

CONTROL BLOCK:

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7	8	9						14						25						30						57				58	
		LICENSEE CODE												LICENSE NUMBER												LICENSE TYPE					

0	1	REPORT SOURCE		L	6	0	5	0	0	0	3	3	3	7	1	1	1	8	7	9	8	1	2	1	7	7	9	9	
7	8			60	61	DOCKET NUMBER										68	69	EVENT DATE				74	75	REPORT DATE					80

0	2	SEE ATTACHMENT
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0	3	
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0	4	
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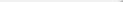

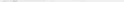
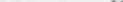
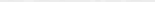

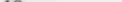
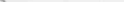
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0	6	
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07 | Page

08 |

7 8 9

(17) LER NO. REPORT NUMBER: 79
 EVENT YEAR: 79
 SEQUENTIAL REPORT NO.: 106
 OCCURRENCE CODE: 03
 REPORT TYPE: L
 NO.: 0

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

X 18 X 19 Z 20 Z 21 0 0 0 0 Y 23 N 24 Z 25 Z 9 9 9

33 34 35 36 37 40 41 42 43 44

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 SEE ATTACHMENT

1	1	
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1	2	
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1	3
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1	4
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8 9 FACILITY STATUS 1 5 E 28 10 11 12 13 % POWER 0 7 5 29 OTHER STATUS NA 30 44 45 46 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Surveillance Test 32

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35)

1 6 2 33 4 34 NA

LOCATION OF RELEASE (36)

NA

PERSONNEL EXPOSURES

NUMBER			TYPE	DESCRIPTION
1	7	00	(37) Z	(38) NA

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	2	3	4	5	6
1	2	0	0	0	40
NA					

7 8 9 11 12
LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
1 9 Z (42) NA

8 9 10
 PUBLICITY
 ISSUED DESCRIPTION (45) 7912210 322 NRC USE ONLY
 2 0 N (44) NA
 7 8 9 10 68 69

NAME OF PREPARER W. Verne Childs

PHONE: (315) 342-3840

NRC USE ONLY

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-106/03L-0

Page 1 of 1

During normal steady state operation, while conducting Operations Surveillance Test F-ST-II titled "Main Steam Isolation Valve Limit Switch Instrument Functional Test" to satisfy the requirements of Technical Specification Appendix A, Table 4.1-1, MSIV 29-A0V-80A did not provide a half scram signal at equal to or less than 10% closure as required. In accordance with the requirements of Table 3.1-1 the instrument channel relays were placed in the tripped condition by removing the fuses associated with the relays. Since the limit switches for 29-A0V-80A are located inside the primary containment, the precise nature of the failure can not be determined at this time. When plant conditions permit investigation as to the cause of the failure, repair will be completed and a follow-up report submitted.

Since the trip system channel of concern is in a tripped condition and the other channels were shown to be fully operable, the event does not represent a significant hazard to the public health and safety.

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