UNITED STATES OF AMERICA NUCLEAR REGULATORY COMMISSION

In the Matter of
WISCONSIN ELECTRIC POWER COMPANY
(Point Beach Nuclear Plant,
Unit 1)

Docket No. 50-266

CONFIRMATORY ORDER FOR MODIFICATION OF LICENSE

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Wisconsin Electric Power Company (the licensee) is the holder of Facility Operating License No. DPR-24 which authorizes the licensee to operate the Point Beach Nuclear Plant, Unit 1, located in Two Creeks, Wisconsin, under certain specified conditions. License No. DPR-24 was issued by the Atomic Energy Commission on October 5, 1970 and is due to expire on July 25, 2008.

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Inservice inspections of the Point Beach Unit 1 steam generators performed during the August 1979 and October 1979 outages indicated extensive general intergranular attack and caustic stress corrosion cracking on certain of the external surfaces of the steam generator tubes. As a result of information provided in discussions with the licensee and its representatives, which is documented in a letter dated November 23, 1979 from S. Burstein to H.R. Denton, and the Staff's Safety Evaluation Report, dated November 30, 1979, on Point Beach Unit 1, Steam Generator Tube Degradation Due to Deep Crevice Corrosion, it was determined that additional operating conditions would be required to assure safe operation prior to resumption of operation of Unit 1 from the current refueling outage.

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The licensee in letters dated November 29, 1979 and November 30, 1979 has agreed to additional conditions which are necessary to provide reasonable assurance for safe operation of Unit 1 for a period of 60 effective full power days.

IV

After review of the Licensee's commitment, it has been determined that this commitment should be formalized by order. Accordingly, pursuant to the Atomic Energy Act of 1954, as amended, and the Commission's Rules and Regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED THAT the above license be amended, in the manner hereinafter provided, to include the following conditions:

- 1. a) Within 30 effective full power days, a 2,000 psid primary to secondary hydrostatic test and a 800 psid secondary to primary hydrostatic test will be performed. Should any significant leakage develop as a result of either test, the leaking tubes will be identified and plugged.
 - b) Within 60 effective full power days, the same primary to secondary and secondary to primary hydrostatic tests will be repeated, and an eddy current examination of the steam generator tubes will be performed. This eddy current program will be submitted to the NRC for Staff review.
- 2. Primary coolant activity for Point Beach Nuclear Plant Unit 1 will be limited in accordance with the provisions of sections 3.4.8 and 4.4.8 of the Standard Technical Specifications for Westinghouse Pressurized Water Reactors, Revision 2, July 1979, rather than Technical Specification 15.3.1.C.

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- 3. Close surveillance of primary to secondary leakage will be continued and the reactor will be shut down for tube plugging on detection and confirmation of any of the following conditions:
 - a) Sudden primary to secondary leakage of 150 gpd (0.1 gpm) in either steam generator;
 - Any primary to secondary leakage in excess of 250 gr (0.17 gpm) in either steam generator; or
 - c) An upward trend in primary to secondary leakage in excess of 15 gpd (0.01 gpm) per day, when measured primary to secondary leakage is above 150 gpd.
- 4. The reactor will be shut down, any leaking steam generator tubes plugged, and an eddy current examination performed if any of the following conditions are present:
 - a) Confirmation of primary to secondary leakage in either steam generator in excess of 500 gpd (0.35 gpm); or,
 - b) Any two identified leaking tubes in any 20 calendar day period.
 This eddy current program will be submitted to the NRC for Staff review.
- 5. The NRC Staff will be provided with a summary of the results of the eddy current examination performed under items 1 and 4 above, including a description of the quality assurance program covering tube examination and plugging. This summary will include a photograph of the tubesheet of each steam generator which will verify the location of tubes which have been plugged.
- 6. The licensee will not resume operation after the eddy current examinations required to be performed in accordance with condition 1(b) or 4 until the Director, Office of Nuclear Reactor Regulation determine in writing that the results of such tests are acceptable.

- 7. The licensee will complete a review of Emergency Operating Procedure 3A.
 Revision 9, dated March 29, 1978, confirm that this procedure is appropriate for use in the case of a steam generator tube rupture, and have completed a retraining program for all licensed reactor operators and senior reactor operators in this procedure before return to power.
- Unit 1 will not be operated with more than 18% of tubes plugged in either steam generator.

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Copies of the above referenced documents are available for inspection at the Commission's Public Document Room at 1717 H Street, N.W., Wasnington, D.C. 20555, and are being placed in the Commission's local public document room at Document Department, University of Wisconsin-Stevens Point Library, Stevens Point, Wisconsin 54451.

VI

Any person whose interest may be affected by this Order may within twenty days of the date of this Order request a hearing with respect to this Order.

Any such request shall not stay the effectiveness of this Order. Any request for a hearing shall be addressed to the Director of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555.

In the event a bearing is requested, the issues to be considered at such hearing shall be:

- Whether the facts stated in Section II and III of this Order are correct; and.
- 2) Whether this Order should be sustained.

FOR THE NUCLEAR REGULATORY COMMISSION

Edson G. Case, Acting Director Office of Nuclear Reactor Regulation

Attachment: Staff Safety Evaluation Report, dated November 30, 1979

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Effective date: November 30, 1979. Bethesda, Maryland.

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