C 03/20 8

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL 50-289

REC: HARTFIELD R A
NRC

ORG: HERBEIN J G
METROPOL EDISON

DOCDATE 03/14/79
DATE RCVD 03/16/79

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:
COMPLETED REFUELING INFORMATION REQUEST FOR SUBJECT FACILITY LIC. NO DPR-50.

COPIES RECEIVED
LTR 1 ENCL 10

PLANT NAME: THREE MILE ISLAND - UNIT 1

REVIEWER INITIAL: XRS
DISTRIBUTOR INITIAL: ml

********** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS **********

GENERAL DISTRIBUTION FOR AFTER ISSUANCE OF OPERATING LICENSE.
(DISTRIBUTION CODE A001)

FOR ACTION: BR CHIEF [redacted]**W/7 ENCL

INTERNAL: REG FILE**W/ENCL
[redacted]
HANAUER**W/ENCL
EISENHUT**W/ENCL
BAER**W/ENCL
GRIMES**W/ENCL
J. MCGOUCH**W/ENCL
NRC PDR**W/ENCL
OELD**LTR ONLY
CHECK**W/ENCL
SHAO**W/ENCL
BUTLER**W/ENCL
J COLLINS**W/ENCL
MIPC w/2 ENCL

EXTERNAL: LPDR'S
HARRISBURG, PA**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

POOR ORIGINAL

1489 126

DISTRIBUTION: LTR 40 ENCL 39
SIZE: 1P+2P

CONTROL NBR: 7907302P2

********** THE END **********

7910240 820 P
1

METROPOLITAN EDISON COMPANY

POST OFFICE BOX 542 READING, PENNSYLVANIA 19603 TELEPHONE 215 – 929-3601

March 14, 1978
GQL 0429

Mr. R. A. Hartfield, Acting Director
Office of Management Information & Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit 1 (TMI-1)
Operating License No. DPR-50
Docket No. 50-289

Enclosed please find ten (10) copies of the completed Refueling Information Request for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

J. G. Herbein
Vice President-Generation

JGH:DGM:cjg

Enclosure

1489 127

780760292

A001/5 *
1/10

REFUELING INFORMATION REQUEST

1. Name of Facility: Three Mile Island Unit No. 1

2. Scheduled date for next refueling shutdown: March 17, 1978

3. Scheduled date for restart following refueling:

   May 15, 1978 for Reactor Criticality (tentative)

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   Yes

   If answer is yes, what, in general, will these be?

   The Technical Specification change for Cycle 4 will provide revised operational limits for rod withdrawal index and Power Imbalance. Also the Core Protection Safety Limits and the Protection System Maximum Allowable Setpoints for Reactor Power Imbalance have been slightly revised. The quadrant power tilt limit will be changed from a maximum actual core tilt of 3.41% to 4.92%.

   If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   N/A

   If no such review has taken place, when is it scheduled?

   N/A

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

   Technical Specification Change Request #70 was submitted to the NRC on 1/9/78.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   There are no important licensing considerations associated with Cycle 4 operation. TMI-1 will change from a rodded to a feed-bleed mode of operation for Cycle 4. This change is not regarded as a major change in the operating mode since TMI-1 was operated in essentially a rods-out configuration during the latter part of Cycle 1 and Cycle 3.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

   (a) 177 (b) 104 (start of 1978 refueling outage)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   Recently licensed for 752 fuel assembies storage locations in the A&B spent fuel pools. 496 location B pool racks to be installed starting in February 1978.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   1986 is last refueling discharge which allows full core off-load capacity (177 fuel assemblies).