

**SEPTEMBER 30, 1979**  
**NUREG—0485**  
**VOL. 2 NO. 2**

1365 051

# **REGULATORY LICENSING**

<p><b>UNITED STATES NUCLEAR REGULATORY COMMISSION</b></p>	<p><b>STATUS SUMMARY REPORT</b></p>
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POOR ORIGINAL

**SYSTEMATIC EVALUATION PROGRAM**

7911200 141

**SEPTEMBER 30, 1979**  
**NUREG-0485**  
**VOL. 2 NO. 2**

1365 052

# **REGULATORY LICENSING**

<b>UNITED STATES NUCLEAR REGULATORY COMMISSION</b>	<b>STATUS SUMMARY REPORT</b>
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**SYSTEMATIC EVALUATION PROGRAM**

Available from

GPO Sales Program  
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U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

and

National Technical Information Service  
Springfield, Virginia 22161

1365 053

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# SECTION I

# DESIGN BASIS EVENT REVIEWS

## THIS SECTION DEFINES DESIGN BASIS EVENT (DBE) REVIEWS FOR THE ELEVEN SEP FACILITIES

### ACCIDENT AND TRANSIENT GROUPS:

#### GROUP I (PWR):

DECREASE IN FEEDWATER TEMPERATURE (TOPIC XV-1)  
INCREASE IN FEEDWATER FLOW (TOPIC XV-1)  
INCREASE IN STEAM FLOW (TOPIC XV-1)  
INADVERTENT OPENING OF STEAM GENERATOR RELIEF/SAFETY VALVE (TOPIC XV-1)  
STARTUP OF INACTIVE LOOP (TOPIC XV-9)  
SYSTEM MALFUNCTION CAUSING BORON DILUTION (TOPIC XV-10)

#### GROUP I (BWR):

DECREASE IN FEEDWATER TEMPERATURE (TOPIC XV-1)  
INCREASE IN FEEDWATER FLOW (TOPIC XV-1)  
INCREASE IN STEAM FLOW (TOPIC XV-1)  
STARTUP OF INACTIVE LOOP (TOPIC XV-1)  
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#### GROUP II:

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LOSS OF CONDENSER VACUUM (TOPIC XV-3)  
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STEAM LINE BREAK INSIDE CONTAINMENT (TOPIC XV-2)  
STEAM LINE BREAK OUTSIDE CONTAINMENT (TOPIC XV-2)

#### GROUP IV:

LOSS OF AC POWER TO STATION AUXILIARIES (TOPIC XV-4)  
LOSS OF ALL AC POWER (TOPIC XV-24)

#### GROUP V:

LOSS OF FORCED COOLANT FLOW (TOPIC XV-7)  
PRIMARY PUMP ROTOR SEIZURE (TOPIC XV-7)  
PRIMARY PUMP SHAFT BREAK (TOPIC XV-7)

#### GROUP VI:

UNCONTROLLED ROD ASSEMBLY WITHDRAWAL AT POWER (TOPIC XV-13)  
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CONTROL ROD MISOPERATION (TOPIC XV-8)  
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#### GROUP VII:

SPECTRUM OF LOSS OF COOLANT ACCIDENTS (TOPIC XV-19)

#### GROUP VIII:

DROP OF CASK OR HEAVY EQUIPMENT (TOPIC XV-21)

#### GROUP IX:

INADVERTENT OPENING OF PWR PRESSURIZER RELIEF VALVE OR BWR SAFETY/RELIEF VALVE (TOPIC XV-15)

#### GROUP X:

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FLOODING POTENTIAL AND PROTECTION REQUIREMENTS (CAPABILITY OF OPERATING PLANT TO COPE WITH DESIGN BASIS FLOODING CONDITION) (TOPIC II-3.B)  
SAFETY RELATED WATER SUPPLY (ULTIMATE HEAT SINK) (TOPIC II-3.C)

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TECTONIC PROVINCE (TOPIC II-4.A)  
PROXIMITY OF CAPABLE TECTONIC STRUCTURES NEAR PLANT (TOPIC II-4.B)  
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TORNADO MISSILES (TOPIC III-4.A)  
TURBINE MISSILES (TOPIC III-4.B)  
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### WIND AND TORNADO LOADING: (TOPIC III-2)

1365 056

# SYSTEMATIC EVALUATION PROGRAM TOPIC LIST

## THIS SECTION DEFINES THE SEP SAFETY AND ENVIRONMENTAL TOPICS

- |   |  |
|---|--|
| <p>II <u>SITE CHARACTERISTICS</u></p> <p>II-1 <u>Site</u></p> <p>A. Exclusion Area Authority and Control</p> <p>B. Population Distribution</p> <p>* C. Potential Hazards or Changes in Potential Hazards Due to Transportation, Institutional, Industrial, and Military Facilities</p> <p>II-2 <u>Meteorology</u></p> <p>* A. Severe Weather Phenomena</p> <p>B. Onsite Meteorological Measurements Program</p> <p>C. Atmospheric Transport and Diffusion Characteristics for Accidents Analysis</p> <p>D. Availability of Meteorological Data in the Control Room</p> <p>II-3 <u>Hydrology</u></p> <p>A. Hydrologic Description</p> <p>* B. Flooding Potential and Protection Requirements</p> <p>1. Capability of Operating Plant to Cope with Design Basis Flooding Conditions</p> <p>C. Safety-related Water Supply (Ultimate Heat Sink [UHS])</p> <p>II-4 <u>Geology and Seismology</u> (These Topics to be Done with Seismic DBE)</p> <p>A. Tectonic Province</p> <p>* B. Proximity of Capable Tectonic Structures in Plant Vicinity</p> <p>C. Historical Seismicity within 200 Miles of Plant</p> <p>D. Stability of Slopes</p> <p>E. Dam Integrity</p> <p>F. Settlement of Foundations and Buried Equipment</p> <p>III <u>DESIGN OF STRUCTURES, COMPONENTS, EQUIPMENT, AND SYSTEMS</u></p> <p>III-1 <u>Classification of Structures, Components and Systems (Seismic and Quality)</u></p> <p>III-2 * <u>Wind and Tornado Loading</u></p> <p>III-3 <u>Hydrodynamic Loads</u></p> <p>A. Effects of High Water Level on Structures</p> <p>B. Structural and Other Consequences (e.g., Flooding of Safety-Related Equipment in Basements) of Failure of Underdrain Systems</p> <p>C. Inservice Inspection of Water Control Structures</p> | <p>III-4 <u>Missile Generation and Protection</u></p> <p>A. Tornado Missiles</p> <p>B. Turbine Missiles</p> <p>C. Internally Generated Missiles</p> <p>D. Site Proximity Missiles (Including Aircraft)</p> <p>III-5 <u>Evaluation of Pipe Breaks</u></p> <p>A. Effects of Pipe Break on Structures, Systems and Components Inside Containment</p> <p>B. Pipe Break Outside Containment</p> <p>III-7 <u>Category I Structures Integrity</u></p> <p>A. Inservice Inspection, Including Prestressed Concrete Containments with Either Grouted or Ungouted Tendons</p> <p>B. Design Codes, Design Criteria, Load Combinations, and Reactor Cavity Design Criteria</p> <p>C. Delamination of Prestressed Concrete Containment Structures</p> <p>D. Containment Structural Integrity Tests</p> <p>III-8 <u>Reactor Vessel Internals Integrity</u></p> <p>A. Loose Parts Monitoring and Core Barrel Vibration Monitoring</p> <p>B. Control Rod Drive Mechanism Integrity</p> <p>C. Irradiation Damage, Use of Sensitized Stainless Steel and Fatigue Resistance</p> <p>D. Core Supports and Fuel Integrity</p> <p>III-9 <u>Support Integrity</u></p> <p>III-10 <u>Pumps and Valves Integrity</u></p> <p>A. Thermal-Overload Protection for Motors of Motor-Operated Valves</p> <p>B. Pump Flywheel Integrity</p> <p>C. Surveillance Requirements on BWR Recirculation Pumps and Discharge Valves (2)</p> <p>III-11 <u>Component Integrity</u></p> <p>III-12 <u>Environmental Qualification of Safety Related Equipment</u></p> <p>IV <u>REACTOR</u></p> <p>IV-1 <u>Thermal Hydraulic Design and Performance</u></p> <p>A. Operation with Less Than All Loops In Service</p> <p>IV-2 <u>Reactivity Control Systems Including Functional Design and Protection Against Single Failures</u></p> <p>IV-3 <u>BWR Jet Pumps Operating Indications</u></p> |
|---|--|

\*Reviewed as part of Design Basis Event evaluation.

- (1) To be performed independent of SEP as a Division of Operating Reactors generic item.  
 (2) Not applicable to SEP facilities.  
 (3) To be performed independent of SEP as NRR Category A technical activity.  
 (4) Completed.

@Consideration being given to deleting topic from SEP review.

# SYSTEMATIC EVALUATION PROGRAM TOPIC LIST

1365 058

- |        |   |  |
|--------|---|--|
| V      | <u>REACTOR COOLANT SYSTEM AND CONNECTED SYSTEMS</u>   |  |
| @ V-1  | <u>Compliance with Codes and Standards (10 CFR 50.55a)</u>  |  |
| @ V-2  | <u>Applicable Code Cases</u>  |  |
| @ V-3  | <u>Overpressurization Protection (1,3)</u>  |  |
| V-4    | <u>Piping and Safe End Integrity</u>  |  |
| V-5    | <u>Reactor Coolant Pressure Boundary (RCPB) Leakage, Detection</u>  |  |
| V-6    | <u>Reactor Vessel Integrity</u>   |  |
| @ V-7  | <u>Reactor Coolant Pump Overspeed</u>   |  |
| @ V-8  | <u>Steam Generator (SG) Integrity (1,3)</u>   |  |
| V-9    | <u>Reactor Core Isolation Cooling System (BWR)</u>  |  |
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|        | B. RHR Reliability  |  |
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|        | B. RHR Interlock Requirements   |  |
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| @ V-13 | <u>Water Hammer (1,3)</u>   |  |
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| VI-2   | <u>Containment Functional Design</u>  |  |
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|        | @ B. Subcompartment Analysis  |  |
|        | @ C. Not applicable to the 11 facilities:   |  |
|        | D. Mass and Energy Release for Possible Pipe Break Inside Containment   |  |
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|        | 3. ECCS Actuation System  |  |
|        | 4. Core Spray Nozzle Effectiveness  |  |
|        | B. ESF Switchover from Injection to Recirculation Mode (Automatic ECCS Realignment)   |  |
|        | C. ECCS Single Failure Criterion and Requirements for Locking Out Power to Valves Including Independence of Interlocks on ECCS Valves |  |
|        | 1. Appendix K - Electrical Instrumentation and Control (EIC) Reviews  |  |
|        | 2. Failure Mode Analysis - ECCS   |  |
|        | 3. The Effect of PWR Loop Isolation Valve Closure During a LOCA on ECCS Performance   |  |
|        | D. Long Term Cooling - Passive Failures (e.g., Flooding of Redundant Components) (4)  |  |
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\*Reviewed as part of Design Basis Event evaluation.

- (1) To be performed independent of SEP as a Division of Operating Reactor generic item.
- (2) Not applicable to SEP facilities.
- (3) To be performed independent of SEP as NRR Category A technical activity.
- (4) Completed.



# SYSTEMATIC EVALUATION PROGRAM TOPIC LIST

VIII	<u>ELECTRIC POWER</u>		
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XV-2	<u>Spectrum of Steam System Piping Failures Inside and Outside of Containment (PWR)</u>		
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XVII		<u>OPERATIONAL QA PROGRAM (SEP) (4)</u>	

\* Reviewed as part of Design Basis Event evaluation.

(1) To be performed independent of SEP as a Division of Operating Reactors generic item.

(2) Not applicable to SEP for this activity.

(3) To be performed independent of SEP as NRR Category A technical activity.

(4) Completed.

# SEP TOPIC STATUS SUMMARY REPORT

NUMBER OF TOPICS	NOT STARTED (N)	STARTED BUT NO WRITTEN OUTPUT (S)	INFORMATION NEEDS IDENTIFIED (Q)	INITIAL ASSESSMENT UNDERWAY (I)	REVISED ASSESSMENT UNDERWAY (R)	COMPLETED (C)	UNDER SEPARATE GENERIC REVIEW (G)	NOT APPLICABLE (NA)	TOTAL
PALISADES	33	45	5	11	7	3	17	16	137
GINNA	24	41	4	27	2	5	17	17	137
DRESDEN-2	27	30	20	4	3	1	24	28	137
OYSTER CREEK	55	14	5	15	0	3	15	30	137
MILLSTONE-1	51	12	7	14	7	1	16	29	137
HADDAM NECK	42	34	12	7	1	5	17	19	137
SAN ONOFRE	30	17	23	22	1	3	22	19	137
YANKEE ROWE	46	16	23	9	3	0	22	18	137
DRESDEN-1	32	27	22	4	2	1	20	29	137
BIG ROCK POINT	42	25	8	10	4	1	14	33	137
LA CROSSE	48	20	8	9	3	2	13	34	137

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SAC CASE

1365 061

# SECTION II

# PALISADES

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: PALISADES

LICENSEE: CONSUMERS POWER

REGION/LOCATION: 5 MILES S OF SOUTH HAVEN, MI

DOCKET NO. 05000255

POWER CAPACITY: (THERMAL) 2530, (ELEC) 0805

OL NUMBER: DPR-20

N.S.S.S.: COMB

A/E FIRM: BECH

## KEY PERSONNEL

SYSTEMATIC EVALUATION P.M.: J. WETMORE

OPERATING REACTOR B.C.: D. ZIEMANN

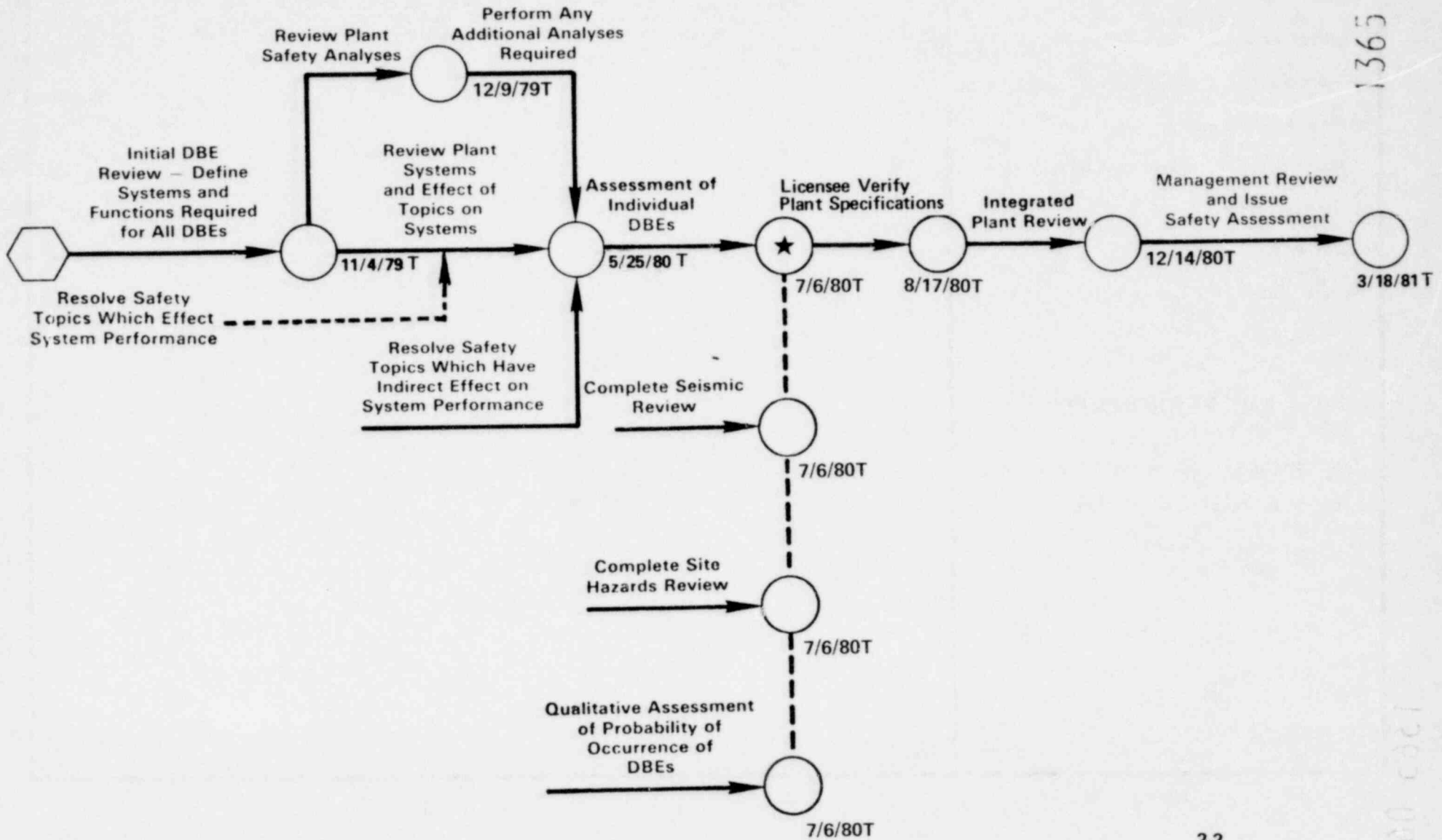
LIC. ASSISTANT: H. SMITH

## SUMMARY

NEW SCHEDULE FOR SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED DATE FOR THE PALISADES REVIEW HAS BEEN CHANGED FROM 1/81 TO 3/81.

1365 062

# SUMMARY SCHEDULE FALISADES



1365 063

A20 0001

# GINNA

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: GINNA

LICENSEE: ROCHESTER GAS & ELECTRIC

REGION/LOCATION: 15 MILES NE OF ROCHESTER, NY

DOCKET NO. 05000244

POWER CAPACITY: (THERMAL) 1520, (ELEC) 0490

OL NUMBER: DPR-18

N.S.S.S.: WEST

A/E FIRM: GIL

## KEY PERSONNEL

SYSTEMATIC EVALUATION P.M.: T. WAMBACH

OPERATING REACTOR B.C.: D. ZIEMANN

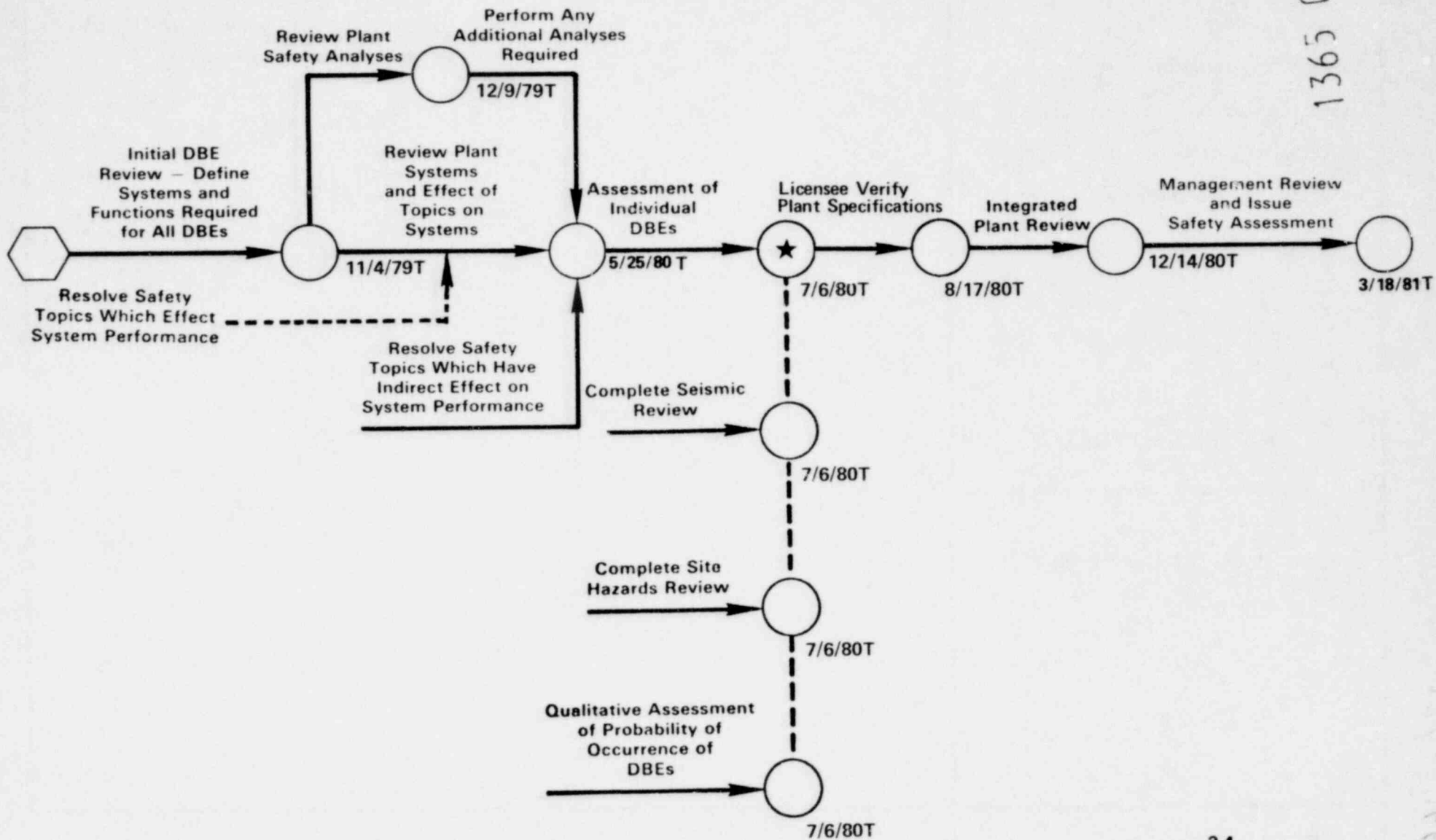
LIC. ASSISTANT: H. SMITH

## SUMMARY

NEW SCHEDULE FOR SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE GINNA REVIEW HAS BEEN CHANGED FROM 1/81 TO 3/81.

1365 064

# SUMMARY SCHEDULE GINNA



1365 065

# DRESDEN 2

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: DRESDEN 2

LICENSEE: COMMONWEALTH EDISON

REGION/LOCATION: 9 MILES E OF MORRIS, ILL

DOCKET NO. 05000237

POWER CAPACITY: THERMAL) 2527, (ELEC) 794

OL NUMBER: DPR-19

N.S.S.S.: GE

A/E FIRM: S&L

## KEY PERSONNEL

SYSTEMATIC EVALUATION PROGRAM MANAGER:

P. OCONNOR

OPERATING REACTOR B.C.: D. ZIEMANN

LIC. ASSISTANT: H. SMITH

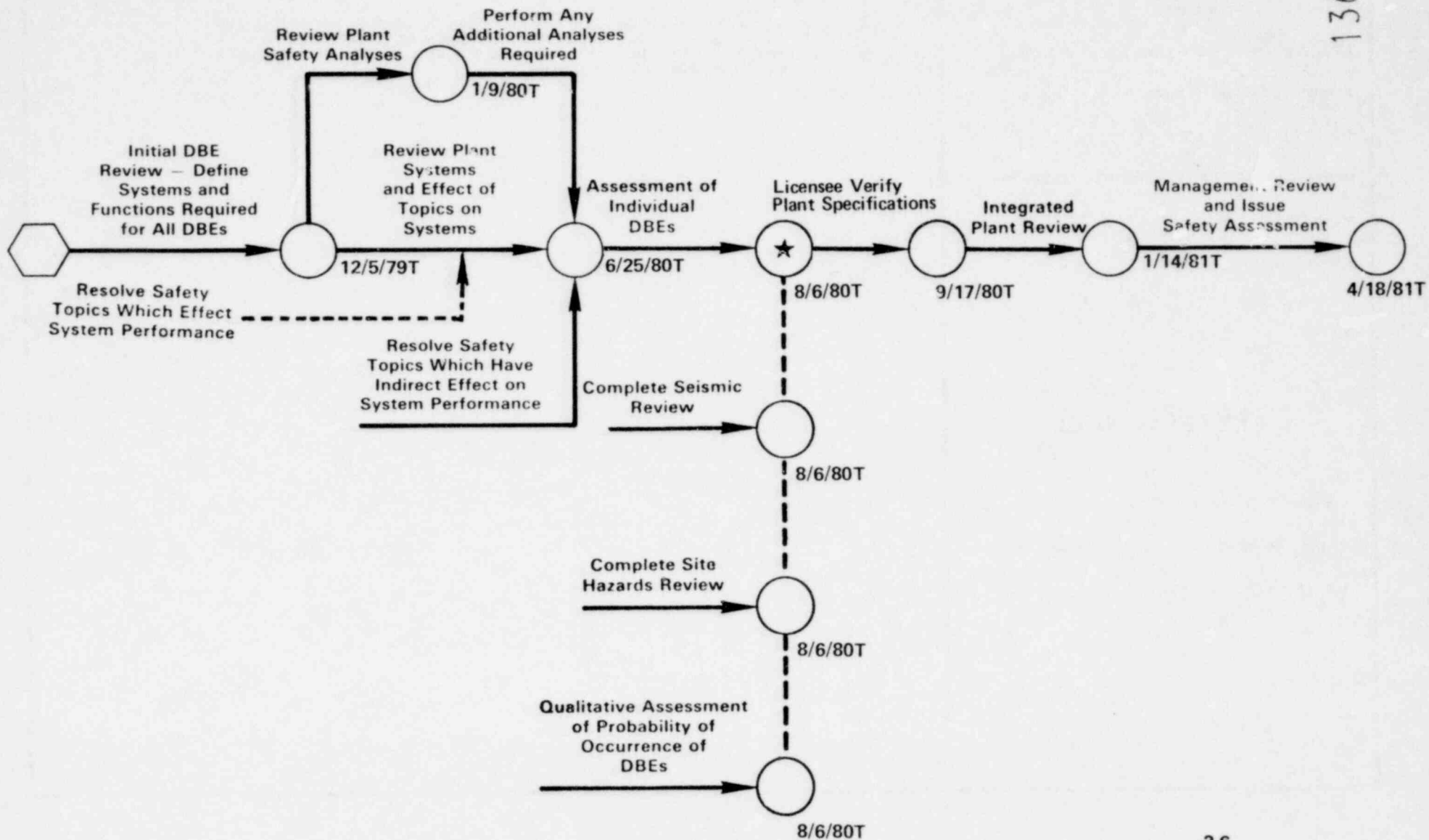
## SUMMARY

NEW SCHEDULES FOR THE COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE DRESDEN 2 REVIEW HAS BEEN CHANGED FROM 01/81 TO 04/81.



# SUMMARY SCHEDULE DRESDEN UNIT 2

1365 067



# OYSTER CREEK

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: OYSTER CREEK 1

LICENSEE: JERSEY CENTRAL POWER & LIGHT

REGION/LOCATION: 9 MILES S OF TOMS RIVER, NJ

DOCKET NO. 05000219

POWER CAPACITY: (THERMAL) 1930, (ELEC) 0650

OL NUMBER: DPR-16

N.S.S.S.: GE

A/E FIRM: B&B

## KEY PERSONNEL

SYSTEMATIC EVALUATION P.M.: T. WAMBACH

OPERATING REACTOR B.C.: D. ZIEMANN

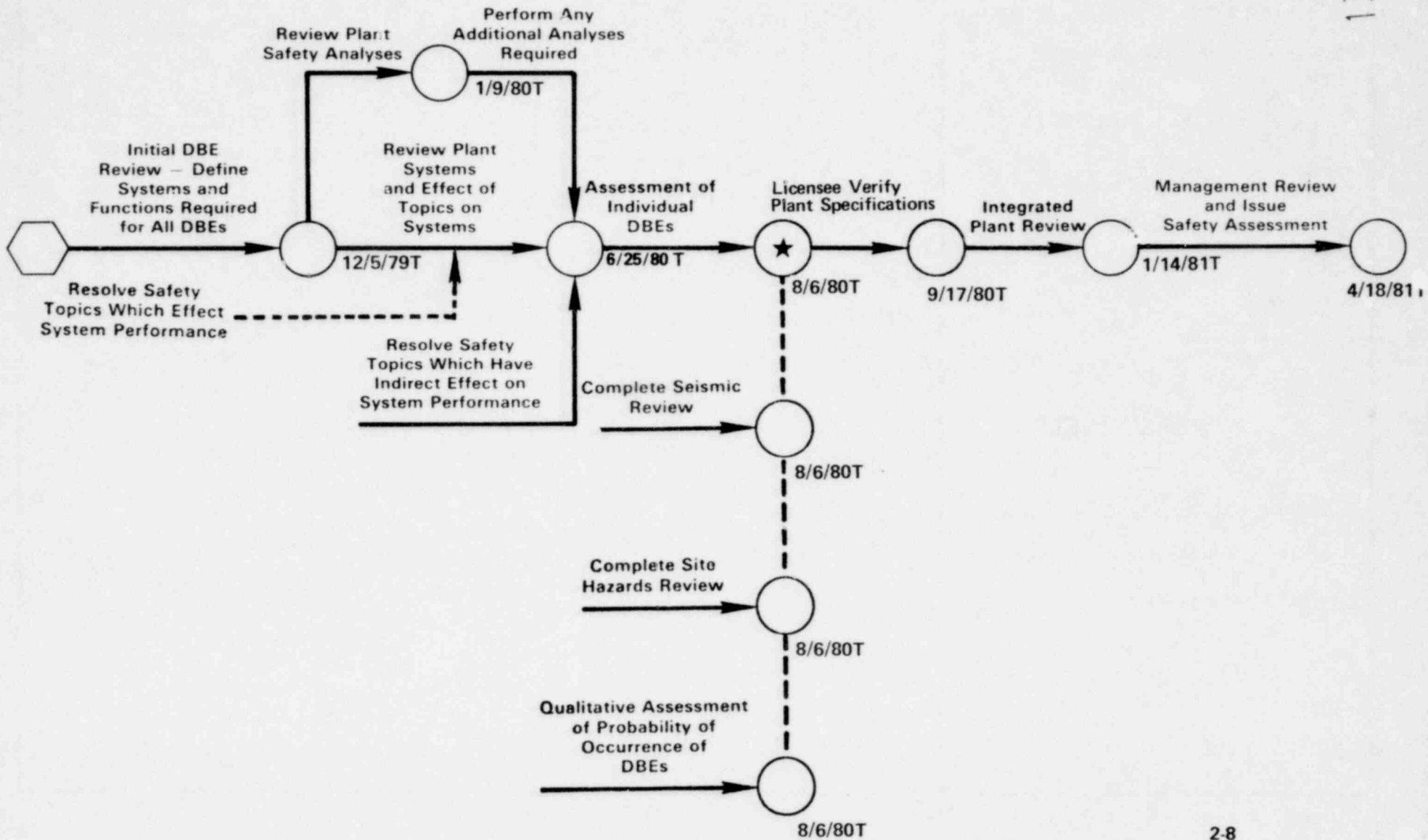
LIC. ASSISTANT: H. SMITH

## SUMMARY

NEW SCHEDULE FOR COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE OYSTER CREEK REVIEW HAS BEEN CHANGED FROM 1/81 TO 4/81.

# SUMMARY SCHEDULE OYSTER CREEK

1365 069



# MILLSTONE 1

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: MILLSTONE 1

LICENSEE: NORTHEAST NUCLEAR ENERGY

REGION/LOCATION:

5 MILES SW OF NEW LONDON, CONN

DOCKET NO. 05000245

POWER CAPACITY: (THERMAL) 2011, (ELEC) 0600

OL NUMBER: DPR-21

N.S.S.S.: GE

A/E FIRM: EBASCO

## KEY PERSONNEL

SYSTEMATIC EVALUATION P.M.: J. WETMORE

OPERATING REACTOR B.C.: D. ZIEMANN

LIC. ASSISTANT: H. SMITH

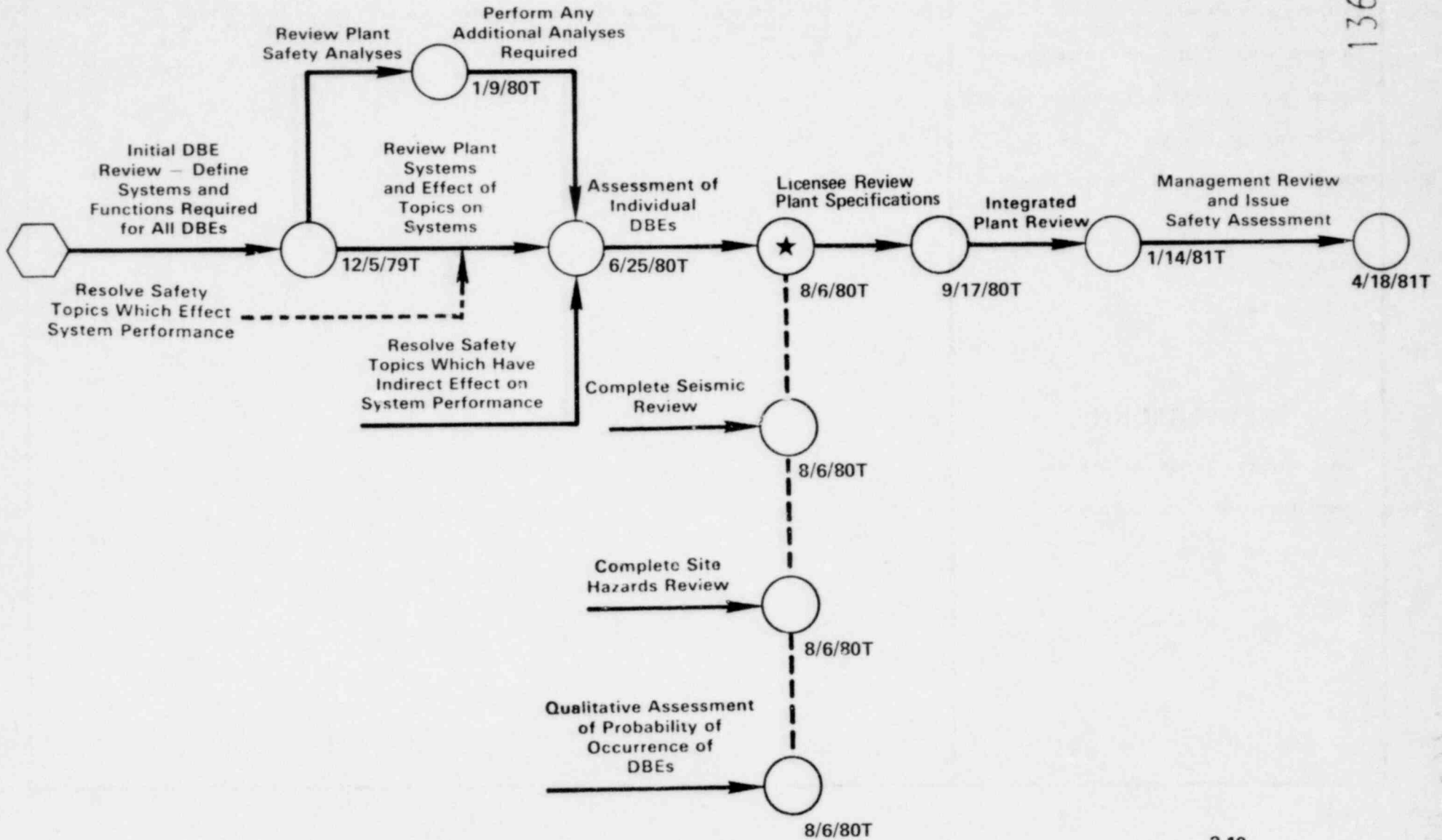
## SUMMARY

NEW SCHEDULES FOR COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE MILLSTONE 1 REVIEW HAS BEEN CHANGED FROM 1/81 TO 4/81.

1365 070

# SUMMARY SCHEDULE MILLSTONE UNIT 1

1365 071



100-015

# HADDAM NECK

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: HADDAM NECK

LICENSEE: CONNECTICUT YANKEE ATOMIC POWER

REGION/LOCATION: 13 MILES E OF MERIDEN, CT

DOCKET NO. 05000213

POWER CAPACITY: (THERMAL) 1825, (ELEC) 0575

OL NUMBER: DPR-61

N.S.S.S.: WEST

A/E FIRM: S&W

## KEY PERSONNEL

SYSTEMATIC EVALUATION P.M.: T. WAMBACI

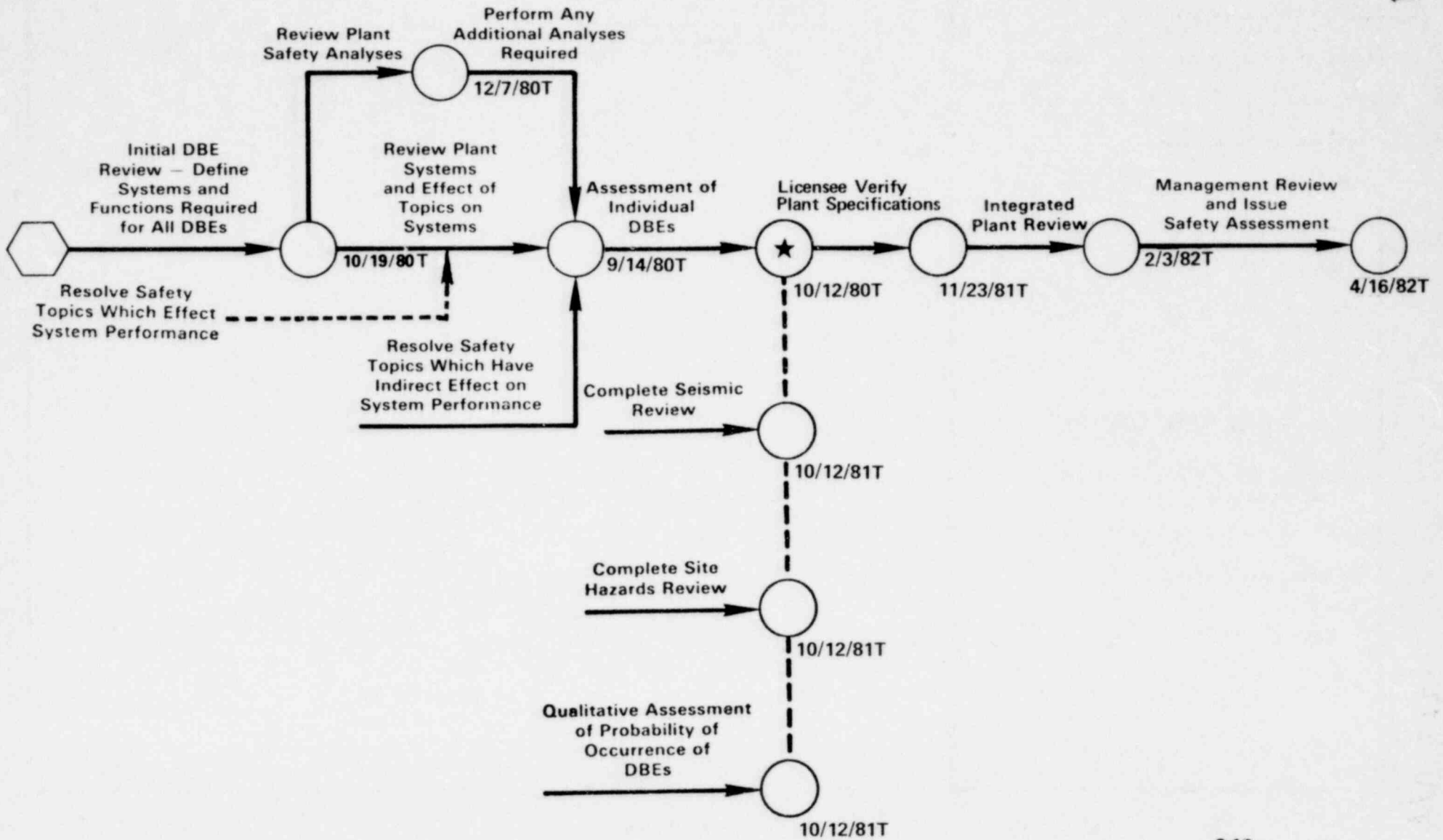
OPERATING REACTOR B.C.: D. ZIEMANN

LIC. ASSISTANT: H. SMITH

## SUMMARY

NEW SCHEDULES FOR COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE HADDAM NECK REVIEW HAS BEEN CHANGED FROM 1/81 TO 4/82.

# SUMMARY SCHEDULE HADDAM NECK



1265013

# SAN ONOFRE

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: SAN ONOFRE 1

LICENSEE: SOUTHERN CALIFORNIA EDISON

REGION/LOCATION: 5 MILES S OF SAN CLEMENTE, CA

DOCKET NO. 05000206

POWER CAPACITY: (THERMAL 1347, (ELEC) 0430

OL NUMBER: DPR-13

N.S.S.S.: WEST

A/E FIRM: BECH

## KEY PERSONNEL

SYSTEMATIC EVALUATION PROGRAM MANAGER:

P. W. OCONNOR

OPERATING REACTOR B.C.: D. ZIEMANN

LIC. ASSISTANT: H. SMITH

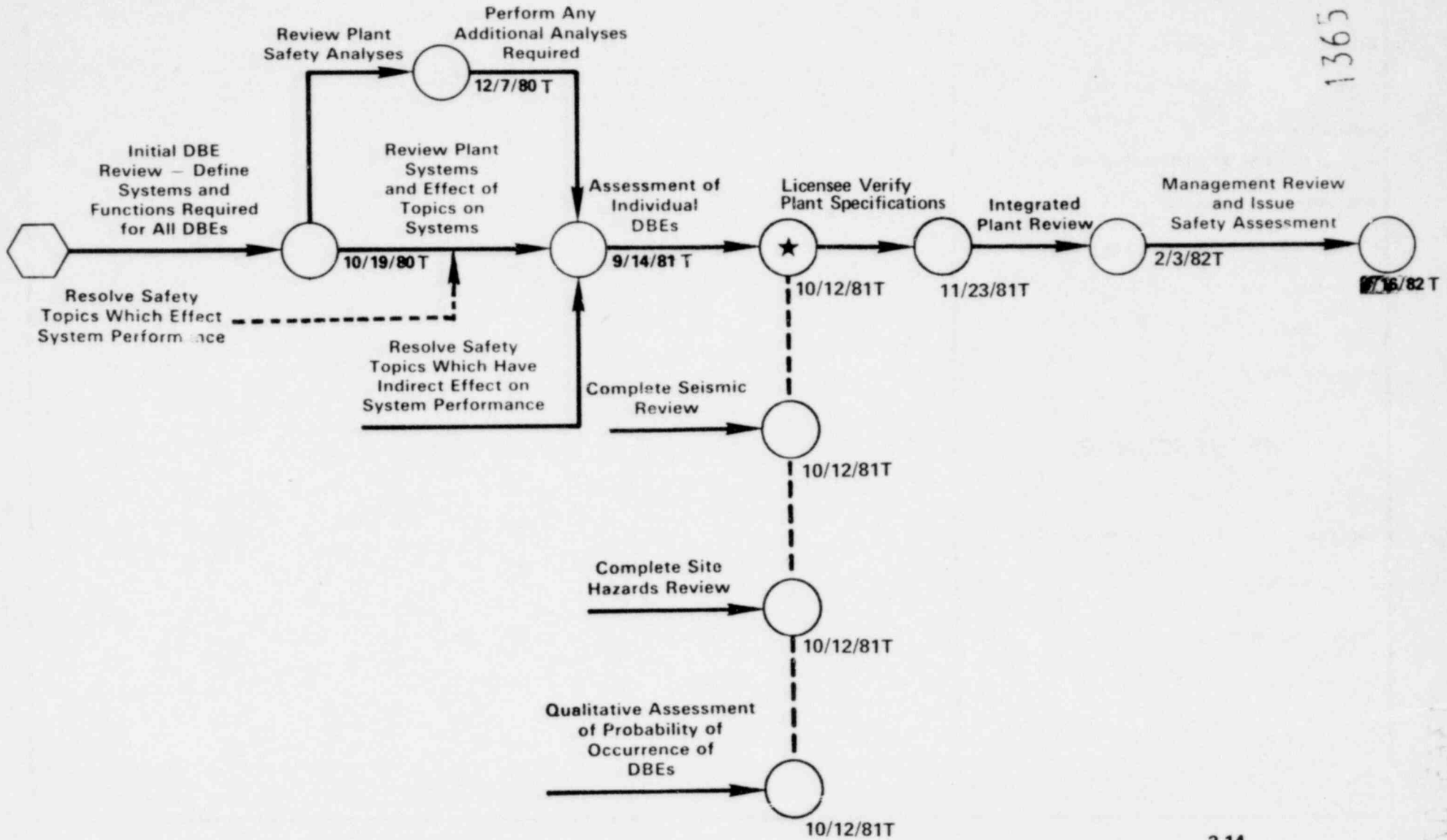
## SUMMARY

NEW SCHEDULES FOR COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE SAN ONOFRE REVIEW HAS BEEN CHANGED FROM 01/81 TO 04/82.



# SUMMARY SCHEDULE SAN ONOFRE

1365 075



# YANKEE ROWE

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: YANKEE ROWE 1

LICENSEE: YANKEE ATOMIC ELECTRIC

REGION/LOCATION:

25 MILES NE OF PITTSFIELD, MASS

DOCKET NO. 05000029

POWER CAPACITY: (THERMAL) 600, (ELEC) 175

OL NUMBER: DPR-3

N.S.S.S.: WEST

A/E FIRM: S&W

## KEY PERSONNEL

SYSTEMATIC EVALUATION PROGRAM MANAGER:

P.W. OCONNOR

OPERATING REACTOR B.C.: D, ZIEMANN

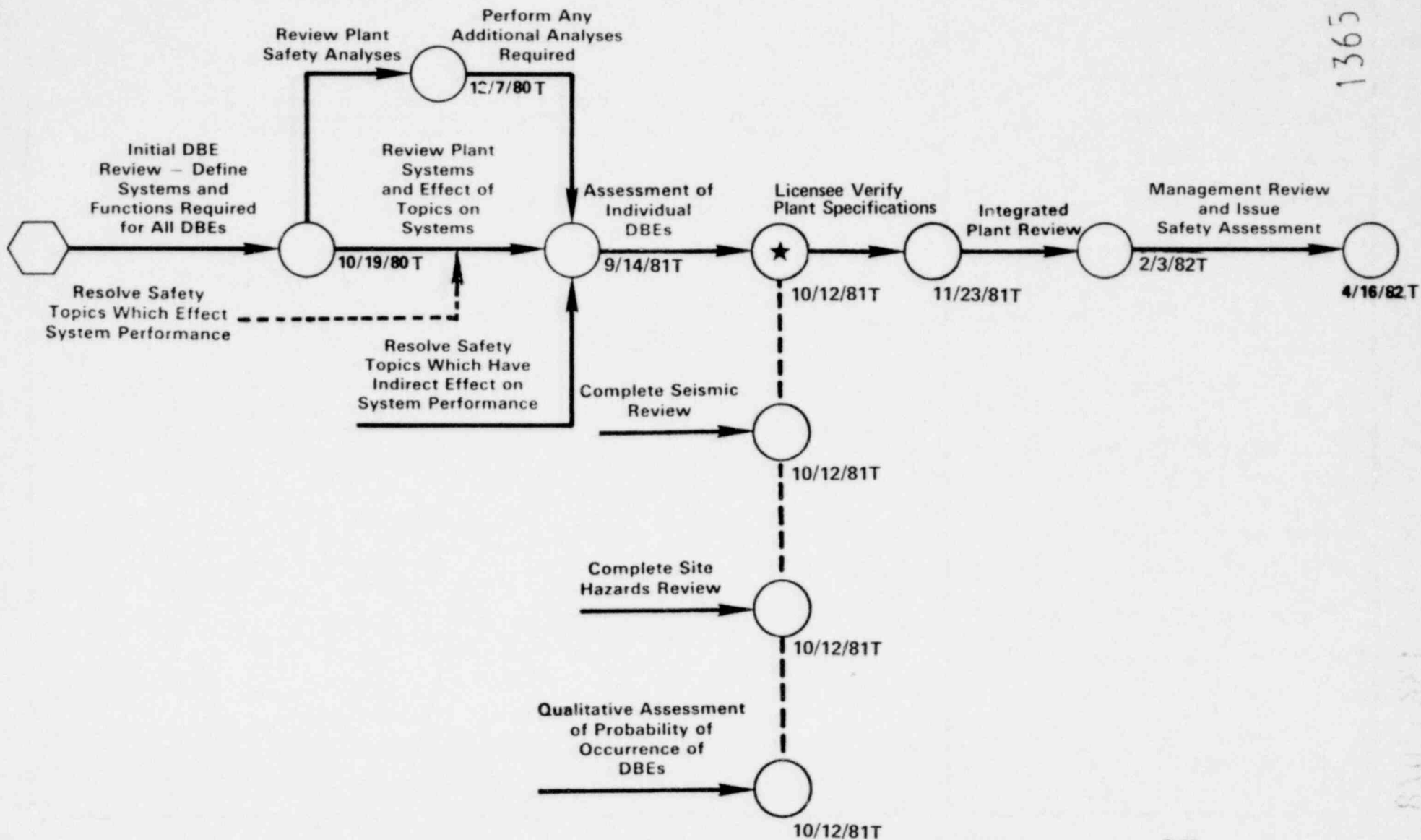
LIC. ASSISTANT: H. SMITH

## SUMMARY

NEW SCHEDULES FOR COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE YANKEE ROWE REVIEW HAS BEEN CHANGED FROM 01/81 TO 04/82.

1365 076

# SUMMARY SCHEDULE YANKEE ROWE



1365 077

8/11/81

# DRESDEN 1

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: DRESDEN 1

LICENSEE: COMMONWEALTH EDISON

REGION/LOCATION: 9 MILES E OF MORRIS, ILL

DOCKET NO. 05000010

POWER CAPACITY: (THERMAL) 700, (ELEC) 200

OL NUMBER: DPR-2

N.S.S.S.: GE

A/E FIRM: BECH

## KEY PERSONNEL

SYSTEMATIC EVALUATION PROGRAM MANAGER:

P. OCONNOR

OPERATING REACTOR B.C.: D. ZIEMANN

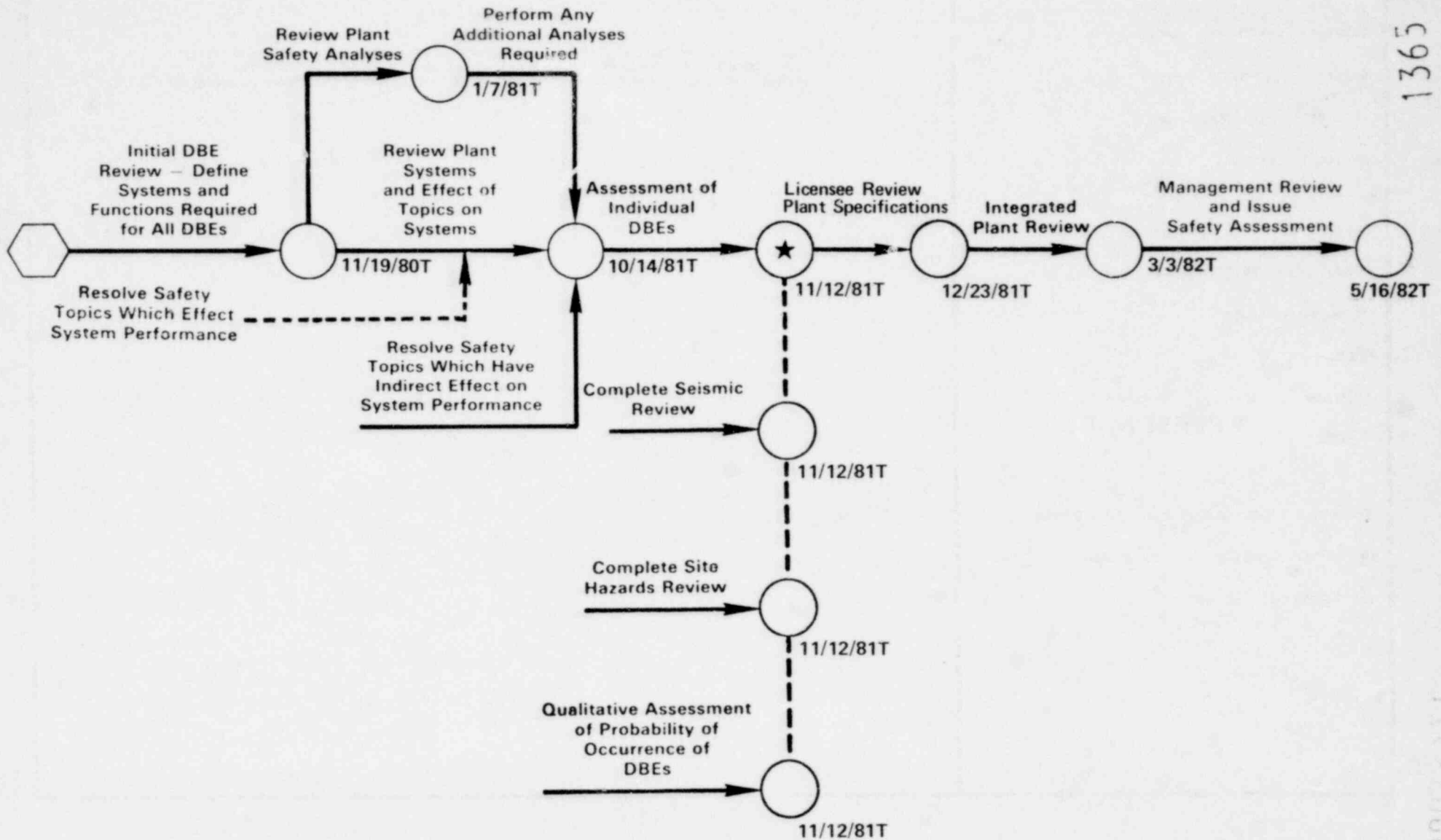
LIC. ASSISTANT: H. SMITH

## SUMMARY

NEW SCHEDULES FOR COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE DRESDEN 1 REVIEW HAS BEEN CHANGED FROM 01/81 TO 05/82.

1365 078

# SUMMARY SCHEDULE DRESDEN UNIT 1



1365 079

0807681

# BIG ROCK POINT

AS OF QUARTER ENDING SEPTEMBER 30, 1979

## FACILITY INFORMATION

FACILITY: BIG ROCK POINT 1

LICENSEE: CONSUMERS POWER

FACILITY: BIG ROCK POINT 1

LICENSEE: CONSUMERS POWER

REGION/LOCATICN:

4 MILES NE OF CHARLEVOIS, MICH.

DOCKET NO. 05000155

POWER CAPACITY: (THERMAL) 0240, (ELEC) 067

OL NUMBER: DPR-6

N.S.S.S.: GE

A/E FIRM: BECH

## KEY PERSONNEL

SYSTEMATIC EVALUATION P.M.: J. WETMORE

OPERATING REACTOR B.C.: D. ZIEMANN

LIC. ASSISTANT: H. SMITH

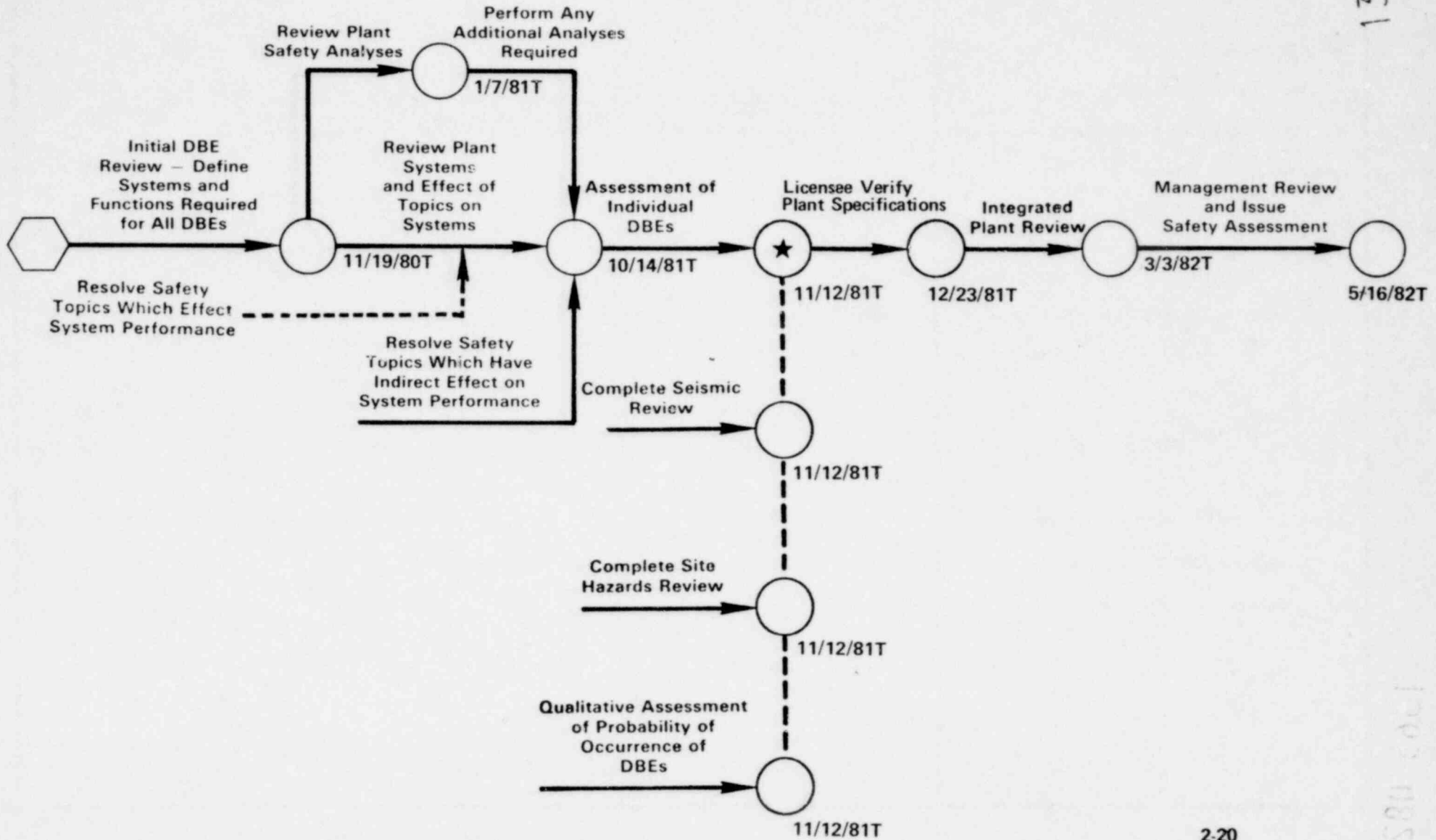
## SUMMARY

NEW SCHEDULES FOR COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE BIG ROCK POINT REVIEW HAS BEEN CHANGED FROM 1/81 TO 5/82.

1365 080

# SUMMARY SCHEDULE BIG ROCK POINT

1365 081



S80 6061

18  
**LACROSSE**

AS OF QUARTER ENDING SEPTEMBER 30, 1979

**FACILITY INFORMATION**

FACILITY: LACROSSE

LICENSEE: DAIRYLAND POWER

REGION/LOCATION: /19 MILES S OF LACROSSE, WISC

DOCKET NO. 05000409

POWER CAPACITY: (THERMAL) 0165, (ELEC) 0050

OL NUMBER: DPR-45

N.S.S.S.: AC

A/E FIRM: S&L

**KEY PERSONNEL**

SYSTEMATIC EVALUATION P.M.: J. WETMORE

OPERATING REACTOR B.C.: D. ZIEMANN

LIC. ASSISTANT: H. SMITH

**SUMMARY**

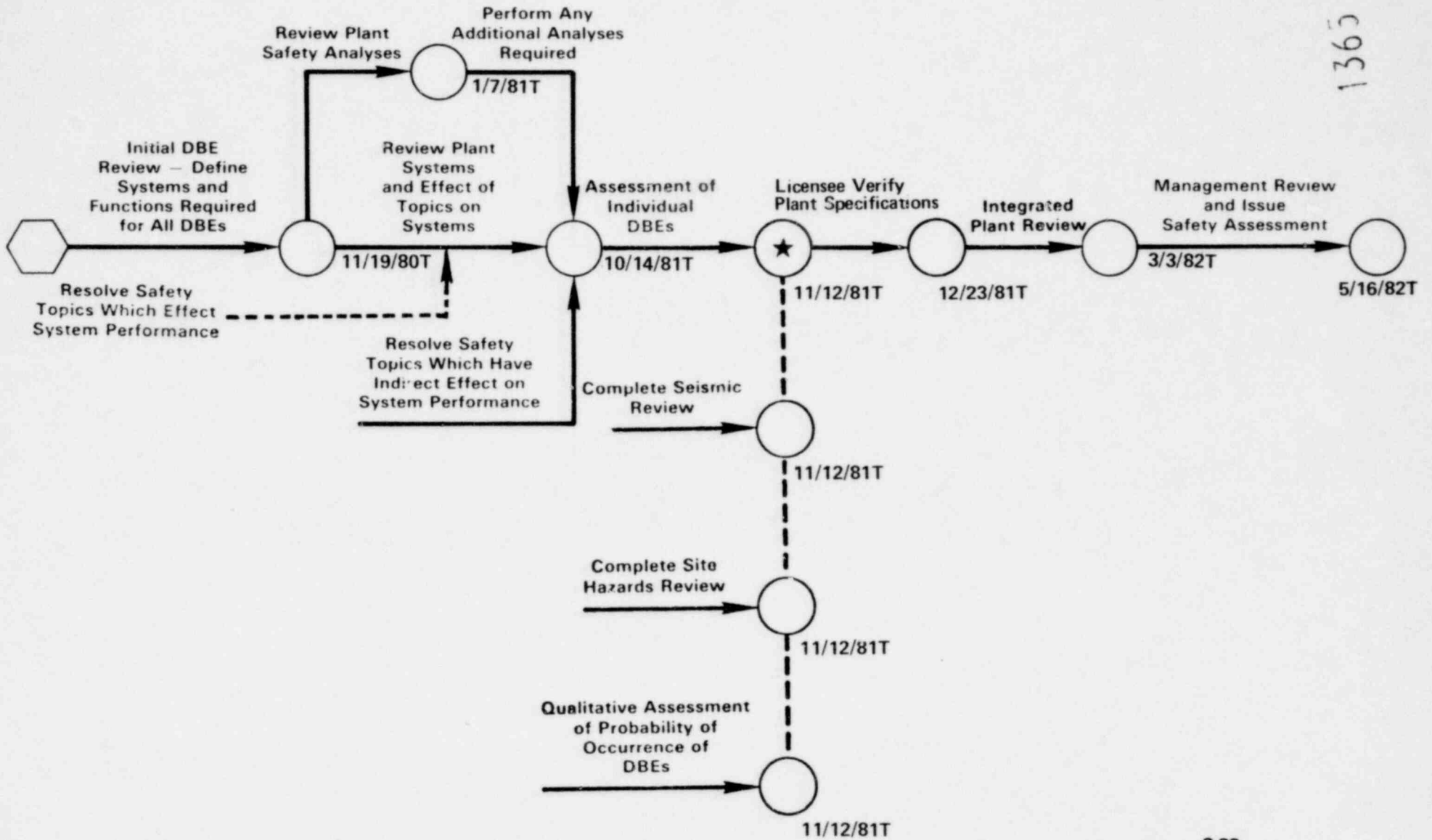
NEW SCHEDULES FOR COMPLETION OF SEP TOPICS HAVE BEEN ESTABLISHED TO REFLECT THE ASSIGNMENT OF DEDICATED REVIEWERS TO LONG TERM SEP ASSIGNMENTS. THE PROJECTED END DATE FOR THE LACROSSE REVIEW HAS BEEN CHANGED FROM 1/81 TO 5/82.

1365 082



# SUMMARY SCHEDULE LACROSSE

1365 083



1365 084

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