

U. S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
REGION IV

IE Inspection Report No. 50-267/79-14

Docket No. 50-267

License No. DPR-34

Licensee: Public Service Company of Colorado
P. O. Box 840
Denver, Colorado 80201

Facility: Fort St. Vrain Nuclear Generating Station

Inspection at: Fort St. Vrain Site, Platteville, Colorado

Inspection Conducted: August 27 through September 28, 1979

Inspector: T. F. Westerman for. 10-12-79
M. W. Dickerson, Reactor Inspector Date

Reviewed by: T. F. Westerman 10-12-79
T. F. Westerman, Chief, Reactor Projects Date
Section

Inspection Summary

Inspection conducted during the period August 27 through September 28, 1979
(Report No. 50-267/79-14)

Areas Inspected: Routine, announced inspection of review of plant operations, maintenance, physical protection, follow-up on bulletins/circulars, review of special and periodic reports, follow-up on previous items of noncompliance, and follow-up on inspector identified and unresolved problems. The inspection involved 154 inspector-hours on-site by one (1) NRC inspector.

Results: No items of noncompliance or deviations were identified.

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Details Section1. Persons ContactedPublic Service Company of Colorado

D. Alps, Lead Security Officer
L. Brey, QA Manager
W. Crane, Maintenance Superintendent
R. Craun, Engineer - Site
J. Gamm, Technical Services Supervisor
R. Gunderson, Senior Engineer, Project Engineering
W. Hillyard, Administrative Services Manager
T. Howard, Superintendent Operations, QA
J. Liebelt, Maintenance Supervisor, Electrical
F. Mathie, Operations Manager
M. McBride, Site Engineering Coordinator
J. Oliver, Shift Supervisor Operations
J. Solakiewicz, QA Engineer
D. Warembourg, Manager, Nuclear Production
V. Wetzberger, Scheduling/QC Supervisor

Stearns-Roger

H. Bates, Field Engineer
R. Lawrence, Field Engineer

The inspector also contacted other plant personnel, including reactor operators, maintenance men, electricians, technicians, and administrative personnel.

2. Licensees Action on Previous Inspection Findings

(Closed) Item of Noncompliance (50-267/79-12): Reserve Shutdown System Apparent Failure to Complete Valve Checkoff List. The licensee is in the process of rewriting the system checkoff lists and has issued instructions to assign valve lineups on an individual basis to one individual.

(Closed) Items of Noncompliance (50-267/79-13): Failure to comply with procedural requirements. The inspector examined the licensee's corrective action contained in his August 15, 1979 reply relative to comply with certain procedural requirements. Individuals have been reprimanded and Administrative Procedure ADM 28 has been revised to require the graveyard shift supervisor to take corrective action to resolved problems uncovered in his daily review of logs.

(Open) Unresolved Items (50-267/76-13): Plant Operating Cycles. This matter has been assigned to Engineering on commitment log item number 454 which is due January 1, 1980.

(Closed) Unresolved Item (50-267/77-17): Core Safety Limit Procedure and T/S Change. Closed based on the submittal to NRR for a T/S change to SL 3.1., PSC letter P-79164 dated August 29, 1979.

(Open) Unresolved Item (50-267/79-01): PORC Review of Operating Items or Required by T/S 7.1.2.e.(7). A Policy procedure on this matter is due for issue on October 15, 1979.

(Closed) Open Item (50-267/79-07): Cleaning Schedule for Oil under Diesel Generators. A new PMO-M-37-W, dated July 24, 1979, now provides for weekly inspection and cleanup of these areas.

(Closed) Open Item (50-267/79-10): Procedures require revision. The procedure requiring revision or clarification, reviewed by the inspector during inspection 79-10, have now been revised or corrected.

3. Physical Protection

The inspector reviewed portions of the licensee's physical protection program by review of selected records and observation of the activities of access control, search, escorting, and communications.

No items of noncompliance or deviations were identified.

4. Review of Plant Operations

A review of plant operations was conducted to ascertain whether facility operation was in conformance with the requirements established in Technical Specifications, 10 CFR, and Administrative Procedures, or licensee commitments discussed in correspondence to the Commission.

Included in the inspection were observation of control room activities, review of operational log records, and a tour of accessible areas. Logs and records reviewed included:

Shift Supervisor Logs
 Reactor Operator Logs
 Technical Specification Compliance Log
 Operating Order Book
 Form 1 Log (Jumper Log)
 Plant Trouble Reports
 Selected Valve Lineups

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During the tour of accessible areas, particular attention was directed to the following:

Monitoring Instrumentation
 Radiation Controls
 Housekeeping
 Fluid Leaks
 Piping Vibration
 Hanger/Seismic Restraints
 Clearance Tags
 Fire Hazards
 Control Room Manning
 Annunciators

During the tour the inspector noted that all three doors to the ACM Diesel Generator were unlocked. Since the generator provides power independent of the plant distribution network to equipment or an alternate means of maintaining PCPW Cooling during the loss of forced circulation, access to the generator should be controlled. Discussion with the licensee indicated that the keys to the diesel generator door had been collected and that the doors would be kept locked and the keys placed under control. Additionally, the doors would be placed on the routine patrol checklist. This matter remains as an unresolved item.

No items of noncompliance or deviations were identified.

5. Maintenance

The inspector reviewed selected maintenance activities to determine that these activities met acceptable maintenance practices, the licensee's applicable administrative and maintenance procedures, and where applicable were in accordance with Technical Specification. Additionally, the inspector observed portions of maintenance activities to verify that procedural requirements were met, personnel were qualified, QC hold points were provided. Tagging procedures were appropriate and the items were not reportable as reportable occurrence.

The maintenance activities reviewed were:

PTR-6-612 Temperature indicates TE-2323 does not appear to be correct.
 Calibrated per RP-163A.

PTR-7-49 Remove FE 2368 and install a temporary bypass line around FE 2368. Completed per CN-1090 and MP-100.

PTR-7-314 Compressor C-2302 needs total rebuild.

PTR-5-263 Shaft Seal gave on P-2109, replaced shaft per cup 405, CN-1031 and MP-21-10.

PTR-6-55 Bearing Water "Y" strainer on "D" circulator dirty. Completed per PM 21.45.

PTR-6-477 "B" Circulator Buffer Supply, filter dirty. Completed per MP-31.
 PTR-6-478 "C" Circulator Buffer Supply, filter dirty. Completed per MP-31.
 PTR-7-99 Relief (V-21268-2) in Backup Bearing Water line has hole in casting and relief in lifting. Repaired per MP-29.
 PTR-8-131 SV-2106 "C" Circulator Steam SV Operator filter dirty. Completed per MP-91.6.

It was noted during the review of the records stated above that certain portions of page 2 of the 31 for PTR-6-477 and 6-478 were not completed as required by Administrative Procedure No. 12. These same types of omissions had been noted as an item of noncompliance in IE report 50-267/79-13. Corrective Action had been completed for those omission noted in that report. This matter was discussed with a representative of the licensee who informed the inspector that the subject of compliance with regulations was the subject of a letter issued to all Supervisors and Managers on September 12, 1979. Moreover, work on all of the above noted PTRs had been completed prior to the IE report and the letter noted previously.

The maintenance activities reviewed and observed were:

SR 5.4.1.1.9 b-m and SR 5.4.1.1.2.9 a-m, Primary Coolant Pressure Scram Test.
 PTR-8-604 Removal and Replacement of 4160/480 Transformer. Completed per EMP-35.
 PTR-7-496 MSV-2247 Leaking Through. Completed per MP-16.

No items of noncompliance or deviations were identified.

6. Startup Report

The twelfth Startup Report for Fort St. Vrain for the period May 22, 1979 through August 22, 1979, reviewed by the inspector, contained no relevant information on the initial approach to power tests since none were performed during this period of time.

7. IE Circulars/Bulletins

The inspector verified by record review, observation, and discussion with representatives of the licensee, the actions taken in response to IE Circulars/Bulletins.

The following Circulars were reviewed:

79-05 Moisture Leakage in Stranded Wire Conductors.
 79-09 Occurrences of Split or Punctured Regulator Diaphragms in Certain Self-Contained Breathing Apparatus.

- 79-12 Potential Diesel Generator Turbocharger Problem.
- 79-13 Replacement of Diesel Fire Pump Starting Contactors.
- 79-15 Bursting of High Pressure Hose and Malfunction of Relief Valve and "O"-Ring in Certain Self-Contained Breathing Apparatus.
- 79-17 Contact Problem in SB-12 Switches on General Electric Metalclad Circuit Breakers.

The following Bulletins were reviewed:

- 79-14 (Open) Seismic Analysis For As-Built Safety-Related Piping Systems.
The inspector reviewed the licensee's organization and qualifications for those individuals who developed the initial random sample inspection elements. Also reviewed were portions of the documentation of the analysis performed on the hangers.

Hanger analysis was completed by General Atomic (GA) and line analysis was done by Public Service Company (PSC). However, since the initial random sample audit and analysis, the licensee has decided to hire a firm to do the analysis. GA is to conduct the initial hanger inspection utilizing Stearns-Roger forces, marking up existing hanger drawings if necessary. This information will then be provided to a draftsman for redrawing. The redrawn hanger drawings will then proceed on to GA Quality Assurance who will provide 100% audit of the hangers. If these are found to be correct, they will then be given to an engineer who will initially decide (sort) into categories of major and minor. From the check engineer it will go to the analyst. The results of the analyst's work will be checked by another analyst and after this check is completed and a fix designed, if required, a final 100% audit will be made by PSC Quality Assurance.

Observation of the physical inspection was performed by the inspector by witnessing the inspection of Hanger 11A-H-46463, the inspection utilized a procedure developed by GA (SDCC Procedure No. 005, Procedure for Field Auditing 2-1/2 Inch and Larger Class I Piping, Isometrics and Hangers). Observations confirmed that the marked up hanger drawings reflect the work being done as required by the procedure. The procedure merely requires data taking and no decision making process is involved. However, an independent inspection of three hangers (11A-H-46451, 11A-H-46446 and 11A-H-46448) by the inspector indicated that some errors are being made in the data taking. No errors were noted for two of the hangers; however, the marked up drawing for Hanger 11A-H-46448 did not contain information relative to four cutoff bolts welded into the hanger. While these findings have been discussed with the licensee, corrective measures have not been taken since it is prudent at this point in time

to observe whether the first QA audit will find these omissions. For this reason this will be held as an open item pending a review of the first QA audit.

The PSC letter P-79178 dated September 4, 1979 contained the initial results of the hanger audit as reported by PSC. There were however, 14 unqualified hangers at that time. Nine of these were eventually qualified by reanalysis. Four of the remaining five were considered as failed hanger but the system was found to be unimpaired. After considerable analysis, the remaining hanger, 9A-H-46130, was finally repaired to bring it into conformance with original requirements for the hanger, i.e., required eight member welds on "H" member structure to be increased from 3/16 inch to 1/8 inch (the originally required weld size). An analysis of the repaired hanger indicated it was unimpaired.

At the present time the hanger inspection (markup of drawings) is pending the establishment of acceptance criteria for the analyst and establishment of a contract with a firm to do the analysis.

79-15 (Closed) Deep Draft Pump Deficiencies. The inspector reviewed the information contained in the licensee's reply P-99208 dated September 14, 1978, and reviewed selected records maintained by the licensee as required by the bulletin. Based on the licensee's reply and the inspectors selective review of records it appears that the deep draft pumps used at Fort St. Vrain have not experienced the difficulties noted in the IE Bulletin.

79-18 (Closed) Auditability Problems Encountered on Evacuation of Personnel From High-Noise Area. Closed out based on the corrective action taken as outlined in PSC letter P-79204 dated September 10, 1979.

8. Exit Interviews

Exit interviews were conducted at the end of various segments of this inspection with Mr. D. Warembourg (Manager, Nuclear Production) and other members of the Public Service Company staff. At the interviews, the inspector discussed the findings indicated in the previous paragraphs. The licensee acknowledged these findings.

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