

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | V | T | V | Y | S | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 14 15 25 26 30 37 58
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

CON'T
0 1 | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 7 | 1 | 7 | 1 | 1 | 1 | 0 | 7 | 9 | 8 | 1 | 2 | 1 | 0 | 7 | 9 | 9
7 8 60 61 68 69 74 75 80
REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | _____
0 3 | See attached sheet
0 4 | _____
0 5 | _____
0 6 | _____
0 7 | _____
0 8 | _____

0 9 | I | B | 11 | A | 12 | F | 13 | I | N | S | T | R | U | 14 | T | 15 | Z | 16 |
7 8 9 10 11 12 13 18 19 20
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
17 | LER RO REPORT NUMBER | 7 | 9 | 21 | 22 | - | 23 | 0 | 3 | 0 | 24 | 26 | / | 27 | 0 | 3 | 28 | 29 | L | 30 | - | 31 | 0 | 32 |
7 8 21 22 23 24 26 27 28 29 30 31 32
EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
B | 18 | H | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | 37 | Y | 23 | Y | 24 | N | 25 | R | 2 | 9 | 0 | 26 |
33 34 35 36 37 40 41 42 43 44 47
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | _____
1 1 | The packing leak was repaired, the reference leg refilled, and the instruments
1 2 | returned to operation. A similar occurrence was reported as RO 79-24.
1 3 | _____
1 4 | _____

1 5 | Z | 28 | 1 | 0 | 0 | 29 | NA | 30 | A | 31 | Operator observation | 32 |
7 8 9 10 12 13 44 45 46 80
FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

1 6 | Z | 33 | Z | 34 | NA | 35 | NA | 36 |
7 8 9 10 11 44 45 80
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE

1 7 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39 |
7 8 9 11 12 13 80
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION

1 8 | 0 | 0 | 0 | 40 | NA | 41 | 1552 238 |
7 8 9 11 12 13 80
PERSONNEL INJURIES NUMBER DESCRIPTION

1 9 | Z | 42 | NA | 43 | 7912180 385 |
7 8 9 10 80
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION

2 0 | Z | 44 | NA | 45 | _____ | 80
7 8 9 10 80
PUBLCITY ISSUED DESCRIPTION NRC USE ONLY

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

During steady state operation, Control Room personnel observed an increasing trend in indicated torus water level on Control Room Indicators LI 16-19-46A/B. When an indicated level of 1.26' was reached (corresponding to 70,000 ft³) an orderly shutdown was initiated as necessitated by Technical Specification 3.7.A.8.

Upon investigation, it was determined that a bypass valve for the torus level transmitters had developed a packing leak thereby slowly draining the transmitters common reference leg and causing the erroneous readings. Technical Specification 3.2.G requires this post-accident instrumentation be operable during reactor operation. At no time was the torus water inventory out of specification. Since this indication was restored within the limiting 6 hours of Tech. Spec. Table 3.2.6, the plant shutdown was terminated and power operation was continued. Based on the above, the potential consequences to the health and safety of the public were minimal.

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