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DESCRIPTION: Ltr trans the following:

ENCLOSURES: Non-Routine 10-Day Report 75-07
Report of an unplanned release of radioactive materials occurring on 11-13-75....

(1 cy encl rec'd)

ACKNOWLEDGED

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PLANT NAME: Three Mile Island Unit 1

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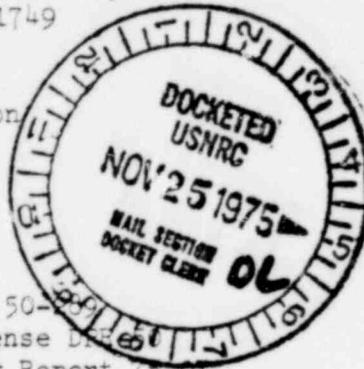
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TELEPHONE 215 - 929-3601

November 21, 1975
GQL 1749

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



Dear Sir:

Docket No. 50-249
Operating License Docket 60-01
Nonroutine 10-Day Report 75-07

In accordance with Section 6.7.2.a.2 of the Technical Specifications for our Three Mile Island Nuclear Station Unit 1 and 10 CFR 20.405 (a) (2) & (3), enclosed please find Nonroutine 10-Day Report 75-06 that deals with an unplanned release of radioactive material which occurred on November 13, 1975.

We would like to point out that no member of the public and no station or contractor personnel received a radiation dose in excess of the limits stated in 10 CFR 20 as a result of the release. Further, none of the limits given in the Technical Specifications were exceeded.

We trust that this satisfies the reporting requirements referenced above and adequately answers any concerns you may have.

Sincerely,

R. C. Arnold
Vice President

RCA:JMC:ilm

File: 20.1.1/7.7.3.12.1

Enclosure: Nonroutine 10-Day Report 75-07

cc: Mr. J. P. O'Reilly (U.S. NRC-Region 1)
Ms. Margaret Reilly (PaDER)

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NON-ROUTINE 10-DAY REPORT 75-07
REPORT OF AN UNPLANNED RELEASE OF RADIOACTIVE MATERIALS
OCCURRING ON 13 NOVEMBER 1975

11-21-75

Description of Occurrence

On November 13, 1975, Between the hours of 1838 and 2123 (two (2) hours and forty-five (45 minutes)) an inadvertant release of radioactive material occurred while draining the Reactor Coolant System from the pressurizer surge line to the C Reactor Coolant Bleed Tank. The reactor coolant in the pressurizer surge line was at a temperature of 260°F which, when drained to the bleed tank, flashed to steam resulting in an increase in pressure in the bleed tank gas space. The overpressure condition caused the over flow loop seal for the Miscellaneous Waste Storage and Reactor Coolant Bleed Tanks to partially evacuate and release radicactive gas to the Auxiliary Building sump. This gas then was released to the atmosphere by the Auxiliary Building Ventilation Exhaust System. The release was noted by Control Room personnel during routine checking of the Radiation Monitoring System recorders and the transfer of reactor coolant to the bleed tank was terminated. The overflow loop seal was refilled and the release was terminated.

Apparent Cause of the Occurrence

Lack of procedural guidance is the apparent cause of the occurrence in that the procedure restricted draining of the reactor coolant system to periods when the average temperature of the system was less than 150°F. Even though the average temperature at the point of draining was 110°F, the pressurizer surge line temperature was 260°F.

Analysis of the Occurrence

For the following reasons it is believed that the unplanned release of radioactive material on the 13th of November did not endanger either the health or safety of the public.

- a. None of the limits in the TMI-1 Technical Specifications were exceeded.
- b. None of the maximum permissible concentration limits for non-radiation workers as given in 10 CFR 20 were exceeded at the site boundary.
- c. No individual on site at the time of the release received a radiation dose in excess of the limits for radiation workers specified in 10 CFR 20.

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Corrective Action

Immediate corrective action as described above was taken to terminate the release. Additionally procedural changes will be incorporated to insure that draining of the reactor coolant system from the pressurizer surge line will be accomplished only when the pressurizer temperature is less than 150° F.

Failure Data

Not applicable.

Release Data

Note: In no case did any member of the public or any station or contractor personnel receive a radiation dose near the applicable limits given in 10 CFR 20.

The total release consisted of 6.88 curies of predominantly (>99%) Xe-133 based on Radiation Monitoring System Strip Chart Recordings and samples of the vent header system. The maximum instantaneous noble gas release rate during the two hour and forty-five minute period was 7.86×10^3 m³/sec which is below the Technical Specification limit of 1.2×10^5 m³/sec. The average release rate during the period was 2.31×10^3 m³/sec. The 24 hour average concentration in the auxiliary building sump was 3.374×10^{-4} µCi/cc (based on the design air flow of 500 CFM through the cubicle), which is reportable under Technical Specification 6.7.2.a.2 and 10 CFR 20. There were no personnel exposures involved during the release since there were no personnel in the auxiliary building sump, (a normally unoccupied area).

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