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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

RELATED CORRESPONDENCE

October 17, 1979

Docket No. 50-312

Mr. J. J. Mattimoe
Assistant General Manager and
Chief Engineer
Sacramento Municipal Utility District
6201 S Street
P. O. Box 15330
Sacramento, CA 95813



Dear Mr. Mattimoe:

RE: RANCHO SECO NUCLEAR GENERATING STATION

By letter dated July 18, 1979, you submitted a proposed inservice inspection and testing program description and a request for relief from selected ASME Code requirements pursuant to 10 CFR 50.55a(g). Although we have not completed our detailed review of your submittal, our preliminary review makes clear to us that your proposed program to implement those ASME Code requirements that you have found to be practical would increase the scope of inservice inspection and testing for your facility beyond that currently required by your Technical Specifications. We have concluded that this upgrading of your inservice inspection and testing program will further enhance safety.

Based on our preliminary review, we agree with your determination that it is impractical within the limitations of design, geometry and materials of construction of components, for you to meet certain of the specified ASME Code requirements and that imposition of those requirements would result in hardships or unusual difficulties without a compensating increase in the level of quality or safety. Therefore, pursuant to 10 CFR 50.55a(g)(6)(i), we hereby grant relief, on an interim basis, pending completion of our detailed review, from those inservice inspection and testing requirements of the ASME Code that you have requested as modified by NRC staff comments on your proposal made during the working session held with your staff on October 3 and 4, 1979. Moreover, since the scope of the inservice inspection and testing will be increased by your proposed program, and the granting of this relief is based only on the impracticality of selected ASME Code requirements, we have determined that the relief granted neither increases the probability or consequences of accidents previously considered nor decreases safety margins and that, therefore, it does not involve a significant hazards consideration. Therefore, you are authorized to, and should proceed to, implement your proposed program as modified by NRC staff comments made during the working session (except where your current Technical Specifications are more restrictive).

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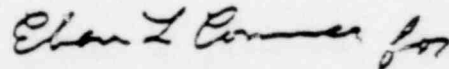
Within 30 days of receipt of this letter, you should submit to the NRC documentation of the modifications you will be implementing with your proposed program based on the comments received from the staff during the working session on October 3 and 4, 1979.

During the period between the date that the requirements of 10 CFR 50.55a(g) become effective for your facility and the date we complete our detailed review of your submittal you must comply with both your existing Technical Specifications and your proposed inservice inspection and testing program. In the event conflicting requirements arise for some components, you must comply with the more restrictive requirements (e.g., shorter inspection intervals, increased number of parameters measured). In other words, the granting of this relief from ASME Code requirements should not be interpreted to give you relief from any of the requirements in your existing Technical Specifications.

When our detailed review of your July 18, 1979 submittal is complete, we will: (1) issue final approval of your program (which may contain modifications resulting from the staff's final review), (2) grant relief from any ASME Code requirements that are determined to be impractical for your facility for the duration of the inspection interval and (3) issue appropriate changes to your Technical Specifications.

A copy of the Federal Register Notice related to this action is enclosed.

Sincerely,



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosure:
Federal Register Notice

cc: w/enclosure
See next page

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Sacramento Municipal Utility
District

w/enclosure(s):

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Sacramento, California 95814

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RELATED CORRESPONDENCE

Sacramento Municipal Utility
District

cc w/enclosure(s):

Atomic Safety and Licensing Board
Panel
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Atomic Safety and Licensing Appeal
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California Department of Health
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Radiation Control Unit
Radiological Health Section
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Sacramento, California 95814

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