

OPERATING DATA REPORT

DOCKET NO. 50-266

DATE November 6, 1979

COMPLETED BY SOL BURSTEIN

TELEPHONE 414 277 2121

OPERATING STATUS

- 1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 1
- 2. REPORTING PERIOD: OCTOBER 1979
- 3. LICENSED THERMAL POWER (MWT): 1518.
- 4. NAMEPLATE RATING (GROSS MWE): 523.8
- 5. DESIGN ELECTRICAL RATING (NET MWE): 497.
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
- 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
- 10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

THIS MONTH YR TO DATE CUMULATIVE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745	7,296	78,768
12. NUMBER OF HOURS REACTOR WAS CRITICAL	105.3	6,049.0	65,936.9
13. REACTOR RESERVE SHUTDOWN HOURS	3.3	9.4	449.0
14. HOURS GENERATOR ON LINE	98.0	5,991.4	63,928.4
15. UNIT RESERVE SHUTDOWN HOURS	0.0	6.3	389.7
16. GROSS THERMAL ENERGY GENERATED (MWH)	141,578	8,879,112	90,245,727
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	47,870	3,039,190	30,466,830
18. NET ELECTRICAL ENERGY GENERATED (MWH)	44,390	2,903,123	29,037,877
19. UNIT SERVICE FACTOR	13.2	82.1	81.2
20. UNIT AVAILABILITY FACTOR	13.2	82.2	81.7
21. UNIT CAPACITY FACTOR (USING MDC NET)	12.0	80.4	75.8
22. UNIT CAPACITY FACTOR (USING DER NET)	12.0	80.1	74.2
23. UNIT FORCED OUTAGE RATE	0.0	5.6	3.2
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): NONE			

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: November 24, 1979

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

POOR ORIGINAL

1364 141 7911200396

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE November 6, 1979

COMPLETED BY SOL BURSTEIN

TELEPHONE 414 277 2121

OPERATING STATUS

- 1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 2
- 2. REPORTING PERIOD: OCTOBER 1979
- 3. LICENSED THERMAL POWER (MWT): 1518.
- 4. NAMEPLATE RATING (GROSS MWE): 523.8
- 5. DESIGN ELECTRICAL RATING (NET MWE): 497.
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
- 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
- 10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745	7,296	63,553
12. NUMBER OF HOURS REACTOR WAS CRITICAL	745.0	6,347.3	57,436.4
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	16.1	165.7
14. HOURS GENERATOR ON LINE	745.0	6,272.8	56,267.6
15. UNIT RESERVE SHUTDOWN HOURS	0.0	18.6	106.4
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,117,434	9,226,269	75,511,486
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	383,260	3,165,970	25,641,080
18. NET ELECTRICAL ENERGY GENERATED (MWH)	366,296	3,019,693	24,387,370
19. UNIT SERVICE FACTOR	100.0	86.0	88.5
20. UNIT AVAILABILITY FACTOR	100.0	86.2	88.1
21. UNIT CAPACITY FACTOR (USING MDC NET)	99.3	83.6	78.3
22. UNIT CAPACITY FACTOR (USING DER NET)	98.9	83.3	77.2
23. UNIT FORCED OUTAGE RATE	0.0	0.0	1.0
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refueling shutdown scheduled for March 7, 1980, to last approximately five weeks.			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

POOR ORIGINAL

1361142

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1979

DOCKET NO. 50-266
 UNIT NAME Point Beach Unit 1
 DATE November 6, 1979
 COMPLETED BY Sol Burstein
 TELEPHONE 414/277-2121

No.	Date	Type ¹	Duration (Hours) -	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
7	791005	S	647.0	C	1	N/A	N/A	N/A	

¹ F: Forced
 S: Scheduled

² Reason:
 A- Equipment Failure (explain)
 B- Maintenance or Test
 C- Refueling
 D- Regulatory Restriction
 E- Operator Training & License Exam
 F- Administrative
 G- Operational Error (explain)
 H- Other (explain)

³ Method:
 1- Manual
 2- Manual Scram
 3- Automatic Scram
 4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1979

DOCKET NO. 50-301
 UNIT NAME Point Beach Unit 2
 DATE November 6, 1979
 COMPLETED BY Sol Burstein
 TELEPHONE 414/277-2121

No.	Date	Type ¹	Duration (Hours) -	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component: Code	Cause and Corrective Action To Prevent Recurrence

¹ F: Forced
S: Scheduled

² Reason:
 A- Equipment Failure (explain)
 B- Maintenance or Test
 C- Refueling
 D- Regulatory Restriction
 E- Operator Training & License Exam
 F- Administrative
 G- Operational Error (explain)
 H- Other (explain)

³ Method:
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 4- Other (explain)

⁴ Exhibit G-Instruc-
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 ation of Data Entry
 Sheets for LER File
 (NUREG-0161)

⁵ Exhibit I- Same
 Source

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DOCKET NO. 50-266
 UNIT NAME Point Beach Unit 1
 DATE November 6, 1979
 COMPLETED BY Sol Burstein
 TELEPHONE 414/277-2121

AVERAGE DAILY UNIT POWER LEVEL

MONTH October, 1979

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>483</u>	11	<u>-1</u>	21	<u>-1</u>
2	<u>480</u>	12	<u>-1</u>	22	<u>-1</u>
3	<u>451</u>	13	<u>-1</u>	23	<u>-1</u>
4	<u>474</u>	14	<u>-1</u>	24	<u>-1</u>
5	<u>4</u>	15	<u>-1</u>	25	<u>-1</u>
6	<u>- 6</u>	16	<u>-1</u>	26	<u>-1</u>
7	<u>- 3</u>	17	<u>-1</u>	27	<u>-1</u>
8	<u>- 2</u>	18	<u>-1</u>	28	<u>-1</u>
9	<u>- 1</u>	19	<u>-1</u>	29	<u>-1</u>
10	<u>- 1</u>	20	<u>-1</u>	30	<u>-1</u>
				31	<u>-1</u>

1761 145

DOCKET NO. 50-301
 UNIT NAME Point Beach Unit 2
 DATE November 6, 1979
 COMPLETED BY Sol Burstein
 TELEPHONE 414/277-2121

AVERAGE DAILY UNIT POWER LEVEL

MONTH October, 1979

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>493</u>	11	<u>492</u>	21	<u>491</u>
2	<u>493</u>	12	<u>491</u>	22	<u>492</u>
3	<u>493</u>	13	<u>483</u>	23	<u>492</u>
4	<u>493</u>	14	<u>494</u>	24	<u>491</u>
5	<u>493</u>	15	<u>492</u>	25	<u>491</u>
6	<u>495</u>	16	<u>492</u>	26	<u>492</u>
7	<u>488</u>	17	<u>492</u>	27	<u>492</u>
8	<u>491</u>	18	<u>491</u>	28	<u>492</u>
9	<u>492</u>	19	<u>493</u>	29	<u>491</u>
10	<u>491</u>	20	<u>496</u>	30	<u>492</u>
				31	<u>490</u>

1361 146

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-266
Unit Name Point Beach Unit 1
Date November 6, 1979
Completed By Sol Burstein
Telephone 414/277-2121

Unit 1 was base loaded for only about 12 per cent of the period. The unit had one load reduction to 390 MWe net for nine and one-half hours on October 3, 1979, extending into October 4, 1979, because of boron addition caused by equilibration of the mixed bed demineralizer when it was put in service to check the resin. Unit capacity dropped to 480 MWe net because of core stretch operation prior to Refueling 7 shutdown on October 5, 1979. The unit was taken off line at 1:57 a.m. on October 5, 1979, and cool-down commenced at 12:35 of the same day after completion of end-of-life physics testing and operator licensing exams.

The reactor vessel head was stripped and removed at 8:28 a.m. on October 11, 1979. Fuel movement was initiated at 8:58 p.m. on October 11, 1979, and completed at 4:24 a.m. on October 14, 1979. Installation of the closure head and connection of head area vent ducting, etc., was completed on October 22, 1979.

Sludge lancing of both "A" and "B" steam generators was performed during the outage, with approximately 34 gallons of sludge being removed from "A" and 62 gallons removed from the "B" steam generator.

The initial eddy current inspection program of steam generator tubes consisted of inspecting 37 tubes in the "A" steam generator and 38 in the "B" steam generator. These tubes were suspected as potentially defective based on a reanalysis of eddy current results from a 100% inspection in August, 1979. The inspection identified 13 tubes in the "A" and six tubes in the "B" steam generator that require plugging. Because an additional defective tube was discovered during a subsequent testing program, a 200 tube sample inspection was performed in the "A" steam generator. The 200 tube sample inspection identified more defective tubes so the eddy current inspection program was expanded to a 100% inspection in compliance with the Technical Specifications. A total of 70 tubes with defects which require plugging were discovered in the "A" steam generator. In addition, three tubes with defects less than 40% will be plugged, and two good tubes which are being pulled to ascertain the overall condition of the steam generators will also be plugged.

This raises the total number of tubes that will be plugged in "A" to 75.* In the "B" steam generator, 65 defective tubes have been identified and will be plugged. All of the defects found are within the tubesheet in both steam generators. An over the U-bend inspection of the first row tubes in "A" was also performed as a precautionary inspection; no defects were identified.

Three tubes were pulled from the "A" steam generator. These tubes were pulled to investigate deep crevice cracking (cracking of tubes within the tubesheet) and to determine the overall condition of the steam generators. Tube R15C45 was identified by eddy current inspection as having a defect within the tubesheet and was pulled for investigative study. Tubes R20C73 and R22C37 were both identified as good tubes by eddy current inspection, but were pulled to ascertain the condition of the steam generators. Tube R20C73 represents the area where deep crevice cracking has not been a problem, whereas tube R22C37 is in the area where deep crevice cracking has occurred. All three tubes have been sent to Westinghouse Research for evaluation.

The steam generator 18" x 16" feed line nozzle reducers and a portion of the piping in each steam generator feed line have been replaced. No cracking was visually evident in the replaced nozzles.

The "A" reactor coolant pump upper motor bearings were removed for inspection and replaced. The "B" reactor coolant pump motor was removed for an inspection, and the pump seals were replaced. Assembly of both pumps is complete.

Low pressure turbine LP1 was dismantled and cleaned. Weld repairs were made on the housings and an ultrasonic inspection of the rotor discs was performed. Low pressure turbine LP2 was dismantled so that an ultrasonic inspection of its rotor discs could be performed also. Indications were discovered in the second and third discs of LP1; no indications were discovered in the LP2 discs. A detailed evaluation has shown that the discs will require replacement in the future but that the rotor may be operated for at least 28 months with no significant reduction in safety margin.

Work on IE Bulletins 79-02, anchor bolt inspection, and 79-14, seismic analysis for as-built safety-related piping systems is being performed in containment. Work on inspection of lines containing stagnant borated water, IE Bulletin 79-17, is complete.

The No. 4 feedwater heaters have been cleaned in preparation for an eddy current inspection of the tubes. Moisture separator reheater cleaning and general maintenance is being performed.

Other general work being performed this period includes containment recirculating fan maintenance and balancing, refueling interval calibrations, safety valve work, incore instrumentation thimble work, crossunder and extraction piping changeout and inspection and addition of charging pump stabilizers.

Operations refueling test, ORT #3, loss of AC power was satisfactorily completed at 12:35 p.m. on October 26, 1979.

The 3D emergency Diesel generator was returned to service at 1:00 p.m. on October 5, 1979, after replacement of a defective reconditioned governor. Licensee Event Report No. 79-016/03L-0 has been filed on the failure of the Diesel to start during a test on October 3, 1979.

Licensee Event Report No. 79-015/01T-0 has been filed describing reduced redundancy of containment pressure transmitters and possible violation of containment integrity occasioned by a loose swagelok fitting on a containment pressure instrument line.

No other major safety-related maintenance was performed during the period.

1364 149

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-301
Unit Name Point Beach Unit 2
Date November 6, 1979
Completed By Sol Burstein
Telephone 414/277-2121

Unit 2 was base loaded for nearly 100% of the period with two load reductions.

Unit 2 load was reduced to 345 MWe net for two hours on October 13, 1979, in order to perform turbine stop valve testing. The second load reduction occurred on October 31, 1979, when load was reduced to 470 MWe net for one hour to run a performance test on the No. 4 feedwater heater.

No major safety-related maintenance was performed during the period.

1361 150