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INFORMATION REPORT

For: The Commissioners

From: H. R. Denton, Director, Office of Nuclear Reactor Regulation

Thru: L. V. Gossick, Executive Director for Operations *TAR for L.V.G.*

Subject: INFORMATION REPORT - ISSUANCE OF NUREG-0313, REV. 1

Purpose: To inform the Commission of the issuance of Unresolved Safety Issue (USI) A-42 report, NUREG-0313, Rev. 1 "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping."

Discussion: As indicated in SECY-79-409A USI progress report to the Commission, I promised to transmit to you an information report on each USI task upon its completion. This information report fulfills that commitment and constitutes essentially the completion of USI Generic Task A-42, "Pipe Cracks in Boiling Water Reactors." This information report summarizes the NRC staff's revised technical positions established in NUREG-0313, Rev. 1 and plans for implementing these positions.

The attached "For Comment" edition of NUREG report updates and supersedes the NRC technical positions established in the original NUREG-0313, which was published in July 1977, by incorporating the recommendations made in the recent NRC Pipe Crack Study Group (PCSG) report, NUREG-0531, "Investigation and Evaluation of Stress-Corrosion Cracking in Piping of Light Water Reactor Plants."

This NUREG report sets forth the NRC staff's revised acceptable methods to reduce the intergranular stress corrosion cracking susceptibility of BWR ASME Code Class 1 & 2 pressure boundary piping and safe ends. Recognizing that complete compliance with the guidelines described in the report may not be practical, or even possible for all BWR plants, varying degrees of conformance to the staff's guidelines are provided in the report. For plants that cannot fully comply with the specified material selection, testing, and processing guidelines, varying degrees of augmented inservice inspection and leak detection requirements are presented.

The staff has determined that NUREG-0313, Rev. 1 should be issued for a 60-day public comment period and has prepared, for transmittal to the Office of the Secretary, the information necessary for a Federal Register Notice regarding the issuance. A copy of the Notice is enclosed.

Contact: C. Y. Cheng, DOR
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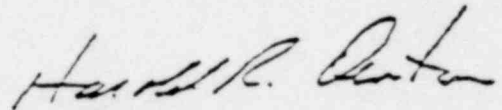
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At the completion of the 60-day period, the staff will evaluate the comments received and, if needed, will issue a supplement or revision to NUREG-0313, Rev. 1. The target for issuing the supplement or revision is late April 1980. Because of the need to review and resolve any comments received from the public, we do not anticipate the commencement of implementation until May 1980.

Concurrent with this public comment, we will submit NUREG-0313, Rev. 1 for review by the Advisory Committee on Reactor Safeguards. However, we do not intend to transmit this NUREG report to the staff's Regulatory Requirements Reviews Committee for review. This determination is based on the fact that (1) full participation of the representatives from various NRC Offices in the recent PCSG which published NUREG-0531 and the Task Group on Generic Task A-42 which is issuing this new NUREG report and (2) NUREG-0531 essentially provides all technical bases for NUREG-0313, Rev. 1. Copies of NUREG-0313, Rev. 1 will also be forwarded to the applicable Congressional Committees for their review.

We will keep you informed of the status of the staff's implementation efforts.



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Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Attachments:

1. NUREG-0313, Rev. 1 - Secy, PE, GC & Commission only
2. Federal Register Notice

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NUCLEAR REGULATORY COMMISSION
NUREG-0313, REV. 1
NOTICE OF ISSUANCE AND AVAILABILITY
TECHNICAL REPORT ON
MATERIAL SELECTION AND PROCESSING
GUIDELINES FOR BWR COOLANT PRESSURE BOUNDARY PIPING

A task force with members from the Nuclear Regulatory Commission (NRC) has prepared a report entitled "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping" (NUREG-0313, Rev. 1), dated October 1979. This report constitutes the resolution of the NRC's Generic Activity A-42, "Pipe Cracks in Boiling Water Reactors," which was an "Unresolved Safety Issue" pursuant to Section 210 of the Energy Reorganization Act of 1974.

This generic study was initiated because of the recent publication of NUREG-0531, "Investigation and Evaluation of Stress-Corrosion Cracking in Piping of Light Water Reactor Plants," by the NRC 1978 Pipe Crack Study Group (PCSG). The new Study Group was specifically chartered among others to reexamine the conclusions and recommendations of the 1975 PCSG report (NUREG-75/067) in view of cracks recently discovered in large diameter pipes, and to evaluate the significance of safe end cracking at Duane Arnold relative to similar material and design aspects at other facilities. Because of the new ideas and issues addressed in NUREG-0531, the implementation document NUREG-0313, which is based on the 1975 PCSG report, needs to be updated to incorporate the latest recommendations made by the 1978 PCSG.

NUREG-0313, Rev. 1 sets forth the NRC staff's revised acceptable methods to reduce the intergranular stress corrosion cracking susceptibility of BWR ASME Code Class 1 & 2 pressure boundary piping and safe end. For plants that cannot fully

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comply with the material selection, testing, and processing guidelines of this report, varying degrees of augmented inservice inspection and leak detection requirements are presented.

Public comments are being solicited from interested organizations, groups, and individuals. These comments will have bearing on final Commission action, particularly with regard to implementation.

Copies of the report will be available after October 1979. Copies will be sent directly to utilities, utility industry groups and associations, and environmental and public interest groups. Other copies will be available for review at the NRC Public Document Room, 1717 H Street, N. W. Washington, D.C. and the Commission's local public document room located in the vicinity of existing nuclear power plants. Addresses of these local public document rooms can be obtained by contacting the Chief, Local Public Document Room Branch, Mail Stop 309, Nuclear Regulatory Commission, Washington, D.C. 20555, telephone (301) 492-7356. A single copy of NUREG-0313, Rev. 1 will be provided free of charge, while the supply lasts, upon written request of a full participant in an ongoing NRC proceeding. This request must identify the requester as a participant and should be addressed to Director, Division of Technical Information and Document Control, Nuclear Regulatory Commission, Washington, D.C. 20555.

Comments should be forwarded to Dr. Stephen H. Hanauer, Director, Unresolved Safety Issues Program, Nuclear Regulatory Commission, Washington, D.C. 20555.

Dated at Bethesda, Maryland, this 17th day of October, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

Darrell G. Eisenhut
Darrell G. Eisenhut, Acting Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

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