U. S. NUCLEAR REGULATORY COMMISSION NRC FORM 366 (7.77) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) (1)CONTROL BLOCK: P F -Ø 3 3 4 Ø Ø N Ø Ø O H D B S (2)0 1 LICENSE NUMBER LICENSEE CODE CON'T 9 8 1 2 1 74 75 REPOR 11 REPORT (7)1 1 2 7 3 4 6 L 10 (6) 0 Ø 0 1 REPORT DATE SOURCE 60 DOCKET NUMBE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) On 11/12/79 while testing High Pressure Injection (HPI) Pump 1-1, it failed to start. 0 2 This lead to the identification of a factory defect in the control circuit wiring. 0 3 The unit was in Mode 5, and HPI Pump 1-2 was operable during the period that HPI 1-1 0 4 There was no danger to the health and safety of the public or to was inoperable. 0 5 HPI 1-2 was operable during the period that HPI 1-1 was inoperastation personnel. 0 6 ble and capable of supplying water to the core in the event of a safety features 0 7 (NP-33-79-127) actuation. 8 0 80 8 COMP. VALVE CAUSE CAUSE SYSTEM SUBCODE SUBCODE COMPONENT CODE CODE CODE K 1(14) X 1(15 TIBIRI (16) B (13) KI C B SIF (12 (11 0 9 10 18 REVISION REFORT OCCURRENCE SEQUENTIAL TYPE CODE REPORT NO EVENT YEAF Ø LER/RO L 013 11 10 19 1 (17) REPORT NUMBER COMPOSTUP ANUFACTUP COMPONENT PRIME COMP. NPRD-4 SUBMITTED METHOD (22) RFF ACTION 1ER ACTION HOURS FORM SUB. SUPPI Y A (25 ØI Ø IY Z (21 Z (20 (23 Z (19) B (18) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27 The cause was due to an improperly terminated lug in the breaker control wiring which 10 caused a bad connection. The condition was aggravated by modification work being 111 done for the arc suppression for the breaker control circuits. Under MWO 79-3522, the 1 2 lead was properly terminated and reinstalled. HPI Pump 1-1 was returned to operabi-1 3 lity at 1327 hours on 11/13/79. 1 4 80 METHOD OF (32) (30) DISCOVERY DESCRIPTION FACILITY OTHER STATUS & POWER Test after modification C (31) 000 NA 1 5 D (28) 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) RELEASED OF RELEASE NA (34) Z (33) Z NA 6 80 PERSONNEL EXPOSURES DESCRIPTION (39) TYPE NUMBER Ø Ø Ø (37) Z (38 NA 7 80 PERSONNEL INJURIES poor original DESCRIPTION (41) NUMBER 0 0 0 40 NA 1 8 229 80 1592 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION VPP Z (42 NA 7912170309 NRC USE ONLY PUBLICITY DESCRIPTION (45 SSUED (44 N NA 2 0 68 69 80 419-259-5397 Richard W. Naylor PHONE :. DVR 79-1.6 NAME OF PREPARER.

TOLEDO EDISON COMPANY DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE SUPPLEMENTAL INFORMATION FOR LER NP-33-79-127

DATE OF EVENT: November 12, 1979

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: High Pressure Injection Pump 1-1 failed to start during special testing

<u>Conditions Prior to Occurrence</u>: The unit was in Mode 5, with Power(MWT) = 0, and Load (Gross MWE) = 0.

Description of Occurrence: On November 12, 1979 while testing High Pressure Injection (HPI) Pump 1-1 after a modification for surge suppression in the cubicle, HPI Pump 1-1 failed to start. Maintenance personnel stopped the test and commenced troubleshooting. This lead to the identification of a factory defect in the control circuit wiring. A lead had been improperly stripped prior to the lug being crimped, thus the lead was insulated rather than the wire being crimped in the termination lug.

The unit was in Mode 5, and HPI Pump 1-2 was operable during the period that HPI 1-1 was inoperable.

Designation of Apparent Cause of Occurrence: The cause of the occurrence was due to an improperly terminated lug in the breaker control wiring which caused a bad connection. The lug was crimped to the lead insulation rather than the wire. This was due to improper insulation removal when the cabinet was manufactured. The condition was aggravated by the modification work which caused wires in that area to be moved which caused the discontinuity in that circuit. This modification work was being done for the arc suppression for the breaker control circuits.

Analysis of Occurrence: There was no danger to the health and safety of the public or to station personnel. HPI 1-2 was operable during the period that HPI 1-1 was inoperable and capable of supplying water to the core in the event of a safety features actuation.

Corrective Action: Under Maintenance Work Order 79-3522, the lead was properly terminated and reinstalled. The breaker was cycled several times and placed back to the original position and the pump started. Following the work, all testing for the arc suppression was completed, and the unit returned to operability at 1327 hours on November 13, 1979. Wiring within the cabinet was inspected, and no other improperly crimped wires were found.

Failure Data: No previously similar occurrences have been identified or reported.

1592 230

LER #79-110