.7 (Contd)

All significant leaks (drops/second) revealed by these tests shall require repair of valve seals and retests.

Automatic controls and instrumentation associated with these valves shall be tested at approximately quarterly intervals; these tests may be conducted with a simulated signal or in such other manner as to obviate plant shutdown.

- (c) Each reactor shutdown for refueling, but in no case at intervals greater than two years, the following shall be visually examined for evidence of corrosion, cracking or deterioration:

All Electrical and Accessible Piping Penetration  
Nipple Welds  
All Accessible Piping Welds to Nipples  
All Expansion Joints and Welds on Expansion Joints  
Potting Compound in All Electrical Penetrations

Insulation at piping penetration welds shall be removed to permit visual examination.

The probable cause of any significant corrosion, cracking or deterioration revealed by such visual examination shall be determined, and evaluated in terms of likelihood of recurrence and probable effect upon other containment sphere penetration components. An individual component leak detection test shall be performed with air at 10 psig on the faulty component prior to its repair or modification. The faulty component, and other components if necessary, shall be repaired or modified, and an individual component leak detection test performed with air at 10 psig upon each repaired or modified component. All components so repaired or modified shall be visually reexamined at appropriate intervals, but not less frequently than once every six months, until the adequacy of annual visual inspection is reestablished to the operator's satisfaction.

After cutting into the sphere or its components, or any disassembly of components that would affect sphere integrity, an individual component leakage rate or an integrated leakage rate test, whichever is deemed more appropriate by the operator, shall be performed, with air at a pressure not less than 10 psig. It shall be permissible to employ a leak detection test in lieu of the above for insuring containment integrity following disassembly of the emergency condenser or the gasketed, bolted closure of the coaxial cable electrical penetrations.

The individual component leakage rate determined from the above tests when combined with the previously measured integrated

1735 195