DECEMBER 1 2 1979

Docket No. 50-295

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Mr. D. Louis Peoples Director of Nuclear Licensing Commonwealth Edison Company Post Office Box 767 Chicago, Illinois 60690

Dear Mr. Peoples:

You are requested to provide the information specified in the enclosure to this letter regarding auxiliary feedwater system automatic initiations and flow indications to the steam generators for your facilities. A copy of the enclosure was telecopied to you on December 10, 1979. Receipt of your response is requested by December 14, 1979.

Sincerely,

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A. Schwencer, Chief Operating Reactors Branch #1 Division of Operating Reactors

Enclosure: Request for Information

Distribution Docket NRC PDR ORB #1 Reading NRR Reading A. Schwencer C. Parrish E. Reeves M. Grotenhuis Attorney, OELD OI&E (3) NSIC TERA ACRS (16)

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LICENSEE INFORMATION REQUIRED FOR STAFF EVALUATION OF LICENSEE RESPONSE OF LESSONS LEARNED RECOMMENDATIONS 2.1.7a and b

1. 2.1.7a AFW System Automatic Initiation

We require that the licensee provide information that describes <u>how</u> the AFW automatic initiation system design meets each of the control grade functional criteria in NUREG 0578 (2.1.7a) and the clarifications to NUREG-0578 in the October 30 1979 NRC (H. Denton) letter to all operating plants. The information should be sufficiently descriptive to permit independent technical review of the information and a conclusion that each of the criteria are met.

To demonstrate conformance with criteria 1, 2, 5 and 7, specific descriptive information (reference to specific FSAR sections is acceptable) supported by <u>electrical drawings</u>, such as schematic or elementary wiring diagrams, should be submitted.

For power supply criteria 4 and 6, the licensee should submit a table or a detailed power distribution one-line diagram indicating power supply channels for each component.

To demonstrate testability criterion 3, the licensee should submit a summary description of the functional test procedures along with the existing or proposed test interval - (e.g., Standard Technical Specification surveillance requirements for typical engineered safety feature systems).

2. 2.1.7b AFW System Flow Indication to Steam Generators

We require information of a similar nature to that described in 2.1.7a above to demonstrate conformance with the 2.1.7b criteria and clarifications to NUREG-0578 in the October 30, 1979 NRC (H. Denton) letter to all operating plants; namely, single failure, testability, power supply and indication accuracy.

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