VIRGINIA ELECTRIC AND POWER COMPANY

RICHMOND, VIRGINIA 23261

December 17, 1979

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. A. Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Serial No. 973
FR/RWC: mvc

Docket No.: 50-338
50-339

License No. NPF-4

Dear Mr. Denton:

PROCEDURAL CHANGES
REQUIRED TO ADDRESS CONTROL
ROD DROP ANALYSIS CONCERNS

Westinghouse Electric Corporation notified Vepco of an unreviewed safety question on November 15, 1979 concerning the potential for predicting more limiting results for single dropped rod events. Discussions with Westinghouse confirmed that the only Vepco plants affected were North Anna Units 1 and 2. Consequently, the appropriate Nuclear Regulatory Commission (NRC) notifications were made.

The event was investigated further by Vepco through detailed discussions with Westinghouse and the NRC Staff at a meeting in Bethesda, Maryland on November 19, 1979. The interim solution agreed upon at the meeting is provided in Attachment 1 and will be incorporated in the appropriate Operating Procedures applicable to both North Anna Units 1 and 2 prior to operation. Since the current Technical Specifications limit on control rod insertion is nonconservative, we are immediately implementing an administrative reduction in the control rod insertion limits consistent with Attachment 1. Operation in compliance with this solution will eliminate any safety concerns associated with potential dropped rod events. Westinghouse is continuing to address the issue, and we will evaluate their proposed permanent solutions for applicability to North Anna Units 1 and 2 when the solutions are finalized.

In conjunction with this interim solution, we are withdrawing our proposed Technical Specifications Change No. 21 transmitted by my letter to you dated July 6, 1979, Serial No. 504. As discussed at the meeting on November 19, 1979, these changes are no longer appropriate.

The proposed solution has been reviewed and approved by the Station Nuclear Safety and Operating Committee and the System Nuclear Safety and Operating Committee. If you require additional information, please contact us.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice-President-Power Supply
and Production Operations

Attachment

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cc: Mr. James P. O'Reilly, Director
Office of Inspection and Enforcement
Region II

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ATTACHMENT 1

INTERIM PROCEDURAL CHANGES
REQUIRED FOR NORTH ANNA UNITS 1 AND 2 AS A RESULT OF
CONTROL ROD DROP CONCERNS

- MANUAL ROD CONTROL

  - O TO 100% POWER: NO CHANGE FROM CURRENT ROD INSERTION LIMITS (RIL)

- AUTOMATIC ROD CONTROL

  - O to 90% POWER: NO CHANGE FROM CURRENT RIL

  - 90 TO 100% POWER: D BANK GREATER THAN 215 STEPS

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