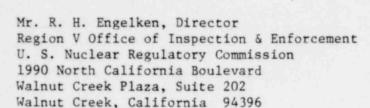


SACRAMENTO MUNICIPAL UTILITY DISTRICT 
6201 S Street, Box 15830, Sacramento, California 95813; (916) 452-3211

November 9, 1979



Re: Operating License DPR-54 Docket No. 50-312

Reportable Occurrence 79-14

Dear Mr. Engelken:

In accordance with Technical Specifications for Rancho Seco Nuclear Generating Station, Section 6.9.4.2b, and Regulatory Guide 1.16, Revision 4, Section C.2.b.2, the Sacramento Municipal Utility District is hereby submitting a thirty-day report of Reportable Occurrence 79-14.

While performing the required inspection/analysis review of safety-related piping systems required by I&E Bulletin 79-14, the District became aware of a potential nonconformance between the design documents and the original seismic analyses on the "A" Nuclear Service Cooling Water System.

To accomplish the requirements of I&E Bulletin 79-14 in the appropriate time frame, the Architect Engineer, Bechtel Norwalk, was contracted to assist in the review and analysis. On October 8, 1979, Bechtel informed the District that their analyses indicated the necessity for an additional support on line No. 48222-18" HE. This is the discharge line of the "A" NSCW pump P-482A. The original seismic analysis took into account a support in this area. However, the design documents and construction drawings did not show a support. As a result, the as-built configuration and design documents, although in agreement, did not conform to the original seismic analysis. The present analysis indicated that without the axial restraints provided by the support, the seismic DBE stress would exceed the allowable design stresses and the pipe loading on P-482A would exceed the manufacturer's allowable nozzle load.

Upon receipt of this information, the "A" NSCW system was declared inoperable. In accordance with Technical Specifications Section 3.3.1, a reactor shutdown was commenced within 48 hours and the reactor was in hot shutdown within an additional 12 hours. Simultaneously, work was progressing on the fabrication and installation of the necessary support. This was completed on October 13, 1979. With the added support, the "A" NSCW system was determined operable and a reactor startup commenced. The unit was back to 100% power on October 14, 1979.

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Page 2 November 9, 1979 R. H. Engelken On October 16, 1979, Bechtel informed the District that additional analysis on line No. 48222-18" HE indicated that even with the addition of the support installed the previous week, another support was required and two support modifications were necessary to satisfy the design criteria. Again, upon receipt of this information, the "A" NSCW system was declared inoperable. Work was started on the fabrication and installation of the additional support and modifications to two existing supports. This was completed within the 48 hours allowable per Technical Specifications Section 3.3.1 for continued reactor operation. In summary, although the as-built configuration for the supports was in accordance with their design drawings, a review and analysis of the "A" NSCW system revealed discrepancies relative to the original seismic analyses. Upon the October 8, 1979, notification, an additional seismic support was installed. This new support was assigned the I.D. number 8G48222-6. On October 16, 1979, based upon a reanalysis, further discrepancies were brought to the District's attention. This prompted the installation of a second additional support and modification of two existing supports. The second additional support was assigned the I.E. number 7G48222-12. The two existing supports which were modified were I.D. No. 7A48222-9 and I.D. No. 7A48222-7. Due to Technical Specifications time criteria for continued reactor operation with a safety-related system out of service, the unit was shut down and off line for approximately 70 hours while the October 8, 1979, discrepancy was corrected. Respectfully submitted, Wm. C. Walbridge General Manager JJM: RWC: HH: jim 1261 138