

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT



JOHN D. LEONARD, JR.
Resident Manager

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September 18, 1979
JAFP 79-490

Boyce H. Grier, Director
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406

SUBJECT: DOCKET NO. 50-333
NRC I&E INSPECTION NO. 79-07

Dear Mr. Grier:

With reference to the inspection conducted by Mr. R. Zimmerman of your office on May 29-June 1, 1979, at the James A. FitzPatrick Nuclear Power Plant, and in accordance with the provisions of Section 2.201 of Part II of Title 10 of the Code of Federal Regulations, we are submitting our response to Appendix A, Notice of Violation transmitted by your letter dated August 24, 1979 as received by the undersigned on August 29, 1979.

APPENDIX A

NOTICE OF VIOLATION

Based on the results of an NRC inspection conducted May 29 through June 1, 1979, it appears that one of your activities was not conducted in full compliance with conditions of your NRC Facility License No. DPR-59 as indicated below. This item has been categorized as a Deficiency.

Technical Specification 6.8.A states, in part, "Written Procedures... shall be established, implemented, and maintained that meet or exceed the requirements and recommendations of Section 5, Facility Administrative Policies and Procedures of ANSI 18.7-1972..." ANSI 18.7-1972, Section 5.3.6, Instrument Calibration and Test Procedures, states, in part, "The procedures shall have provisions for assuring measurement accuracies adequate to keep safety parameters within operational and safety limits."

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Contrary to the above, surveillance calibration F-ISP-4-5, Drywell Suppression Chamber Differential Pressure, written in part to satisfy Technical Specification 3.2.5 and performed May 2, 1979, did not contain acceptance criteria for assuring measurement accuracy for drywell pressure indicator PI-102B. Further, the output data recorded for PI-102B did not correspond to the input signals from pressure transmitter PT-102B as would be expected. This was found to be due to an incompatibility in operating ranges between PT-102B (10-20 PSIA) and PI-102B (0-20 PSIA). The above calibration surveillance test had received secondary level review and approval with no discrepancies noted.

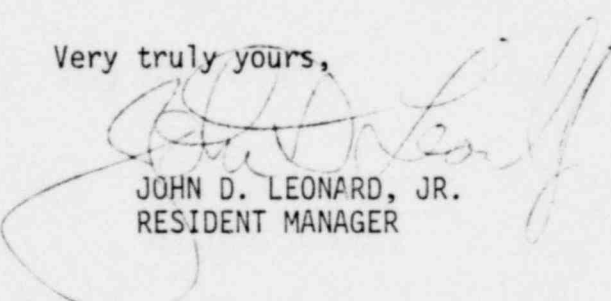
RESPONSE TO NOTICE OF VIOLATION

The James A. FitzPatrick Nuclear Power Plant staff acknowledges the deficiency noted above and has reviewed the deficiency and this Response this date at PORC Meeting 79-056. The acknowledgement of this deficiency is further indicated by the submittal of Licensee Event Report No. 79-034 on June 21, 1979.

As noted in the reference Licensee Event Report, Instrument Surveillance Procedure F-ISP-4.5 was revised to correct the discrepancy in the indicator scale for Instrument 27-PI-102B. In addition, the temporary scale which was placed on the indicator during Inspection 79-07 has been replaced by a permanent scale.

During the PORC meeting referenced above, the Committee also discussed the responsibility of the plant's management personnel with respect to the requirements of Technical Specification, Appendix A, Paragraph 6.8.A. The importance of the careful review of test data and procedural criteria was emphasized.

Very truly yours,


JOHN D. LEONARD, JR.
RESIDENT MANAGER

JDL:VC:brp
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