(7.77)	LICENSEE EVENT REPORT
	CONTROL BLOCK:
	N   Y   J   A   F   1   20   0   -   0   0   -   0   0   0   1
	REPORT L 6 0 5 0 0 3 3 3 7 1 0 0 8 7 9 8 1 1 2 8 7 9 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80
0 2	SEE ATTACHMENT
03	
04	
0 5	
0 6	
07	
08 78	9 SYSTEM CAUSE CAUSE COMP. VALVE 30
09 78	CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCO
	17 LER/RO REPORT EVENT YEAR REPORT NO. CODE TYPE NO.   17 NUMBER 17 19 1 0 8 3 1 10 3 1 12   10 82 21 22 23 24 26 27 28 29 30 31 32
	ACTION FUTURE EFFECT SHUTDOWN HOURS 22 ATTACHMENT NPRD-4 PRIME COMP. COMPONENT TAKEN ACTION ON PLANT METHOD HOURS 22 ATTACHMENT FORM SUB. SUPPLIER MANUFACTURER 33 18 2 19 19 19 9 9 9 26 34 35 36 27 40 41 23 12 25 2 25 2 9 9 9 9 9 26 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27
10	SEE ATTACHMENT
1 2	
13	
	30
1 5	ACILITY NOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32 E 28 1 0 0 29 NA A A 31 OPERATOR OBSERVATION
16	CTIVITY CONTENT. 12 13 44 45 46 80 ELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 38 Z 33 Z 34 NA NA
7 8	9 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 0 0 0 (37) Z (38) NA
7 8	PERSONNEL INJURIES NUMBER DESCRIPTION (41) 10101010101 NA
7 8	30 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION Z [42] NA 7912130 454
7 8	9 10 PUBLICITY ISSUED DESCRIPTION (45) NAC USE ONLY ISSUED DESCRIPTION (45)
7 8	9 10 68 69 80 5 W. Verne Childs (315) 342-3840 9
	NAME OF PREPARER

1.1.84

POWER AUTHORITY OF THE STATE OF NEW YORK JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-083/03X-2

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Page 1 of 1

During normal operation, operations personnel noted that a routine computer printout of the core performance log indicated that the Maximum Fraction of Limiting Power Density (MFLPD) was 1.014 and 1.016 for two nodes compared to a Technical Specification limit of equal to or less than 1.000. Reactor power was immediately reduced approximately 3% in accordance with the requirements of Technical Specification, Appendix A, Paragraph 3.5.I. This action restored MFLPD to within the limits of Technical Specifications within the time frame allowed and is considered to be equivalent to operation in a degraded mode.

Following the initial power reduction, the control rod pattern was adjusted to reduce local power and power was returned to approximately 100% of rated power three hours later.

The event did not represent any significant hazard to the public health and safety.

NOTE: Revision 2 of this LER is submitted to correct the technical specification paragraph reference above.

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