5.1 RESPONSIBILITY

5.1.1 The Plant Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

5.2 ORGANIZATION

OFFSITE

5.2.1 The offsite organization for facility management and technical support shall be as shown in Figure 5.2-1.

FACILITY STAFF

5.2.2 The Facility oganization shall be shown on Figure 5.2-2 and:

a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 5.2-1.

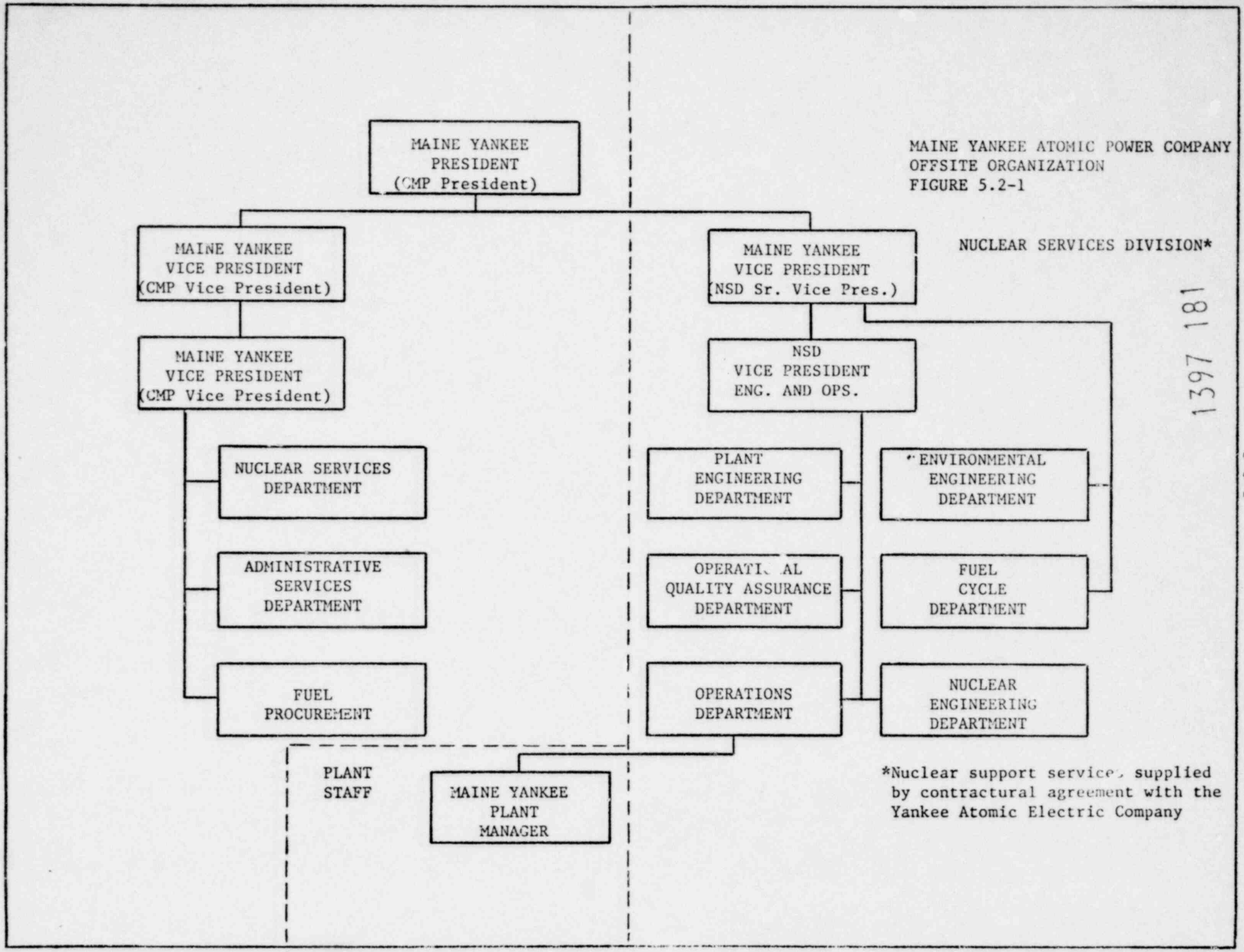
b. At least one licensed Operator shall be in the control room when fuel is in the reactor.

c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.

d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.

e. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

f. A fire brigade of at least 5 members shall be maintained onsite at all times. Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements. This excludes 2 members of the minimum shift crew necessary for the safe shutdown of the plant and any personnel required for other essential functions during a fire emergency.



MAINE YANKEE ATOMIC POWER COMPANY
OFFSITE ORGANIZATION
FIGURE 5.2-1

NUCLEAR SERVICES DIVISION*

*Nuclear support services supplied by contractural agreement with the Yankee Atomic Electric Company

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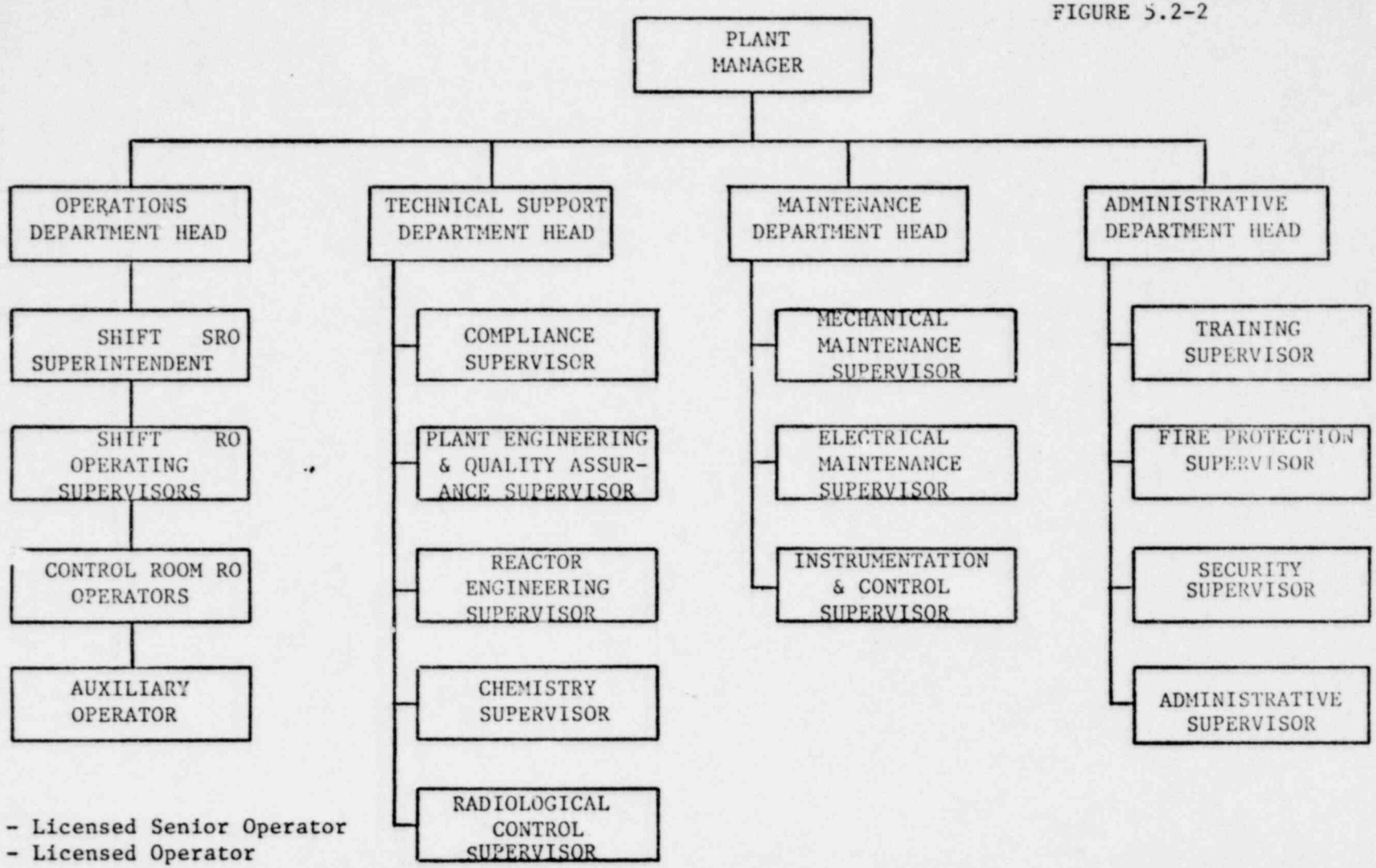
MAINE YANKEE ATOMIC POWER COMPANY
FACILITY ORGANIZATION
FIGURE 5.2-2



SRO - Licensed Senior Operator
RO - Licensed Operator

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TABLE 5.2-1

MINIMUM SHIFT CREW COMPOSITION 4/

LICENSE CATEGORY	APPLICABLE CONDITIONS	
	1/	2/
SOL	1	1 3/
OL	2	1
Non-Licensed	2	1

1/ All conditions except cold shutdown and refeuling shutdown condition.

2/ Cold shutdown and refueling shutdown condition.

3/ Does not include the licensed Senior Reactor Operator or Senior Reactor Operated Limited to Fuel Handling, supervising CORE ALTERATIONS.

4/ Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accomodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 5.2-1.

5.4 TRAINING

5.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Plant Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

5.4.2 A training program for the Fire Brigade shall be maintained under the direction of a plant staff member appointed to perform the duties of the Fire Protection Coordinator and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976.

5.5 REVIEW AND AUDIT

A. PLANT OPERATION REVIEW COMMITTEE

1. FUNCTION

The Plant Operation Review Committee (PORC) shall function to advise the Plant Manager on all matters related to nuclear safety.

2. COMPOSITION

The Plant Operation Review Committee shall be composed of the:

Chairman: Plant Manager

Vice Chairman: Assistant Plant Manager - Designated

Member: Operations Dept. Head

Member: Maintenance Dept. Head

Member: Technical Support Dept. Head

Member: Reactor Engineering Supervisor

Member: Chemistry Supervisor

Member: Instrument and Control Supervisor

Member: Radiological Controls Supervisor

3. ALTERNATES

All alternate members shall be appointed in writing the by PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PORC activities or count toward a PORC quorum at any one time.

4. MEETING FREQUENCY

The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or Vice Chairman.

5. QUORUM

A quorum of the PORC shall consist of a minimum of five people as follows:

a. The Chairman or Vice Chairman plus four members, or

b. The Chairman and Vice Chairman plus three members.

6. RESPONSIBILITIES

The Plant Operation Review Committee shall be responsible for:

5.3 FACILITY STAFF QUALIFICATIONS

5.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiological Control Supervisor who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

a. Review of 1) all procedures required by Specification 5.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes to Appendix "A" Technical Specifications.

d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

e. Investigation of all violations of the Technical Specifications including the preparation of forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager of Operations and to the Chairman of the Nuclear Safety Audit and Review Committee.

f. Review of events requiring 24 hour written notification to the Commission.

g. Review of facility operations to detect potential nuclear safety hazards.

h. Performance of special reviews, investigations of analyses and reports thereon as requested by the Chairman of the Nuclear Safety Audit and Review Committee.

i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Security Supervisor and Chairman of the Nuclear Safety Audit and Review Committee.

j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Plant Manager.

7. AUTHORITY

The Plant Operation Review Committee shall:

a. Recommend to the Plant Manager in writing, approval or disapproval of items considered under 6 (a) through (d) above.

b. Render determinations in writing with regard to whether or not each item considered under 6(a) through (e) above constitutes an unreviewed safety question

c. Provide written notification within 24 hours to the Manager of Operations of disagreement between the PORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 5.1.1 above.

8. RECORDS

The Plant Operation Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Manager of Operations and Chairman of the Nuclear Safety Audit and Review Committee.

B. NUCLEAR SAFETY AUDIT AND REVIEW COMMITTEE

1. FUNCTION

The Nuclear Safety Audit and Review (NSAR) Committee shall function to provide independent review and audit of all aspects of plant safety. Adequacy of this review and audit is assured by the cross section of disciplines required of the Committee membership as described in Section B-3.

2. COMPOSITION

The NSAR Committee shall be composed of at least six persons with the Committee membership and its Chairman and Vice Chairman appointed by the Nuclear Services Division Senior Vice President.

a. Chairman

b. Vice Chairman

c. Four technically qualified persons who are not members of the plant staff.

3. QUALIFICATION

Membership to the NSAR Committee requires that an individual meet one of the below academic and/or experience requirements:

a. Bachelor Degree or equivalent, plus five (5) years total experience in the below listed disciplines.

b. Nine (9) years total experience in the below listed disciplines.

(1) Nuclear Power Plant Operations

(2) Nuclear Engineering

(3) Chemistry and Radiochemistry

(4) Mechanical and Electrical Engineering

(5) Radiological Safety

(6) Instrumentation and Control

(7) Metallurgy

(8) Quality Assurance practices

4. ALTERNATES

All NSAR Committee alternate members shall be appointed in writing by the Nuclear Services Division Senior Vice President to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NSAR Committee activities at any one time.

5. CONSULTANTS

Consultants may be utilized as determined by the NSAR Committee Chairman to provide expert advice, when needed, to the NSAR Committee.

6. MEETING FREQUENCY

The NSAR Committee shall meet at least once per six months, plus 25%. Special meetings may be held when deemed necessary by Company management or by the Chairman of the NSAR Committee, or, in the absence of the Chairman, by the Vice Chairman.

7. A quorum of the NSAR shall consist of the Chairman or his designated alternate and at least 4 NSAR members, including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

8. REVIEW

The NSAR Committee shall review:

a. The Safety Evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provisions of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.

b. Proposed Changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.

c. Proposed test or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.

d. Proposed Changes to Technical Specifications or this Operating License.

e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.

f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety, defined as Plant Information Reports.

g. Events requiring 24 hour written notification to the Commission.

h. Reports and meeting minutes of the Plant Operation Review Committee.

i. Perform special reviews and investigations and render reports thereon as requested by the Nuclear Services Division Vice President or to his delegated alternate.

9. AUDITS

Audits of facility activities shall be performed under the cognizance of the NSAR Committee. These audits shall encompass:

a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualification of those members of the facility staff who have a direct relationship to operation, maintenance or technical aspects of the plant, at least once per 12 months.

c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.

d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.

e. The Facility Emergency Plan and implementing procedures at least once per 24 months.

f. The Facility Security Plan and implementing procedures at least once per 24 months.

g. The Facility Fire Protection Program and implementing procedures at least once per 24 months.

h. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.

i. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

j. Any other area of facility operation considered appropriate by the NSAR Committee or the Vice President.

10. AUTHORITY

The NSAR Committee shall report to and advise the NSD Senior Vice President on those areas of responsibility specified in Section 5.B.8 and 5.B.9.

11. RECORDS

Records of NSAR activities shall be prepared, approved and distributed as indicated below:

a. Minutes of each NSAR meeting shall be prepared, and forwarded to the NSD Senior Vice President within 20 working days following such meeting.

b. Reports of reviews encompassed by Section 5.B.8 above, shall be prepared, and forwarded to the NSD Senior Vice President within 20 working days following completion of the review.

c. Audit reports encompassed by Section 5.B.9 above, shall be forwarded to the management positions responsible for the areas audited within 30 working days after completion of the audit.

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5.6 REPORTABLE OCCURRENCE ACTION

5.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 5.9.

b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PORC and submitted to the NSAR Committee and the Manager of Operations.

5.7 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

a. The facility shall be placed in at least HOT STANDBY within one hour.

b. The Safety Limit violation shall be reported to the Commission, the Manager of Operations and to the NSAR Committee within 24 hours.

c. A safety Limit Violation Report shall be prepared. The report shall be reviewed by the Plant Operations Review Committee. This report shall describe (1) applicable circumstances preceding the violations, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the NSAR Committee and the Manager of Operations within 14 days of the violation.

5.8 PROCEDURES

5.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.

b. Refueling operations.

c. Surveillance and test activities of safety related equipment.

d. Security Plan implementation.

e. Emergency Plan implementation.

f. Fire Protection Program implementation.

5.8.2 Each procedure of 5.8.1 above, and changes thereto, shall be reviewed by the PORC and approved by the (Plant Manager) prior to implementation and reviewed periodically as set forth in administrative procedures.

5.8.3 Temporary changes to procedures of 5.8.1 above may be made provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License.

c. The change is documented, reviewed by the PORC and approved by the Plant Manager within 14 days of implementation.

5.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

5.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

STARTUP REPORT

5.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

5.9.1.2 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

ANNUAL OPERATING REPORT

5.9.1.3 A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,* e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

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*This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

MONTHLY OPERATING REPORT

5.9.1.4 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Management Information and Program Control, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office, to arrive no later than the fifteenth of each month following the calendar month covered by the report.

REPORTABLE OCCURRENCES

5.9.1.5 The REPORTABLE OCCURRENCES of Specifications 5.9.1.6 and 5.9.1.7 below, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully described final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

5.9.1.6 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

a. Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function

b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the Technical Specifications.

c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

d. Reactivity anomalies involving disagreement with the predicted value of reactivity balance under steady-state conditions during power operation greater than or equal to 1% delta k/k; a calculated reactivity balance indicating a SHUTDOWN MARGIN less conservative than specified in the Technical Specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if subcritical, an unplanned reactivity insertion of more than 0.5% delta k/k; or occurrence of any unplanned criticality.

e. Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.

f. Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

g. Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by Technical Specifications.

h. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the Technical Specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

i. Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or Technical Specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or Technical Specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

THIRTY DAY WRITTEN REPORTS

5.9.1.7 The types of events listed below shall be the subject of written reports to the Director of the Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

a. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the Technical Specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

b. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

c. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

d. Abnormal degradation of systems other than those specified in 5.9.1.6.c above designed to contain radioactive material resulting from the fission process.

UNIQUE REPORTING REQUIREMENTS

Environmental Radiological Monitoring

(1) A report on the Environmental Radiological Monitoring Program for the previous twelve months of operation shall be submitted as a separate document within ninety days after January 1 of each year.

(a) For each medium sampled during the reporting period, e.g., air, baybottom, surface water, soil, fish; include:

(1) Number of Sampling locations,

(2) Total number of samples,

(3) Number of locations at which levels are found to be significantly above local backgrounds, and

(4) Highest, lowest, and average concentrations or levels of radiation for the sampling point with the highest average and description of the location of that point with respect to the site.

(b) If levels of radioactive materials in environmental media as determined by an environmental monitoring program indicate the likelihood of public intakes in excess of 1% of those that could result from continuous exposure to the concentration values listed in Appendix B, Table II, Part 20, estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.

(c) If statistically significant variations of offsite environmental concentrations with time are observed, correlation of these results with effluent release shall be provided.

(2) Semiannual Effluent Release Report

Within 60 days after January 1 and July 1 of each year, a report shall be submitted covering the radioactive content of effluents released to unrestricted areas during the previous six months of operation. The data shall be summarized on a monthly basis and included as a minimum:

(A) Gaseous Effluents

(1) Gross Radioactivity Releases

(a) Total gross radioactive (in curies), primarily noble and activation gases.

(b) Maximum gross radioactivity release rate during any one-hour period.

(c) Total gross radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.

(d) Percent of Technical Specification limit.

(2) Iodine Release

(a) Total iodine radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.

(b) Percent of Technical Specification Limit for I-131 released.

(3) Particulate Releases

(a) Total gross radioactivity (B,γ) released (in curies) excluding background radioactivity.

(b) Gross alpha radioactivity released (in curies) excluding background radioactivity.

(c) Total gross radioactivity (in curies) of nuclides with half-lives greater than 8 days.

(d) Percent of Technical Specification limits for particulate radioactivity with half-lives greater than 8 days.

(B) Liquid Effluents

(a) Total gross radioactivity (B,γ) released (in curies) excluding the tritium and average concentration released to the unrestricted area.

(b) The maximum concentration of gross radioactivity (B,γ) released to the unrestricted area (averaged over the period of release).

(c) Total tritium and total alpha radioactivity (in curies) released and average concentration released to the unrestricted area.

(d) Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area.

(e) Total volume (in liters) of liquid waste released.

(f) Total volume (in liters) of dilution water used prior to

release from the restricted area.

(g) Total gross radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.

(h) Percent of Technical Specification limit for total radioactivity.

(C) Solid Wastes

(a) The total amount of solid waste shipped (in cubic feet).

(b) The total estimated radioactivity (in curies) involved.

(c) Disposition including date and destination.

Special Reports

Special reports shall be submitted to the Director of the NRC Regional Office within the time period specified for each report. These reports shall be submitted concerning the activities identified below pursuant to the requirements of the applicable specification.

a. Reactivity anomolies, Specification 3.15.

b. Excessive radioactive release, Specifications 3.16A2 and 3.17A2.

c. Plans for restoration of 115 KV service, Specification 3.12.

d. Containment Type A test failure, Specification 4.4IC2.

e. Integrated leakage rate test report, Specification 4.4III.

f. Reactor Coolant System Activity Specification 3.2.

5.10 RECORD RETENTION

5.10.1 The following records shall be retained for at least five years:

a. Records and logs of facility operation covering time interval at each power level.

b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.

c. All REPORTABLE OCCURRENCES submitted to the Commission.

d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.

e. Records of changes made to Operating Procedures.

f. Records of radioactive shipments.

g. Records of sealed source and fission detector leak tests and results.

h. Records of annual physical inventory of all sealed source material of record.

5.10.2 The following records shall be retained for the duration of the Facility Operating License:

a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.

b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.

c. Records of facility radiation and contamination surveys.

d. Records of radiation exposure for all individuals entering radiation control areas.

e. Records of gaseous and liquid radioactive material released to the environs.

f. Records of transient of operational cycles for the reactor pressure vessel and the reactor coolant system.

g. Records of reactor tests and experiments.

h. Records of training and qualification for current members of the plant staff.

i. Records of in-service inspections performed pursuant to these Technical Specifications.

j. Records of Quality Assurance activities required by the QA Manual.

k. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

l. Records of meetings of the PORC and the NSAR Committees.

5.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

5.12 HIGH RADIATION AREA

5.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is 1000 mrem/hr or less shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit.* Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.

c. A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and who will perform periodic radiation surveillance at the frequency specified in the RWP. The surveillance frequency will be established by the Radiological Control Supervisor.

5.12.2 The requirements of 5.12.1 above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Plant Shift Superintendent on duty and/or Radiological Control Supervisor.

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*Health Physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.