



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

November 28, 1979

Docket No. 50-346

Mr. Lowell E. Roe, Vice-President Facilities Development Toledo Edison Company Edison Plaza 300 Madison Avenue Toledo, Ohio 43652

Dear Mr. Roe:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - TECHNICAL SPECIFICATION CHANGES

We have reviewed your submittals dated March 23 and July 13, 1979, concerning your requests for amendments and Technical Specification changes to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station Unit No. 1.

In order to complete our review, we need additional information. You are requested to provide responses to the enclosed request for additional information within 30 days of receipt of this letter. This information has been previously discussed with your Mr. Fred Miller.

Sincerely,

Robert W. Reid, Chief

Operating Reactors Branch #4
Division of Operating Reactors

Enclosure: As stated

cc w/enclosure: See next page

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Toledo Edison Company

cc w/enclosure(s):

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Ohio Department of Health ATTN: Director of Health 450 East Town Street Columbus, Ohio 43216

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Enclosure

Davis Besse Unit 1

Docket Number 50-346

Request for Additional Information Concerning Technical Specification Changes

- Anticipated Reactor Trip Provide a Technical Specification for the proposed number of acceptable scrams in Table 5.7-1 which addresses the anticipated reactor trips.
- 2. RC Seal Injection Alignment Discussions with B. J. Pumps indicate that they feel there is no need for RC seal injection, provided component cooling water flow and temperature are maintained at a satisfactory level. Justify why you feel seal injection is required. Discuss the safety need to maintain these lines in a open condition.
- 3. PORV Operation The proposed Technical Specifications do not address station operation with the electromatic relief isolation valve closed or the electromatic relief valve (PORV) otherwise inoperable. In this regard, include reporting requirements in Section 6.9.2 for a special report on the status of a PORV which has been inoperable or isolated for more than 24 hours when in modes 1, 2, or 3. This report is to be prepared and submitted to the staff within 10 days of PORV inoperability lasting more than 24 hours.
- 4. Auxiliary Feedwater Surveillance Requirement Your surveillance requirements do not include monthly verification for valves in the flow path that are locked, sealed, or otherwise secured in position. Revise your requirements to include a monthly check of these valves to ensure they are locked, sealed, or otherwise secured in their proper position.
- 5. Makeup Pump Termination Provide a list of transients and accidents in the Davis Besse FSAR which currently take credit for the operation of the pressurizer heaters. Assuming the requested Technical Specification change in the makeup pump trip set point is granted by the staff, which transients or accidents would still have sufficient pressurizer water level displacement to produce a pressurizer heater trip? Propose Technical

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Specifications for the pressurizer heaters for those modes where credit is taken for the heaters in accident analyses.

- 6. Anticipated Trip Testing Provide appropriate minimum response times for the functional units of Table 4.3-15 and commit to do response time testing at each refueling.
- Anticipated Trip Testing Provide the bases for the bypass set point (15% for turbine trip) for the anticipated reactor trip of Table 3.3-15.
- 8. Channel Operability Requirements Change the action statements 16 and 17 of Table 3.3-15 to "With the number of channels OPERABLE one less than required by the minimum channels Operable requirements, be in at least HOT STANDBY within 12 hours."

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