



**SMUD**

SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, Box 15830, Sacramento, California 95813; (916) 452-3211

November 16, 1979

Mr. R. H. Engelken, Director  
Region V Office of Inspection & Enforcement  
U. S. Nuclear Regulatory Commission  
1990 North California Boulevard  
Walnut Creek Plaza, Suite 202  
Walnut Creek, California 94396

Re: Operating License DPR-54  
Docket No. 50-312  
Reportable Occurrence 79-15

Dear Mr. Engelken:

In accordance with Technical Specifications for Rancho Seco Nuclear Generating Station 6.9.4.2b, and Regulatory Guide 1.16, Revision 4, Section C.2.b.2, the Sacramento Municipal Utility District is hereby submitting a thirty-day report of Reportable Occurrence 79-15.

While performing the required inspection/analysis review of safety-related piping systems required by I&E Bulletin 79-14, the District became aware of a potential nonconformance between the design documents and the original seismic analyses on the "A" Nuclear Service Raw Water System.

To accomplish the requirements of I&E Bulletin 79-14 in the appropriate time frame, the Architect Engineer, Bechtel Norwalk, was contracted to assist in the review and analysis. On October 31, 1979, Bechtel informed the District that their analysis indicated the necessity for an additional support and modification of two existing supports on line No. 47360-10"-HE2. This line is in the "A" Nuclear Service Raw Water System and is the return line from the Diesel Generator Cooling Water Heat Exchanger E-473A. The original seismic analysis assumed a support in this area capable of withstanding a seismic load greater than the design load of the actual support. The addition of another support and modification of two existing supports would then meet the design criteria.

Upon receipt of this information, the "A" NSRW system was declared inoperable. Work was started on the fabrication and installation of the additional support and modifications to two existing supports. This was completed within the 48 hours allowable per Technical Specifications Section 3.3.1 for continued reactor operation.

The additional support installed was assigned the I.D. number 8G-4730-12. The two existing supports which were modified were I.D. No. 8G-47360-1 and I.D. No. 8G-47360-10.

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This occurrence, as well as a similar occurrence on the "A" NSCW system reported to your office via LER 79-14, is also addressed in the District's submittal of phase 3 of the District's response to I&E Bulletin 79-14 dated October 31, 1979.

As stated in the phase 3 submittal, the inspection/verification of all accessible category 1 systems has been completed. The remaining verifications, phase 4, of the inaccessible piping will be accomplished at the next refueling outage in January, 1980.

There were no transients or power reductions associated with this event.

Respectfully submitted,



J. J. Mattimoe  
Assistant General Manager  
and Chief Engineer

JJM:RWC:HH:jr

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