UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

OCT 2 2 1979

MEMORANDUM FOR: Paul S. Check, Chief, Reactor Safety Branch, Division of Operating Reactors

THRU: Franklin D. Coffman, Section Leader, Section B, Reactor Safety Branch, Division of Operating Reactors

FROM: Robert Riggs, Reactor Safety Branch, Division of Operating Reactors

SUBJECT: MEETING SUMMARY

On October 12, 1979, members of the NRC staff met with members of the Westinghouse staff. The list of Attendees is provided in Attachment A. The purpose of the meeting was to discuss Westinghouse data and analysis techniques related to control rod guide thimble wear.

The Agenda of the meeting followed the format as provided in Attachment B. During the meeting, Westinghouse discussed and provided "Draft" responses to questions the NRC staff formulated (References 1 and 2) concerning previous Westinghouse submittals (References 3, 4 and 5) on this subject. The information was supplemented during the discussion by additional "Draft" material. The contents of the "Draft" materials and the discussions are proprietary to Westinghouse.

At the end of the presentation, the NRC staff concluded that the information provided by Westinghouse was responsive to the NRC questions of References 1 and 2 with one exception: The one exception was related to confirmatory surveillance (question 4 of References 1 and 2) which will be redrafted for NRC comment prior to issuance of the formal response.

The consenses of opinions by the NRC staff was as follows:

1) No throughwall wear was observed in control rod guide thimbles in Westinghouse fuel at Westinghouse NSSS facilities.

2) Westinghouse techniques for measuring the guide thimble wear appear acceptable.

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3) The analysis techniques used by Westinghouse to assess the structural integrity of worn thimbles are adequately conservative provided the conservatism of the Westinghouse mechanistic wear model can be confirmed.

4) The bounding analysis submitted by Westinghouse for the 15x15 and 17x17 fuel design should be confirmed by PIE examination (see exception above).

The formal Westinghouse response to our questions on guide thimble wear will be submitted by early November 1979, assuming that a satisfactory resolution of the one exception (additional surveillance of 15x15 and 17x17 fuel) can be worked out. At that time, DOR will issue a letter addressing NRC acceptance of the Westinghouse analysis of the guide thimble wear problem. DSS will address acceptability on the docket of each new plant in question.

The DOR and DSS statements should be fully coordinated through R. Riggs (RSB/DOR) and D. Powers (CPB/DSS).

Robert Riggs
Reactor Safety Branch
Division of Operating Reactors

Enclosure:
As stated

References

1. B. K. Grimes (USNRC) letter to T. M. Anderson (W), dated September 7, 1979.

2. K. Kniel (USNRC) memorandum to O. Parr (USNRC), "Guide Thimble Tube Wear in Westinghouse Fuel Assemblies," September 4, 1979.

3. T. M. Anderson (W), letter (NS-TMA-1936) to D. G. Eisenhut (USNRC), dated September 12, 1978.

4. T. M. Anderson (W) letter (NS-TMA-1992) to D. G. Eisenhut (USNRC) dated December 15, 1978.

5. T. M. Anderson (W) letter (NS-TMA-2102) to D. G. Eisenhut (USNRC), dated June 27, 1979.

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ATTACHMENT A

Attendees - 10/12/79

R. Riggs	RSB/DOR
J. Reavis	W NFD
G. Antaki	W NFD
M. Arlotti	W NFD
F. Ellingson	W NFD
R. O. Meyer	CPB/DSS
D. A. Powers	CPB/DSS
F. D. Coffman	RSB/DOR
D. K. Hsu	Public Service Electric & Gas
N. P. Wolfhope	VEPCO
D. J. Vito	STSG/DOR/NR
P. C. Wagner	STSG/DOR/NRC
D. Houston	CPB/DSS
R. Lobel	RSB/DOR
T. Heitman	Duke Power
M. Connor	ORB/DOR
S. Sands	RSB/DOR
T. Alexion	RSB/DOR
L. G. Pilgrim	W NFD

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