AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-368
UNIT	ANO-2
DATE	_11/14/79
COMPLETED BY	R. A. Pendergraft
TEL EPHONE	501-968-2519

MON	THOctober 1979		
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL
1	307	17	0
2	389	18	0
3	398	19	0
4	101	20	0
5	0	21	0
6	0	22	0
7		23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

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INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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OPERATING DATA REPORT

Notes

DOCKET NO.	50-368
DATE	11/14/79
COMPLETED BY	R.A. Pendergraft
TELEPHONE	501-968-2519

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2	
2. Reporting Period: October 1-31, 1979	
3. Licensed Thermal Power (MWt): 2815	김 귀엽에 대한 영습을 받는 것을 줄
4. Nameplate Rating (Gross MWe)	
5. Design Electrical Rating (Net MWe): 912	
6. Maximum Dependable Capacity (Gross MWe):	이 방법 모양에 대해야 한다. 나는
7. Maximum Dependable Capacity (Net MWe)NA	

8. If Char.ges Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Non

9. Power Level To Which Restricted. If Any (Net MWe): <u>None</u> 10. Reasons For Restrictions. If Any: <u>NA</u>

	This Month	Yrto-Date	Cumulative
11. Hours In Reporting Period	745.0	7296.0	8040.0
12. Number Of Hours Reactor Was Critical	78.0	2787.6	3222.5
13 Reactor Reserve Shutdown Hours	0.0	2142.9	2285.1
14. Hours Generator On-Line	78.0	2447.0	2499.8
15. Unit Reserve Shutdown Hours	0.0	21.7	21.7
16. Gross Thermal Energy Generated (MWH)	105867.0	2638959.0	2683537.0
17. Gross Electrical Energy Generated (MWH)	31215.0	726523.0	732090.0
18. Net Electrical Energy Generated (MWH)	28698.0 .	648941.0	652925.0
19. Unit Service Factor	1		
20. Unit Availability Factor		-	
21. Unit Capacity Factor (Using MDC Net)	X NA UNTIL	COMMERCIAL OPER	ATION
22. Unit Capacity Factor (Using DER Net)	1	_	
23. Unit Forced Outage Rate			
24. Shutdowns Scheduled Over Next 6 Months (Ty	pe. Date, and Duration	of Each):	

None

November 22, 1979 25. if Shut Down At End Of Report Period. Estimated Date of Startup: _ 26. Units In Test Status (Prior to Commercial Operation): Forecast Achieved 12-5-78 INITIAL CRITICALITY 12-26-78 INITIAL ELECTRICITY Jan. 1980 COMMERCIAL OPERATION

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

50-368 DOCKET NO. DOCKETNO UNITNAME DATE COMPLETED BY TELEPHONE ANO-Unit 2 11/14/79 R. A. Pendergraft 501-968-2519

No.	Date	Type ¹	Duration (Hours)	Reason?	Method of Shutting Down Reactor3	Licensee Event Report =	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
79-22	10-4-79	S	667	н	2	None	IF	AIRDRY	Loss of Instrument Air caused Unit to trip on same day as scheduled outage was to occur. Outage was scheduled for Reactor Internals Inspection.
F: For S Sch	ced eduled	2 Reaso A-Equ B-Mai C-Ref D-Reg E-Opt F-Opt F-Opt G-Opt G-Opt	n: aipment Fa atenance o vieling gulatory Re crator Train crator Train crational te crational te	ilure (E) r Test striction ing & L	xplain) icense Exam plain)	ination	3 Methor 1-Manu 2-Manu 3-Auto 4-Othe	d: ral ral Scram. matic Scram. r (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREC 0161) 5 Exhibit 1 - Same Source

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REFUELING INFORMATION

Scheduled date for next refueling shutdown. 2/1/81 Scheduled date for restart following refueling. 4/1/81 Will refueling or resumption of operation thereafter requir technical specification change or other license amendment? If answer is yes, what. in general, will these be? If answer is no, has the reload fuel design and core config been reviewed by your Plant Safety Review Committee to dete whether any unreviewed safety questions are associated with core reload (Ref. 10 CFR Section 50.59)? Yes. Description of effects of new core loading	
Scheduled date for restart following refueling. 4/1/81 Will refueling or resumption of operation thereafter requir technical specification change or other license amendment? If answer is yes, what. in general, will these be? If answer is no, has the reload fuel design and core config been reviewed by your Plant Safety Review Committee to dete whether any unreviewed safety questions are associated with core reload (Ref. 10 CFR Section 50.59)? Yes. Description of effects of new core loading	
Will refueling or resumption of operation thereafter requir technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core config been reviewed by your Plant Safety Review Committee to dete whether any unreviewed safety questions are associated with core reload (Ref. 10 CFR Section 50.59)? Yes. Description of effects of new core loading 	
Yes. Description of effects of new core loading	uration mine the
Scheduled date(s) for submitting proposed licensing action supporting information. 12/1/80 Important licensing considerations associated with refueling new or different fuel design or supplier, unreviewed design performance analysis methods, significant changes in fuel do new operating procedures. NONE	
Scheduled date(s) for submitting proposed licensing action supporting information. 12/1/80 Important licensing considerations associated with refueling new or different fuel design or supplier, unreviewed design performance analysis methods, significant changes in fuel denew operating procedures.	
<pre>Important licensing considerations associated with refuelin new or different fuel design or supplier, unreviewed design performance analysis methods, significant changes in fuel do new operating procedures. </pre>	and
The number of fuel assemblies (a) in the core and (b) in the fuel storage pool. a) 177 b) 0 The present licensed spent fuel pool storage capacity and the of any increase in licensed storage capacity that has the fuel pool storage capacity and the fuel pool stor	g, e.g., or esign,
The number of fuel assemblies (a) in the core and (b) in the fuel storage pool. a) 177 b) 0 The present licensed spent fuel pool storage capacity and the	
The number of fuel assemblies (a) in the core and (b) in the fuel storage pool. a) 177 b) 0 The present licensed spent fuel pool storage capacity and the fuel pool storage capacity and the fuel pool storage capacity that has been been been been been been been bee	
The present licensed spent fuel pool storage capacity and the	e spent
or is planned. 'n number of fuel assemblies.	ne size requested
present486increase size by566	3. 1. 1
The projected date of the last refueling that can be dischar to the spent fuel pool assuming the present licensed capacit	rged
DATE:1993	

NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY - OCTOBER, 1979

UNIT II

The 50% power plateau testing continued from the previous month. At 0600 on 10-4, the reactor was manually tripped when all instrument air was lost due to an inoperable instrument air dryer valve. This trip occurred on the same day as the scheduled outage was to occur. The unit has remained at 0 power for the remainder of the month while reactor internals inspection and diesel generator replacement is being accomplished.

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